



Carolina Power & Light Company
Harris Nuclear Plant
PO Box 165
New Hill NC 27562

DEC 3 1999

SERIAL: HNP-99-179

10 CFR 50.90

United States Nuclear Regulatory Commission
ATTENTION: Document Control Desk
Washington, DC 20555

SHEARON HARRIS NUCLEAR POWER PLANT
DOCKET NO. 50-400/LICENSE NO. NPF-63
RETYPE TECHNICAL SPECIFICATION PAGE FOR ADDITION OF METHODOLOGY
REFERENCES TO CORE OPERATING LIMITS REPORT

Dear Sir or Madam:

On August 4, 1999, Carolina Power & Light Company (CP&L) requested a License Amendment to revise the Harris Nuclear Plant (HNP) Technical Specification (TS) 6.9.1.6.2 related to "Methodology References to Core Operating Limits Report." As a result of a recently issued License Amendment, CP&L is submitting a retyped TS page which supersedes a page included in the August 4, 1999 submittal.

On October 19, 1999, the NRC issued License Amendment No. 92 to the HNP TS, which revised multiple TS pages as a result of changes to TS Section 6.5, "Review and Audit," TS 6.8.2, TS 6.8.3, and TS Section 6.10, "Record Retention." Issuance of Amendment 92 independently affected page 6-24c from the August 4, 1999 request, which is currently under NRC review. Enclosure 1 contains the retyped TS page which incorporates changes resulting from the issuance of Amendment No. 92.

Please refer any questions regarding this matter to Mr. J. H. Eads at (919) 362-2646.

Sincerely,

Donna B. Alexander
Manager, Regulatory Affairs
Harris Nuclear Plant

AEC/twk

Enclosure

- c: Mr. J. B. Brady (NRC Senior Resident Inspector, HNP)
 - Mr. R. J. Laufer (NRR Project Manager, HNP)
 - Mr. M. Fry (Director N.C. DEHNR)
 - Mr. L. A. Reyes (NRC Regional Administrator, Region II)
- 5413 Shearon Harris Road New Hill NC

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RETYPE TS PAGE

6-24c

ADMINISTRATIVE CONTROLS

6.9.1.6 CORE OPERATING LIMITS REPORT (Continued)

(Methodology for Specification 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, and 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor).

- o. EMF-96-029(A), Volume 1, Volume 2 and Attachment, "Reactor Analysis Systems for PWRs," Siemens Power Corporation, Richland WA 99352.

(Methodology for Specification 3.1.1.2 - SHUTDOWN MARGIN - MODES 3, 4 and 5, 3.1.1.3 - Moderator Temperature Coefficient, 3.1.3.5 - Shutdown Bank Insertion Limits, 3.1.3.6 - Control Bank Insertion Limits, 3.2.1 - Axial Flux Difference, 3.2.2 - Heat Flux Hot Channel Factor, 3.2.3 - Nuclear Enthalpy Rise Hot Channel Factor, and 3.9.1 - Boron Concentration.)

6.9.1.6.3 The core operating limits shall be determined so that all applicable limits (e.g., fuel thermal-mechanical limits, core thermal-hydraulic limits, nuclear limits such as shutdown margin, and transient and accident analysis limits) of the safety analysis are met.

6.9.1.6.4 The CORE OPERATING LIMITS REPORT, including any mid-cycle revisions or supplements, shall be provided, upon issuance for each reload cycle, to the NRC Document Control Desk, with copies to the Regional Administrator and Resident Inspector.

SPECIAL REPORTS

6.9.2 Special reports shall be submitted to the NRC in accordance with 10CFR50.4 within the time period specified for each report.

6.10 DELETED

(PAGE 6-25 DELETED)