

NOV 5 1980

MEMORANDUM FOR: File

THRU: E. C. McCabe, Jr., Chief, Reactor Projects Section No. 2

FROM: D. A. Beckman, Senior Resident Inspector, Reactor Projects Section No. 2, RO&NS Branch

SUBJECT: BEAVER VALLEY POWER STATION, UNIT 1 - MEMORANDUM OF TELECON: CONFERENCE CALL WITH NRR PERSONNEL REGARDING DRAFT IMMEDIATE ACTION LETTER FOR IE BULLETIN 79-14, CONDUCTED OCTOBER 22, 1980

A draft Immediate Action Letter was prepared on October 21, 1980 in response to Duquesne Light Company letters of August 14, September 11, and October 20, 1980 regarding IE Bulletin 79-14 status, extension of previous commitments for completion, and establishment of new commitments for dispositioning identified seismic qualification deficiencies. A draft copy of the Immediate Action Letter and the DLC letter of October 20 were telecopied to the Licensing Project Manager on October 22 to support this conference call. The conference call involved W. Ross (LPM), R. G. LaGrange (NRR:EB), M. Hartzman (NRR:EB), and myself.

The NRR staff members were consulted for informal advice regarding the licensee's positions on incomplete seismic reanalysis and evaluations on certain equipment nozzles, miscellaneous Seismic I Piping System deficiencies, material substitution reviews, evaluation of miscellaneous hangars, base plates, and pipe rack, and the acceptability of the licensee's proposal to utilize interim acceptance criteria (stresses less than allowable for DBE plus DL condition).

The call participants discussed the licensee's position on equipment nozzle load deficiencies with respect to the station's impending start up. Discussion involved clarification of the licensee's stated position with respect to the acceptance criteria used for both the preliminary and final evaluations of calculated nozzle loads. The NRR staff members questioned whether the nozzle load analyses accounted for thermal expansion loading, thermal anchor movement, and pressure stresses. They also questioned whether the load acceptance criteria for both the large and small bore nozzles had been changed from the original design basis acceptance criteria utilized for the Pipe Stress Reanalysis resulting from the Show Cause Orders of March and August 1979. In a subsequent conversation with the DLC Director of Nuclear Operations the inspector confirmed that the acceptance criteria for large bore nozzles is defined by the respective vendors as a combination of acceptable moments and loads. Small bore nozzle acceptance criteria, in some cases, is currently unavailable due to the loss of original design records by the facility AE. These small bore nozzles are being evaluated by the AE and will subsequently be forwarded to the individual equipment vendors for concurrence. The Director of Nuclear Operations informed the inspector that the acceptance criteria discussed above remains unchanged. As a result of these conversations the Immediate Action Letter will be revised to insure that the licensee will address the thermal and pressure loads as well as seismic loads.

①  
RO&NS  
Beckman  
lls

②  
RO&NS  
McCabe  
lls

③  
RO&NS  
Brunner  
lls

Item # 19

419

④

NOV 5 1980

The conference call participants also discussed the status of identified small bore piping deficiencies discussed by the licensee's October 20 letter. The most recent information available from DLC indicates that most or all of these deficiencies will be corrected by modifications prior to reactor criticality. Based on the licensee's statement that all modifications to systems required to prevent or mitigate design basis accidents will be completed prior to reactor start up and that the failure of the remaining deficient but unmodified conditions would not impair the ability to place the plant in cold shut-down, the Licensing Project Manager stated that NRR had no objection to facility re-start. The licensee's schedule for completing the remaining IEB tasks was also discussed and was considered acceptable subject to the additional comments below regarding the identification of new additional seismic qualification deficiencies.

The conference participants discussed the licensee's proposed use of the "interim criteria" discussed above and considered that extended use of such criteria until the Cycle II-III refueling outage was unacceptable (the next refueling outage will occur no earlier than Spring 1982). The NRR Engineering Branch personnel also discussed the possibility of modifying the licensee's proposal to utilize the interim criteria to require acceptability of piping or components under OBE plus DL or DBE plus DL conditions whichever is greater. This matter is under review by the NRR staff and their conclusions will be forwarded to the SRI on or before October 24, 1980 for possible inclusion in the IAL. The use of an acceptable interim criteria is desirable to avoid unnecessary emergency Technical Specification relief request reviews for minor deficiencies identified during the completion of the bulletin evaluations or unnecessary reactor shut-downs because of such minor deficiencies. The conference call participants agreed that imposition of the licensee's proposed interim criteria appeared acceptable for the nozzle and small bore piping deficiencies previously identified by prior licensee IEB 79-14 correspondence.

The NRR staff members were requested to provide any additional input or comments to the SRI no later than Friday, October 24, 1980 to support timely issuance of the IAL.

D. A. Beckman  
Senior Resident Inspector

cc:  
E. McCabe  
D. Beckman  
W. Ross, NRR