

December 6, 1999

Mr. Michael Roche
Vice President and Director
GPU Nuclear, Inc.
Oyster Creek Nuclear Generating Station
P. O. Box 388
Forked River, NJ 08731

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON YOUR PROPOSED LICENSE AMENDMENT CONCERNING THE REACTOR BUILDING CRANE, OYSTER CREEK NUCLEAR GENERATING STATION (TAC NO. MA5663)

Dear Mr. Roche:

By letter dated April 28, 1999, you proposed an amendment that would modify the Oyster Creek Nuclear Generating Station Operating License No. DPR-16, to request approval to handle loads up to and including 45 tons using the reactor building crane during power operations.

In a telephone conference on October 4, 1999, we clarified the enclosed questions and discussed the schedule for responding with your staff. The Nuclear Regulatory Commission needs additional information to complete its review. Please respond to the enclosed request within 30 days of the date of this letter.

If you have any questions regarding this correspondence, please contact me at (301) 415-1261.

Sincerely,
ORIGINAL SIGNED BY:
Helen N. Pastis, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosure: Request for Additional Information

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

A handwritten signature in cursive script, appearing to read "Helen N. Pastis".

Helen N. Pastis, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-219

Enclosure: Request for Additional Information

cc w/encl: See next page

M. Roche
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Kent Tosch, Chief
New Jersey Department of
Environmental Protection
Bureau of Nuclear Engineering
CN 415
Trenton, NJ 08625

REQUEST FOR ADDITIONAL INFORMATION

RELATED TO

GPU NUCLEAR APRIL 28, 1999 LICENSE AMENDMENT REQUEST

FOR MOVEMENT OF LOADS UP TO 45 TONS WITH

THE REACTOR BUILDING CRANE DURING POWER OPERATIONS

AT THE OYSTER CREEK NUCLEAR GENERATING STATION

1. As acknowledged by the licensee, the proposed amendment to move heavy loads up to 45 tons presents a potential load drop accident that has not previously been evaluated in the FSAR (Final Safety Analysis Report). Furthermore, a heavy load drop accident while traversing the 119' elevation of the Reactor Building may create the possibility of an accident and consequences different than those previously evaluated in the FSAR.

Describe the potential accident consequences of a load drop of a 45-ton load traversing the 119' elevation, as it relates to damage to safety-related equipment or equipment that may be required to achieve safe shutdown or continued decay heat removal. Describe the potential accident consequences of a load drop of a 45-ton load traversing the 119' elevation, as it relates to damage to the spent fuel pool (including water leakage and exposure of the fuel in the spent fuel pool), damage to the fuel, and release of radioactive material. Also address what measures are to be implemented by the licensee to mitigate the potential consequences of a dropped load of 45 tons?

2. What is the basis for the statement that the probability of a load drop with the Oyster Creek Reactor Building crane is lower than the previous NUREG-0612 estimate of 2.7×10^{-5} per lift?