



**UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001**

INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-315

DONALD C. COOK NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

**Amendment No. 232.
License No. DPR-58**

- 1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:**
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated October 8, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;**
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;**
 - C. There is reasonable assurance that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;**
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and**
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.**

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-58 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 232, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: November 30, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 232

TO FACILITY OPERATING LICENSE NO. DPR-58

DOCKET NO. 50-315

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

3/4 3-54

B 3/4 3-6

INSERT

3/4 3-54

B 3/4 3-6

3/4 LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS
3/4.3 INSTRUMENTATION

POST-ACCIDENT INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.8 The post-accident monitoring instrumentation channels shown in Table 3.3-11 shall be **OPERABLE**.

APPLICABILITY: MODES 1, 2, and 3.

ACTION:

- a. With the number of **OPERABLE** post-accident monitoring channels less than required by Table 3.3-11 (except item 8), either restore the inoperable channel to **OPERABLE** status within 30 days, or be in **HOT SHUTDOWN** within the next 12 hours.
- b. With the number of **OPERABLE** post-accident monitoring channels one less than required by Table 3.3-11, item 8, Refueling Water Storage Tank Water Level:
 1. Either restore the inoperable channel to **OPERABLE** status within 72 hours or be in at least **HOT SHUTDOWN** within the next 12 hours, and
 2. Within one hour, bypass the Residual Heat Removal Pump trip function from the Refueling Water Storage Tank Water Level for the pump associated with the out-of-service instrument.
- c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.8 Each post-accident monitoring instrumentation channel shall be demonstrated **OPERABLE** by performance of the **CHANNEL CHECK** and **CHANNEL CALIBRATION** operations at the frequencies shown in Table 4.3-7.

3/4 BASES
3/4.3 INSTRUMENTATION

3/4.3.3.5 REMOTE SHUTDOWN INSTRUMENTATION

The OPERABILITY of the remote shutdown instrumentation ensures that sufficient capability is available to permit shutdown and maintenance of HOT STANDBY of the facility from locations outside of the control room. This capability is required in the event control room habitability is lost and is consistent with General Design Criteria 19 of 10 CFR 50.

3/4.3.3.5.1 APPENDIX R REMOTE SHUTDOWN INSTRUMENTATION

The OPERABILITY of the Appendix R remote shutdown instrumentation ensures that sufficient instrumentation is available to permit shutdown of the facility to COLD SHUTDOWN conditions at the local shutdown indication (LSI) panel. In the event of a fire, normal power to the LSI panels may be lost. As a result, capability to repair the LSI panels from Unit 2 has been provided. If the alternate power supply is not available, fire watches will be established in those fire areas where loss of normal power to the LSI panels could occur in the event of fire. This will consist of either establishing continuous fire watches or verifying OPERABILITY of fire detectors per Specification 4.3.3.7 and establishing hourly fire watches. The details of how these fire watches are to be implemented are included in a plant procedure.

3/4.3.3.8 POST-ACCIDENT INSTRUMENTATION

The OPERABILITY of the post-accident instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. The allowable out-of-service time for the Refueling Water Storage Tank (RWST) level channels is required to provide the overall reliability to support the manual transfer from injection to recirculation following an accident. The bypassing of the Residual Heat Removal (RHR) pump trip from the RWST low level, with a level channel out-of-service, ensures that RHR pump will be available to meet its Engineered Safety Features (ESF) Function of injecting water into the core. The loss of the RHR pump protection will be mitigated by the operator's action to switch from injection to recirculation using the approved Emergency Operating Procedure which causes the RHR pump suction to be realigned well before the RHR pump trip setpoint. The associated RHR pump can be considered OPERABLE with the RWST level channel out-of-service once the trip function has been by-passed since the pump would be available to fulfill its ESF function.



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INDIANA MICHIGAN POWER COMPANY

DOCKET NO. 50-316

DONALD C. COOK NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

**Amendment No. 215
License No. DPR-74**

- 1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:**
 - A. The application for amendment by Indiana Michigan Power Company (the licensee) dated October 8, 1998, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;**
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;**
 - C. There is reasonable assurance that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;**
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and**
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.**

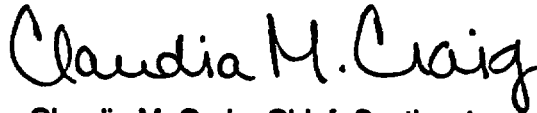
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-74 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 215, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance and shall be implemented within 30 days.

FOR THE NUCLEAR REGULATORY COMMISSION



Claudia M. Craig, Chief, Section 1
Project Directorate III
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: November 30, 1999

ATTACHMENT TO LICENSE AMENDMENT NO. 215

FACILITY OPERATING LICENSE NO. DPR-74

DOCKET NO. 50-316

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

3/4 3-45

B 3/4 3-3

INSERT

3/4 3-45

B 3/4 3-3

3/4 LIMITING CONDITIONS FOR OPERATION AND SURVEILLANCE REQUIREMENTS
3/4.3 INSTRUMENTATION

POST-ACCIDENT INSTRUMENTATION

LIMITING CONDITION FOR OPERATION

3.3.3.6 The post-accident monitoring instrumentation channels shown in Table 3.3-10 shall be **OPERABLE**.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

- a. With the number of **OPERABLE** post-accident monitoring channels less than required by Table 3.3-10 (except item 8), either restore the inoperable channel to **OPERABLE** status within 30 days, or be in **HOT SHUTDOWN** within the next 12 hours.
- b. With the number of **OPERABLE** post-accident monitoring channels one less than required by Table 3.3-10, item 8, Refueling Water Storage Tank Water Level:
 1. Either restore the inoperable channel to **OPERABLE** status within 72 hours or be in at least **HOT SHUTDOWN** within the next 12 hours, and
 2. Within one hour, bypass the Residual Heat Removal Pump trip function from the Refueling Water Storage Tank Water Level for the pump associated with the out-of-service instrument.
- c. The provisions of Specification 3.0.4 are not applicable.

SURVEILLANCE REQUIREMENTS

4.3.3.6 Each post-accident monitoring instrumentation channel shall be demonstrated **OPERABLE** by performance of the **CHANNEL CHECK** and **CHANNEL CALIBRATION** operations at the frequencies shown in Table 4.3-10.

3/4 BASES
3/4.3 INSTRUMENTATION

3/4.3.3.6 POST-ACCIDENT INSTRUMENTATION

The OPERABILITY of the post-accident instrumentation ensures that sufficient information is available on selected plant parameters to monitor and assess these variables during and following an accident. The allowable out-of-service time for the Refueling Water Storage Tank (RWST) level channels is required to provide the overall reliability to support the manual transfer from injection to recirculation following an accident. The bypassing of the Residual Heat Removal (RHR) pump trip from the RWST low level, with a level channel out-of-service, ensures that the RHR pump will be available to meet its Engineered Safety Features (ESF) Function of injecting water into the core. The loss of RHR pump protection will be mitigated by the operator's action to switch from injection to recirculation using the approved Emergency Operating Procedure which causes the RHR pump suction to be realigned well before the RHR pump trip setpoint. The associated RHR pump can be considered OPERABLE with the RWST level channel out-of-service once the trip function has been by-passed since the pump would be available to fulfill its ESF function.

3/4.3.3.7 Deleted.

3/4.3.3.9 EXPLOSIVE GAS MONITORING INSTRUMENTATION

This instrumentation includes provisions for monitoring the concentrations of potentially explosive gas mixtures in the Waste Gas Holdup System. The OPERABILITY and use of this instrumentation is consistent with the requirements of General Design Criteria specified in Section 11.3 of the Final Safety Analysis Report for the Donald C. Cook Nuclear Plant.