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Detroit Edison



November 29, 1999
NRC-99-0099

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington D C 20555-0001

- References:
- 1) Fermi 2
NRC Docket No. 50-341
NRC License No. NPF-43
 - 2) NRC Generic Letter 96-06, "Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions", dated September 30, 1996
 - 3) Detroit Edison Letter to NRC, "Detroit Edison 30-Day Response to NRC Generic Letter 96-06", NRC-96-0118, dated October 30, 1996
 - 4) Detroit Edison Letter to NRC, "Detroit Edison 120-Day Response to NRC Generic Letter 96-06", NRC-97-0003, dated January 28, 1997
 - 5) NRC Letter to Detroit Edison, "Request for Additional Information Related to the Fermi-2 Response to Generic Letter 96-06", dated September 9, 1997
 - 6) Detroit Edison Letter to NRC, "Additional Information Related to Detroit Edison Response to NRC Generic Letter 96-06", NRC-97-0111, dated October 17, 1997
 - 7) Detroit Edison Letter to NRC, "Update of Detroit Edison Response to NRC Generic Letter 98-06", NRC-98-0058, dated March 27, 1998

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- 8) NRC Letter to Detroit Edison, "Request for Additional Information (RAI) Related to the Fermi-2 Response to Generic Letter 96-06, Assurance of Equipment Operability and Containment Integrity During Design-Basis Accident Conditions", dated April 6, 1998
- 9) Detroit Edison Letter to NRC, "Additional Information Related to Detroit Edison Response to NRC Generic Letter 96-06", NRC-98-0094, dated June 30, 1998
- 10) NRC Letter to Detroit Edison, "Request for Additional Information Related to Fermi 2 Response to Generic Letter 96-06", dated May 13, 1999
- 11) Detroit Edison Letter to NRC, "Response to NRC Request for Additional Information Related to Detroit Edison Response to NRC Generic Letter 96-06", NRC-99-0047, dated June 9, 1999

Subject: Additional Information Related to Penetration Re-evaluation in Response to NRC Generic Letter 96-06 (TAC No. M96811)

The purpose of this letter is to update the NRC about Detroit Edison's additional evaluation of three of the six drywell penetrations that were identified as susceptible to thermally induced pressurization in Reference 6.

In Reference 11, Detroit Edison indicated that penetrations X-8, X-49a and X-51a will be re-evaluated to determine an appropriate resolution to the thermally induced pressurization concern discussed in Reference 2.

Thermal-hydraulic analyses of the piping systems were performed to calculate the temperature and pressure of water in the piping following a LOCA. Conservative assumptions regarding the heat transfer coefficients as recommended in NUREG-0588 were used in the analyses. The containment isolation valves were assumed to be leak-tight. Internal pressure response resulting from the thermal-hydraulic analysis in each penetration was combined with other applicable loads to perform finite element stress analyses on models of the penetrations and attached piping systems. The stress analyses results demonstrate that maximum stresses in penetrations X-8, X-49a and X-51a satisfy the acceptance criteria of Appendix F to Section III of the ASME Code. Therefore, no modifications are required for these three penetrations.

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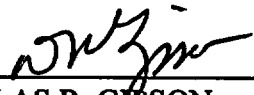
Should you have any questions or require additional information, please contact Mr. Norman K. Peterson of my staff at (734) 586-4258.

Sincerely,



cc: A. J. Kugler
A. Vogel
NRC Resident Office
Regional Administrator, Region III
Supervisor, Electric Operators,
Michigan Public Service Commission

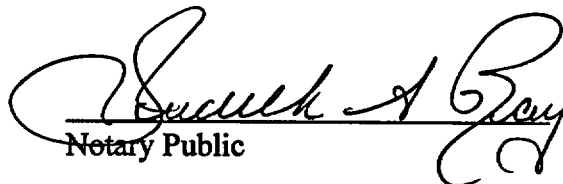
I, DOUGLAS R. GIPSON, do hereby affirm that the foregoing statements are based on facts and circumstances which are true and accurate to the best of my knowledge and belief.



DOUGLAS R. GIPSON
Senior Vice President, Nuclear Generation

On this 29th day of NOVEMBER, 1999 before me personally appeared Douglas R. Gipson, being first duly sworn and says that he executed the foregoing as his free act and deed.

JUDITH A. RAY
Notary Public, Wayne County, MI
My Commission Expires Apr 4 2002



Notary Public

