ATTACHMENT

GENERIC ISSUE MANAGEMENT CONTROL SYSTEM REPORT

OFFICE OF NUCLEAR REGULATORY RESEARCH

NOVEMBER 1999

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GENERIC ISSUE MANAGEMENT CONTROL SYSTEM

DESCRIPTION

The Generic Issue Management Control System (GIMCS) provides information necessary to manage the resolution of generic safety issues (GSIs) as well as non-safety-related generic issues. GSIs have the potential for safety enhancements and the promulgation of new or revised requirements or guidance. For the purpose of this management control system, resolution of a reactor GSI is defined as the point when a close-out memorandum is issued by the lead office to the EDO summarizing the staff's findings and conclusion. This conclusion can either be: (1) no new requirements ; or (2) new requirements, with incorporation of the resolution into one or more of the following documents:

(a) Commission Order	(e) Regulatory Guide
(b) NRC Policy Statement	(f) Generic Letter
(c) Rule	(g) Bulletin
(d) Standard Review Plan (SRP)	(h) Information Notice

For non-safety-related reactor issues and all non-reactor issues, resolution is defined as the point when a close-out memorandum is issued by the lead office documenting the staff's findings and conclusion.

GIMCS is part of an integrated system of reports and procedures that is designed to manage GSIs through the stages of prioritization and resolution (development of new criteria, management review and approval, public comments, and incorporation into the regulations, as appropriate). The priority evaluation for each issue listed in this report is contained in NUREG-0933, "A Prioritization of Generic Safety Issues." For reactor issues, the "Procedures for Identification, Prioritization, Resolution, and Tracking of Generic Issues" are outlined in RES Office Letter No. 7, dated February 16, 1996. The procedures for processing non-reactor issues are documented in NMSS Policy and Procedures Letter 1-57, Revision 1, "NMSS Generic Issues Program," dated October 1997.

GIMCS provides the proposed schedules for managing the resolution of: (1) GSIs that have a HIGH-priority; (2) GSIs that have a MEDIUM-priority; and (3) other issues designated to receive resources for resolution. Reactor GSIs ranked as either LOW or DROP are not allocated resources for resolution and, therefore, are not tracked in GIMCS.

DATA ELEMENTS

Management and control indicators used in GIMCS are defined as follows:

1. Issue No. - Generic Issue Number

2. Title - Generic Issue Title

3. Identification Date - Date the issue was identified

4. Prioritization Date - The date that the prioritization evaluation was approved by the RES Director

5. Type - Generic Safety (GSI), Licensing (LI), or Regulatory Impact (RI)

6. Priority - High (H) or Medium (M)

7. Task Manager - Name of assigned individual responsible for resolution

8. Office/Division/Branch - The Office, Division, and Branch of the Task Manager who has lead responsibility for resolving the issue

9. Action Level
   - Active - Technical assistance funds appropriated for resolution and/or Task Manager actively pursuing resolution
   - Inactive - No technical assistance funds appropriated for resolution, Task Manager assigned to more important work, or no Task Manager assigned
   - Resolved - All necessary work has been completed and no additional resources will be expended

10. Status - Coded summary as follows:
    - NR - (Nearly-Resolved);
    - 3A - (Resolved with requirements);
    - 3B - (Resolved with No requirements);
    - 5 - (Licensing or Regulatory Impact issue that has been assigned resources for completion)

11. TAC Number - Task Action Control (TAC) number assigned to the issue

TABLE 1
REACTOR GSIs SCHEDULED FOR RESOLUTION

ISSUE NUMBER	TITLE	LEAD/OFFICE/ DIVISION/ BRANCH	PRIORITY	DATE APPROVED FOR RESOLUTION	RESOLUTION DATE AT END OF FY-98	CURRENT RESOLUTION DATE
23	Reactor Coolant Pump Seal Failures	RES/DET/ERAB	HIGH	NRR-OP FY83	03/1999	12/1999
145	Actions to Reduce Common Failures	RES/DSARE/REAHFB	HIGH*	02/11/92	06/1999	10/1999
156.6.1	Pipe Break Effects on Systems and Components	RES/DSARE	HIGH	07/16/99	NA	TBD
163	Multiple Steam Generator Tube Leakage	NRR/DE/EMCB	HIGH	01/17/97	09/1999	TBD
168	Environmental Qualification of Electrical Equipment	RES/DET/ERAB	HIGH*	04/01/93	TBD	12/2000
170	Reactivity Transients and Fuel Damage Criteria for High Burn-Up Fuel	RES/DSARE/SMSAB	HIGH*	11/09/94	TBD	TBD
172	Multiple System Responses Program	RES/DRAA/PRAB	HIGH*	12/07/95	12/1998	TBD
173.A	Spent Fuel Storage Pool: Operating Facilities	NRR/DSSA/SPLB	HIGH*	06/24/96	06/1999	08/2000
190	Fatigue Evaluation of Metal Components for 60-Year Plant Life	RES/DET/MEB	HIGH*	08/26/96	07/1999	12/1999
191	Assessment of Debris Accumulation on PWR Sump Performance	RES/DET/ERAB	HIGH*	09/--/96	09/2001	09/2001
B-17	Criteria for Safety-Related Operator Actions	RES/DSARE/REAHFB	MEDIUM	03/22/82	12/1999	03/2000
B-55	Improve Reliability of Target Rock Safety Relief Valves	NRR/DE/EMEB	MEDIUM	NRR-OP FY83	03/1999	12/1999

* Previously listed as Nearly-Resolved but changed to HIGH in SECY-98-166

Total: 12

As of 4th Quarter FY-1999

TABLE 2
NUMBER OF REACTOR GSIs RESOLVED BY FISCAL YEAR
FY-83 TO FY-99 (4TH QUARTER)

FISCAL YEAR	USI	HIGH	MEDIUM	NR	TOTAL
FY-83	2	0	0	4	6
FY-84	2	1	3	9	15
FY-85	0	6	10	7	23
FY-86	1	3	2	3	9
FY-87	2	3	4	1	10
FY-88	5	6	2	3	16
FY-89	4	9	3	2	18
FY-90	0	2	2	3	7
FY-91	0	2	1	1	4
FY-92	0	4	2	1	7
FY-93	0	7	3	0	10
FY-94	0	1	2	2	5
FY-95	0	0	0	1	1
FY-96	0	1	1	1	3
FY-97	0	0	1	2	3
FY-98	0	0	0	0	0
FY-99	0	2	2	0	4
TOTAL	16	47	38	40	141

TABLE 3
REACTOR GSIs RESOLVED BY FISCAL YEAR

ISSUE NUMBER	TITLE	PRIORITY	RESOLUTION PRODUCT	DATE APPROVED FOR RESOLUTION	DATE RESOLVED
FY-1984 (CONT.)					
I.A.1.4	Long-Term Upgrading of Operating Personnel	NR	New Rule	NRR-OP FY83	01/01/84
II.A.1	Siting Policy Reformulation	Medium	No Req.	NRR-OP FY83	09/20/84
II.E.5.2	(B&W) Reactor Transient Response Task Force	NR	NUREG-0667	NRR-OP FY83	09/28/84
III.D.2.5	Offsite Dose Calculation Manual	NR	NUREG/CR-3332	NRR-OP FY83	01/17/84
FY-1985					
22	Inadvertent Boron Dilution Events	NR	GL 85-05	11/05/82	10/15/84
A-41	Long Term Seismic Program	Medium	No Req.	NRR-OP FY83	10/10/84
B-19	Thermal-Hydraulic Stability	NR	GL	01/03/85	05/21/85
B-54	Ice Condenser Containments	Medium	NUREG/CR-4001	NRR-OP FY83	10/22/84
B-58	Passive Mechanical Failures	Medium	No Req.	NRR-OP FY83	07/09/85
C-11	Assessment of Failure and Reliability of Pumps and Valves	Medium	No Req.	NRR-OP FY83	07/09/85
I.A.2.2	Training and Qualifications of Operating Personnel	High	Policy Statement (No Req.)	NRR-OP FY83	06/24/85
I.A.2.6(4)	Operator Workshops	Medium	No Req.	NRR-OP FY83	09/25/85
I.A.2.7	Accreditation of Training Institutions	Medium	Policy Statement (No Req.)	NRR-OP FY83	06/24/85
I.A.3.4	Licensing of Additional Operations Personnel	Medium	Policy Statement (No Req.)	NRR-OP FY83	02/12/85
I.G.2	Scope of Test Program	Medium	No Req.	NRR-OP FY83	10/05/84
II.B.6	Risk Reduction for Operating Reactors at Sites with High Population Densities	High	No Req.	NRR-OP FY83	09/25/85

TABLE 3
REACTOR GSIs RESOLVED BY FISCAL YEAR

ISSUE NUMBER	TITLE	PRIORITY	RESOLUTION PRODUCT	DATE APPROVED FOR RESOLUTION	DATE RESOLVED
FY-1986 (CONT.)					
36	Loss of Service Water	NR	SRP Revision (No Req.)	02/15/84	05/13/86
61	SRV Discharge Line Break Inside the Wetwell Airspace of BWR Mark I and II Containments	Medium	No Req.	11/30/83	08/08/86
A-43	Containment Emergency Sump Performance	USI	SRP Revision (Req.)	01/79	10/85
I.C.9	Long-Term Plan for Upgrading of Procedures	Medium	No Req.	NRR-OP FY83	06/07/85
III.D.3.1	Radiation Protection Plans	High	No Req.	NRR-OP FY83	05/19/86
HF1.2	Engineering Expertise on Shift	High	No Req.	10/01/84	10/28/85
HF1.3	Guidance on Limits and Conditions of Shift Work	High	No Req.	10/01/84	06/26/86
FY-1987					
91	Main Crankshaft Failures in Transamerica Delaval Diesel Generators	NR	NUREG-1216 (No Req.)	07/85	09/87
A-46	Seismic Qualification of Equipment in Operating Plants	USI	GL 87-02 (Req.)	02/81	02/87
A-49	Pressurized Thermal Shock	USI	Rule/Reg. Guide 1.154 (Req.)	12/81	02/87
I.A.2.6(1)	Long-Term Upgrading of Training and Qualifications - Revise Reg. Guide 1.8	High	Reg. Guide 1.8 (Req.)	10/82	05/87
I.A.3.3	Requirement for Operator Fitness	High	No Req.	12/82	01/87
I.A.4.2(1)	Research on Training Simulators	High	Reg. Guide 1.149, Rev. 1 (Req.)	10/84	05/87

TABLE 3
REACTOR GSIs RESOLVED BY FISCAL YEAR

ISSUE NUMBER	TITLE	PRIORITY	RESOLUTION PRODUCT	DATE APPROVED FOR RESOLUTION	DATE RESOLVED
FY-1988 (CONT.)					
43	Reliability of Air Systems	High	GL 88-14 (Req.)	12/87	09/88
66	Steam Generator Requirements	NR	NUREG-0844 (No Req.)	11/83	09/88
102	Human Error in Events Involving Wrong Unit or Wrong Train	NR	NUREG-1192 (No Req.)	02/85	09/88
125.II.7	Reevaluate Provision to Automatically Isolate Feedwater from Steam Generator During a Line Break	High	NUREG-1332 (No Req.)	09/86	09/88
A-3,4,5	Steam Generator Tube Integrity	USI	NUREG-0844 (No Req.)	01/79	09/88
A-45	Shutdown Decay Heat Removal Requirements	USI	NUREG-1289 (No Req.)	02/81	09/88
FY-1989					
51	Proposed Requirements for Improving Reliability of Open Cycle Service Water Systems	Medium	GL 89-13 (Req.)	06/83	08/89
82	Beyond Design Bases Accidents in Spent Fuel Pools	Medium	NUREG-1353 (No Req.)	12/07/83	04/89
99	RCS/RHR Suction Line Interlocks on PWRs	High	GL 88-17 (Req.)	08/85	11/88
101	BWR Water Level Redundancy	High	GL 89-11 (Req.)	05/06/85	06/89
115	Enhancement of the Reliability of Westinghouse Solid State Protection System	High	NUREG-1341 (No Req.)	07/07/86	04/89
122.2	Initiating Feed-and-Bleed	High	No Req.	01/86	04/89
124	Auxiliary Feedwater System Reliability	NR	2 Plant-Specific Backfits (Req.)	02/86	01/89

TABLE 3
REACTOR GSIs RESOLVED BY FISCAL YEAR

ISSUE NUMBER	TITLE	PRIORITY	RESOLUTION PRODUCT	DATE APPROVED FOR RESOLUTION	DATE RESOLVED
FY-1990 (CONT.)					
84	CE PORVs	NR	SECY-90-232 (No Req.)	02/27/85	06/90
94	Additional Low-Temperature Overpressure Protection for LWRs	High	GL 90-06 (Req.)	07/23/85	06/90
103	Design for Probable Maximum Precipitation	NR	GL 89-22 (Req.)	09/04/85	11/89
A-29	Nuclear Power Plant Design for the Reduction of Vulnerability to Industrial Sabotage	Medium	No Req.	NRR-OP FY83	10/89
C-8	Main Steam Line Isolation Valve Leakage Control Systems	High	No Req.	NRR-OP FY83	03/90
FY-1991					
128	Electrical Power Reliability	High	GL 91-06, GL 91-11 (Req.)	11/28/86	09/91
130	Essential Service Water System Failures at Multiplant Sites	High	GL 91-13 (Req.)	03/10/87	09/91
135	Steam Generator and Steam Line Overfill	Medium	No Req.	05/27/86	03/91
II.J.4.1	Revise Deficiency Report Requirements	NR	Rule (Req.)	NRR-OP FY83	07/91
FY-1992					
29	Bolting Degradation or Failure in Nuclear Power Plants	High	No Req.	NRR-OP FY-83	10/91
73	Detached Thermal Sleeves	NR	NUREG/CR-6010 (No Req.)	08/20/91	09/92
79	Unanalyzed Reactor Vessel Thermal Stress During Natural Convection Cooldown	Medium	GL 92-02 (No Req.)	NRR-OP FY-84	05/92
87	Failure of HPCI Steam Line Without Isolation	High	No Req.	09/26/85	12/91
113	Dynamic Qualification Testing of Large Bore Hydraulic Snubbers	High	No Req.	07/02/87	08/92

TABLE 3
REACTOR GSIs RESOLVED BY FISCAL YEAR

ISSUE NUMBER	TITLE	PRIORITY	RESOLUTION PRODUCT	DATE APPROVED FOR RESOLUTION	DATE RESOLVED
FY-1994 (CONT.)					
II.H.2	Obtain Technical Data on the Conditions Inside the TMI-2 Containment Structure	High	No Req.	NRR-OP FY83	02/94
B-64	Decommissioning of Nuclear Reactors	NR	Rule (Req.)	NRR-OP FY84	09/94
FY-1995					
155.1	More Realistic Source Term Assumptions	NR	NUREG-1465 (Req.)	02/26/92	03/95
FY-1996					
15	Radiation Effects on Reactor Vessel Supports	High	NUREG-1509 (No Req.)	02/--/89	05/96
24	Automatic Emergency Core Cooling System Switch to Recirculation	Medium	No Req.	07/--/91	10/95
83	Control Room Habitability	NR	NUREG/CR-5669 (No Req.)	08/--/83	06/96
FY-1997					
78	Monitoring of Fatigue Transient Limits for Reactor Coolant System	Medium	No Req.	07/10/92	02/97
166	Adequacy of Fatigue Life of Metal Components	NR	No Req.	04/01/93	02/97
173.B	Spent Fuel Storage Pool: Permanently Shutdown Facilities	NR	No Req.	06/24/96	10/96
FY-1998					
None.					
FY-1999					
171	ESF Failure from LOOP Subsequent to a LOCA	HIGH	No Req.	06/16/95	12/98
B-61	Allowable ECCS Equipment Outage Periods	MEDIUM	No Req.	NRR OP FY-83	03/99

TABLE 4

NET CHANGE BY FISCAL YEAR IN REACTOR GSIs SCHEDULED FOR RESOLUTION FY-83 TO FY-99 (4TH QUARTER)

FY-1983

PRIORITY CATEGORY	START	ADDITIONS	RESOLVED	INTEGRATED	END
USI	16	0	2	0	14
HIGH	24	2	0	0	26
MEDIUM	31	2	0	0	33
NR	20	3	4	0	19
TOTAL	91	7	6	0	92

FY-1984

PRIORITY CATEGORY	START	ADDITIONS	RESOLVED	INTEGRATED	END
USI	14	0	2	0	12
HIGH	26	1	1	0	26
MEDIUM	33	4	3	0	34
NR	19	5	9	0	15
TOTAL	92	10	15	0	87

FY-1985

PRIORITY CATEGORY	START	ADDITIONS	RESOLVED	INTEGRATED	END
USI	12	0	0	0	12
HIGH	22*	41	6	0	57
MEDIUM	28*	1	10	0	19
NR	15	11	7	0	19
TOTAL	77	53	23	0	107

TABLE 4

NET CHANGE BY FISCAL YEAR IN REACTOR GSIs SCHEDULED FOR RESOLUTION FY-83 TO FY-99 (4TH QUARTER)

FY-1989

PRIORITY CATEGORY	START	ADDITIONS	RESOLVED	INTEGRATED	END
USI	4	0	4	0	0
HIGH	24	1	9	0	16
MEDIUM	13	1	3	1	10
NR	9	0	2	0	7
TOTAL	50	2	18	1	33

FY-1990

PRIORITY CATEGORY	START	ADDITIONS	RESOLVED	INTEGRATED	END
USI	-	-	-	-	-
HIGH	16	0	2	0	14
MEDIUM	10	1	2	0	9
NR	7	0	3	0	4
TOTAL	33	1	7	0	27

FY-1991

PRIORITY CATEGORY	START	ADDITIONS	RESOLVED	INTEGRATED	END
USI	-	-	-	-	-
HIGH	14	2	2	0	14
MEDIUM	9	3	1	0	11
NR	4	1	1	0	4
TOTAL	27	6	4	0	29

TABLE 4

NET CHANGE BY FISCAL YEAR IN REACTOR GSIs SCHEDULED FOR RESOLUTION FY-83 TO FY-99 (4TH QUARTER)

FY-1995

PRIORITY CATEGORY	START	ADDITIONS	RESOLVED	INTEGRATED	END
USI	-	-	-	-	-
HIGH	3	1	0	0	4
MEDIUM	6	0	0	0	6
NR	5	1	1	0	5
TOTAL	14	2	1	0	15

FY-1996

PRIORITY CATEGORY	START	ADDITIONS	RESOLVED	INTEGRATED	END
USI	-	-	-	-	-
HIGH	4	0	1	0	3
MEDIUM	6	0	1	0	5
NR	5	5	1	0	9
TOTAL	15	5	3	0	17

FY-1997

PRIORITY CATEGORY	START	ADDITIONS	RESOLVED	INTEGRATED	END
USI	-	-	-	-	-
HIGH	3	1	0	0	4
MEDIUM	5	0	1	0	4
NR	9	0	2	0	7
TOTAL	17	1	3	0	15

TABLE 4A
NET CHANGE IN REACTOR GSIs RESOLVED
FY-83 TO FY-99 (4TH QUARTER)

PRIORITY CATEGORY	START	ADDITIONS	SUB-TOTAL	RESOLVED	INTEGRATED**	REMAINDER
USI	16	0	16	16	0	0
HIGH	24	43*	67	47	10	10
MEDIUM	31	18	49	38	9	2
NR	20	21*	41*	40	1	0
TOTAL:	91	82	173	141	20	12

- Extensive revisions to Human Factors issues resulted in priority changes in FY-85 and FY-86.

** GSIs Integrated

FY-87 (13):
Issues 48, 49, and A-30 into Issue 128
Issue 65 into Issue 23
Issues 68; 122.1.a; 122.1.b; 122.1.c; and 125.II.1.b into Issue 124
Issues I.B.1.1(6) and I.B.1.1(7) into Issue 75
Issue B-6 into Issue 119.1
Issue 67.7 into 135

FY-88 (6):
Issue 77 into A-17
Issues I.D.5(5), II.B.5(1), II.B.5(2), II.B.5(3), and II.F.5 were integrated into the Research Activities Program and were reclassified as Licensing Issues.

FY-89 (1):
Issue 131 was integrated into the IPEEE Program.

TABLE 6
REACTOR GENERIC ISSUES TO BE REPRIORITIZED

ISSUE NUMBER	TITLE	LEAD OFFICE/ DIVISION/BRANCH	PREVIOUS PRIORITY	IDENTIFICATION DATE	CURRENT SCHEDULE
71	Failure of Resin Demineralizer Systems and Their Effects on Nuclear Power Plant Safety	RES/DET	LOW	04/1996	TBD
80	Pipe Break Effects on Control Rod Drive Hydraulic Lines in the Drywells of BWR MARK I and II Containments	RES/DET	LOW	03/1998	TBD
152	Design Basis for Valves That Might Be Subjected to Significant Blowdown Loads	RES/DET	LOW	04/1996	TBD

TOTAL: 3

TABLE 7
REACTOR GENERIC ISSUES PRIORITIZED

ISSUE NUMBER	TITLE	IDENTIFICATION DATE	DATE PRIORITIZED	CURRENT PRIORITY
FY-1984				
34	RCS Leak	09/1982	02/1984	DROP
35	Degradation of Internal Appurtenances in LWRs	09/82	02/84	LOW
36	Loss of Service Water	09/82	02/84	NR
43	Contamination of Instrument Air Lines	09/82	11/83	DROP
44	Failure of Saltwater Cooling System	09/82	10/83	S(43)
48	LCO for Class IE Vital Instrument Buses in Operating Reactors	09/82	10/83	NR
49	Interlocks and LCOs for Redundant Class IE Tie Breakers	09/82	07/84	MEDIUM
53	Consequences of a Postulated Flow Blockage Incident in a BWR	09/82	09/84	DROP
60	Lamellar Tearing of Reactor Systems Structural Supports	10/82	11/83	S(A-12)
61	SRV Line Break Inside the BWR Wetwell Airspace of Mark I and II Containments	10/82	11/83	MEDIUM
66	Steam Generator Requirements	06/83	11/83	NR
68	Postulated Loss of Auxiliary Feedwater System Resulting from Turbine-Driven Auxiliary Feedwater Pump Steam Supply Line Rupture	06/83	04/84	HIGH
69	Make-up Nozzle Cracking in B&W Plants	06/83	12/83	NR
70	PORV and Block Valve Reliability	06/83	05/84	MEDIUM
75	Generic Implications of ATWS Events at the Salem Nuclear Plant	06/83	10/83	NR
80	Pipe Break Effects on Control Rod Drive Hydrau-lic Lines in the Drywells of BWR Mark I and II Containments	06/83	01/84	LOW
82	Beyond Design Basis Accidents in Spent Fuel Pools	08/83	12/83	MEDIUM
90	Technical Specifications for Anticipatory Trips	02/84	08/84	LOW
92	Fuel Crumbling During LOCA	04/83	07/84	LOW
B-65	Iodine Spiking	09/82	06/84	DROP

TABLE 7
REACTOR GENERIC ISSUES PRIORITIZED

ISSUE NUMBER	TITLE	IDENTIFICATION DATE	DATE PRIORITIZED	CURRENT PRIORITY
FY-1985 (CONT.)				
121	Hydrogen Control For Large Dry PWR Containments	08/85	09/85	HIGH
B-50	Post Operating Basis Earthquake Inspection	02/83	04/85	RI(LOW)
B-59	N-1 Loop Operation in BWRs and PWRs	02/83	06/85	RESOLVED
HF-01	Human Factor Program Plan (HFPP with 24 subtasks)	08/83	10/84	HIGH
HF-02	Maintenance and Surveillance Program Plan (MSPP with 10 subtasks)	04/84	03/85	HIGH
FY-1986				
21	Vibration Qualification of Equipment	03/83	06/86	DROP
30	Potential Generator Missiles - Generator Rotor Retaining Rings	09/82	10/85	DROP
74	Reactor Coolant Activity Limits for Operating Reactors	06/83	05/86	DROP
97	PWR Reactor Cavity Uncontrolled Exposures	09/84	10/85	S(III.D.3.1)
111	Stress Corrosion Cracking of Pressure Boundary Ferritic Steels in Selected Environments	01/85	11/85	LI
112	Westinghouse RPS Surveillance Frequencies andOut-of-Service Times	01/85	10/85	RI(R)
114	Seismic-Induced Relay Chatter	03/85	06/86	S(A-46)
115	Reliability of Westinghouse Solid State Protection System	04/85	07/86	HIGH
122.1.a	Common Mode Failure of Isolation Valves in Closed Position	08/85	01/86	HIGH
122.1.b	Recovery of Auxiliary Feedwater	08/85	01/86	MEDIUM
122.1.c	Interruption of Auxiliary Feedwater Flow	08/85	01/86	HIGH
122.2	Initiating Feed-and-Bleed	08/85	01/86	HIGH
122.3	Physical Security System Constraints	08/85	01/86	LOW
124	Auxiliary Feedwater System Reliability	12/85	02/86	NR
125.I.2.a	PORV Reliability - Test Program	11/85	06/86	S(70)
125.I.2.b	PORV Reliability - Surveillance	11/85	06/86	S(70)

TABLE 7
REACTOR GENERIC ISSUES PRIORITIZED

ISSUE NUMBER	TITLE	IDENTIFICATION DATE	DATE PRIORITIZED	CURRENT PRIORITY
FY- 87 (CONT.)				
125.II.2	Adequacy of Existing Maintenance Requirements for Safety-Related Systems	11/85	06/87	DROP
125.II.5	Thermal-Hydraulic Effects of Loss and Restoration of Feedwater on Primary System Components	11/85	06/87	DROP
125.II.6	Reexamine PRA Estimates of Core Damage Risk from Loss of All Feedwater	11/85	03/87	DROP
125.II.8	Reassess Criteria for Feed-and-Bleed Initiation	11/85	03/87	DROP
125.II.10	Hierarchy of Impromptu Operator Actions	11/85	02/87	DROP
125.II.12	Adequacy of Training Regarding PORV Operation	11/85	03/87	DROP
127	Testing and Maintenance of Manual Valves in Safety-Related Systems	05/86	06/87	LOW
128	Electrical Power Reliability	05/86	11/86	HIGH
130	Essential Service Water Pump Failures at Multiplant Sites	06/86	03/87	HIGH
135	Steam Generator and Steam Line Overfill	05/86	06/87	MEDIUM
FY-1988				
43*	Reliability of Air Systems	04/87	12/87	HIGH
55*	Failure of Class 1E Safety-Related Switchgear Circuit Breakers to Close on Demand	09/85	02/88	DROP
57	Effects of Fire Protection System Actuation on Safety-Related Equipment	09/82	06/88	MEDIUM
62	Reactor Systems Bolting Applications	10/82	08/88	S(29)
88	Earthquakes and Emergency Planning	01/84	10/87	RESOLVED
104	Reduction of Boron Dilution Requirements	10/84	08/88	DROP
106	Piping and Use of Highly Combustible Gases in Vital Areas	10/84	11/87	MEDIUM
125.I.3	SPDS Availability	11/85	05/88	NR
125.I.6	Valve Torque Limit and Bypass Switch Settings	11/85	12/87	DROP
125.I.7A	Recover Failed Equipment	11/85	12/87	DROP
125.II.11	Recovery of Main Feedwater as Alternative to AFW	11/85	06/88	DROP

TABLE 7
REACTOR GENERIC ISSUES PRIORITIZED

ISSUE NUMBER	TITLE	IDENTIFICATION DATE	DATE PRIORITIZED	CURRENT PRIORITY
FY-1990 (CONT.)				
117	Allowable Time for Diverse Simultaneous Equipment Outages	05/1985	02/1990	DROP
129	Valve Interlocks to Prevent Vessel Drainage During Shutdown Cooling	05/1986	02/1990	DROP
137	Refueling Cavity Seal Failure	10/1986	05/1990	DROP
140	Fission Product Removal Systems	03/1987	02/1990	DROP
141	LBLOCA With Consequential SGTR	04/1987	05/1990	DROP
142	Leakage Through Electrical Isolators	06/1987	06/1990	MEDIUM
B-29	Effectiveness of Ultimate Heat Sinks	02/1983	08/1990	LI(RESOLVED)
B-32	Ice Effects on Safety-Related Water Supplies	02/1983	08/1990	S(153)
FY-1991				
24	Automatic Emergency Core Cooling System Switch to Recirculation	03/83	07/91	MEDIUM
38	Potential Recirculation System Failure as a Consequence of Ingestion of Containment Paint Flakes or Other Fine Debris	09/82	08/91	DROP
72	Control Rod Drive Guide Tube Support Pin Failures	06/83	10/90	DROP
73	Detached Thermal Sleeves	06/83	08/91	NR
100	Once-Through Steam Generator Level	09/84	09/91	DROP
120	On-line Testability of Protection Systems	08/85	11/90	MEDIUM
143	Availability of Chilled Water Systems and Room Cooling	10/87	03/91	HIGH
150	Overpressurization of Containment Penetrations	04/89	08/91	DROP
151	Reliability of Anticipated Transient Without Scram Recirculation Pump Trip in BWRs	04/89	08/91	MEDIUM
153	Loss of Essential Service Water in LWRs	05/90	03/91	HIGH
A-19	Digital Computer Protection System	02/83	11/90	LI
B-22	LWR Fuel	02/83	06/91	DROP

TABLE 7
REACTOR GENERIC ISSUES PRIORITIZED

ISSUE NUMBER	TITLE	IDENTIFICATION DATE	DATE PRIORITIZED	CURRENT PRIORITY
FY-1992 (CONT.)				
156.1.1	Settlement of Foundations and Buried Equipment	02/91	08/92	S(IPEEE)
156.1.2	Dam Integrity and Site Flooding	02/91	01/92	DROP
156.1.3	Site Hydrology and Ability to Withstand Floods	02/91	01/92	DROP
156.1.4	Industrial Hazards	02/91	03/92	DROP
156.1.5	Tornado Missiles	02/91	01/92	DROP
156.1.6	Turbine Missiles	02/91	10/91	DROP
156.2.1	Severe Weather Effects on Structures	02/91	01/92	DROP
156.2.2	Design Codes, Criteria, and Load Combinations	02/91	07/92	DROP
156.2.3	Containment Design and Inspection	02/91	05/92	DROP
156.2.4	Seismic Design of Structures, Systems,and Components	02/91	03/92	DROP
156.3.1.1	Shutdown Systems	02/91	03/92	DROP
156.3.1.2	Electrical Instrumentation and Control	02/91	03/92	DROP
156.3.2	Service and Cooling Water Systems	02/91	03/92	DROP
156.3.3	Ventilation Systems	02/91	09/92	DROP
156.3.4	Isolation of High and Low Pressure Systems	02/91	12/91	DROP
156.3.5	Automatic ECCS Switchover	02/91	11/91	S(24)
156.3.6.1	Emergency AC Power	02/91	02/92	S(B-56)
156.3.8	Shared Systems	02/91	06/92	DROP
156.4.1	RPS and ESFS Isolation	02/91	11/91	S(142)
156.4.2	Testing of the RPS and ESFS	02/91	03/92	S(120)
157	Containment Performance	10/91	02/92	RESOLVED

TABLE 7
REACTOR GENERIC ISSUES PRIORITIZED

ISSUE NUMBER	TITLE	IDENTIFICATION DATE	DATE PRIORITIZED	CURRENT PRIORITY
FY-1996 (CONT.)				
173.B	Spent Fuel Storage Pool: Permanently Shutdown Facilities	02/96	05/96	NR
174.A	Fastener Gaging Practices: SONGS Employees'Concern	02/96	05/96	RESOLVED
174.B	Fastener Gaging Practices: Johnson Gage Company Concern	02/96	05/96	RESOLVED
175	Nuclear Power Plant Shift Staffing	02/96	05/96	RESOLVED
176	Loss of Fill-Oil in Rosemount Transmitters	02/96	05/96	RESOLVED
177	Vehicle Intrusion at TMI	02/96	05/96	RESOLVED
178	Effect of Hurricane Andrew on Turkey Point	05/96	05/96	LI
179	Core Performance	02/96	05/96	LI
180	Notice of Enforcement Discretion	02/96	05/96	LI (Resolved)
181	Fire Protection	02/96	05/96	LI
182	General Electric Extended Power Uprate	05/96	05/96	RI
183	Cycle-Specific Parameter Limits in Technical Specifications	02/96	05/96	RI
184	Endangered Species	05/96	05/96	EI
190	Fatigue Evaluation of Metal Components for 60-Year Plant Life	08/96	08/96	NR
191	Assessment of Debris Accumulation on PWR Sump Performance	09/96	09/96	NR
FY-1997				
163	Multiple Steam Generator Tube Leakage	06/92	01/97	HIGH
FY-1998				
169	BWR MSIV Common Mode Failure Due to Loss of Accumulator Pressure	10/93	03/98	DROP
I.F.2(1)*	QA - Assure the Independence of the Organization Performing the Checking Function	04/1997	07/1998	LOW
II.D.2*	Research on Relief and Safety Valve Test Requirements	04/1997	07/1998	DROP

TABLE 8
NUMBER OF REACTOR GSIs PRIORITIZED FROM FY-83 TO FY-99 (4 TH QUARTER)

ISSUE TYPE	FY-83	FY-84	FY-85	FY-86	FY-87	FY-88	FY-89	FY-90	FY-91	FY-92	FY-93	FY-94	FY-95	FY-96	FY-97	FY-98	FY-99	TOTAL
Issues Identified to be Prioritized	56	19	54	45*	6	3	38	3	29	7	5	1	2	17	0	1	2	288
Issues Identified to be Reprioritized	19	2	0	1	1	2	0	0	0	0	0	0	0	3	2	0	0	30
Total:	75	21	54	46	7	5	38	3	29	7	5	1	2	20	2	1	2	318
New Issues (Entered into GIMCS)																		
High	2	1	41	6	4	1	1	0	2	0	0	1	1	0	1	0	1	62
Medium	2	4	1	1	1	2	0	1	3	2	0	1	0	0	0	0	0	18
Nearly-Resolved	3	5	6	1	0	1	0	0	1	2	2	0	1	5	0	0	0	27
Sub-total:	7	10	48	8	5	4	1	1	6	4	2	2	2	5	1	0	0	107
Resolved	4	0	1	0	0	1	1	1	0	2	1	0	0	5	0	0	0	16
Low	1	4	0	2	1	0	0	2	0	4	3	1	0	0	0	1	0	19
Drop	1	4	4	6	13	9	2	8	5	24	6	0	0	0	0	2	1	85
RI/LI/EI	0	0	4	6	0	2	33	1	1	3	0	0	0	7	0	0	0	57
Integrated	8	2	3	6	0	1	1	3	0	5	0	0	0	0	0	0	0	29
Total Issues Prioritized:	21	20	60	28	19	17	38	16	12	42	12	3	2	17	1	3	2	313
[Annual Progress]/ Remaining Issues to be Prioritized or Reprioritized:	[+54	+1	-6	+18	-12	-12	0	-13	+17	-35	-7	-2	0	+3	+1	-2	0]	5

TABLE 12
NON-REACTOR GENERIC ISSUES PRIORITIZED

NMSS ISSUE NUMBER	TITLE	IDENTIFICATION DATE	DATE PRIORITIZED	CURRENT PRIORITY
FY-1997				
0001	Door Interlock Failure Resulting from Faulty MicroSelectron-High Dose Rate Remote Afterloader	04/1996	02/1997	Resolved
0002	Significant Quantities of Fixed Contamination Remain in Krypton-85 Leak-Detection Devices After Venting	07/1996	10/1996	Resolved
0003	Corrosion of Sealed Sources Caused by Sensitization of Stainless Steel Source Capsules During Shipment	07/1996	10/1996	Resolved
0005	Potential for Erroneous Calibration, Dose Rate,or Radiation Exposure Measurements With Victoreen Electrometers	06/1997	06/1997	High
0006	Criticality Concerns With Unusual Moderators in Low-Level Waste	08/1997	08/1997	Medium
FY-1998				
0004	Overexposures Caused by Sources Stolen from Facility of Bankrupt Licensee	07/1996	12/1997	Resolved
0007	Criticality Benchmarks Greater Than 5% Enrichment	05/1998	06/1998	Low
0008	Year 2000 Computer Problem - Non-Reactor Licensees	05/1998	06/1998	High
0009	Amersham Radiography Source Cable Failures	05/1998	06/1998	High
0010	Troxler Gauge Source Rod Weld Failures	05/1998	06/1998	Medium
0011	Spent Fuel Dry Cask Weld Cracks	05/1998	06/1998	Medium
0012	Inadequate Transportation Packaging Puncture Tests	05/1998	06/1998	Medium
0013	Use of Different Dose Models to Demonstrate Compliance	06/1998	07/1998	Medium
0014	Surety Estimates for Groundwater Restoration at In-Situ Leach Facilities	06/1998	07/1998	Medium
0015	Adequacy of Part 150 Criticality Requirements	06/1998	07/1998	Medium
0016	Adequacy of 0.05 Weight Percent Limit in Part 40	06/1998	07/1998	Medium

TABLE 14
NON-REACTOR GSIs RESOLVED BY FISCAL YEAR

NMSS ISSUE NUMBER	TITLE	PRIORITY	RESOLUTION PRODUCT	DATE APPROVED FOR RESOLUTION	DATE RESOLVED
FY-1997					
0001	Door Interlock Failure Resulting from Faulty MicroSelectron-High Dose Rate Remote Afterloader	Resolved	IN 96-21	02/1997	02/1997
0002	Significant Quantities of Fixed Contamination Remain in Krypton-85 Leak-Detection Devices After Venting	Resolved	IN 96-51	10/1996	10/1996
0003	Corrosion of Sealed Sources Caused by Sensitization of Stainless Steel Source Capsules During Shipment	Resolved	IN 96-54	10/1996	10/1996
0005	Potential for Erroneous Calibration,Dose Rate or Radiation Exposure Measurements With Victoreen Electrometers	High	Bulletin 97-01	06/1997	09/1997
FY-1998					
0004	Overexposures Caused by Sources Stolen from Facility of Bankrupt Licensee	Resolved	Staff Report	12/1997	12/1997
FY-1999					
0006	Criticality Concerns With Unusual Moderators in Low-Level Waste	Medium	Staff Report	06/1997	06/1999
0009	Amersham Radiography Source Cable Failures	High	IN 97-91, Supplement 1	06/1998	10/1998
0011	Spent Fuel Dry Cask Weld Cracks	Medium	NUREG-1536	06/1998	10/1998
0012	Inadequate Transportation Packaging Puncture Tests	Medium	Staff Report	05/1998	06/1999
0013	Use of Different Dose Models to Demonstrate Compliance	Medium	Staff Report	07/1998	05/1999

RUN DATE: 11/01/1999

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ISSUE NUMBER: 23 TITLE: REACTOR COOLANT PUMP SEAL FAILURES

IDENTIFICATION DATE: 12/1980C PRIORITIZATION DATE: 04/1983C TYPE: GSI PRIORITY: H

TASK MANAGER: J.JACKSON OFFICE/DIVISION/BRANCH: RES /DET /ERAB ACTION LEVEL: ACTIVE STATUS:

TAC NUMBERS: K81095 RESOLUTION DATE: 12/1999

WORK AUTHORIZATION: NRR FY-84 OPERATING PLAN, (APPENDIX G)

FIN	CONTRACTOR	CONTRACT TITLE
A3751	BNL	REACTOR COOLANT PUMP (RCP) SEAL INTEGRITY
A3771	BNL	ASSESS EFFECT OF MECH. MAINT. RCP SEAL
A3806	BNL	EVAL. ADEQ. OF RCP SEAL INSTR. & RESP. TO RCP
A6322	INEL	RCP SHAFT SEAL BEHAVIOR DURING ST. BLACKOUT
B2953	PNL	REACTOR COOL. PUMP SEAL FAIL. VAL.IMP. ANALYSIS
B3049	ETEC	LEAK RATE ANALYSIS OF WESTINGHOUSE RCP
D1760	SCIENTECH	REG. ANAL. FOR GI-23, RCP SEAL FAILURES
W6211	BNL	SOLUTIONS TO MAINTAIN RCP SEAL INTEGRITY

W O R K S C O P E

THE TAP FOR THIS ISSUE WAS REVISED TO INCORPORATE WORK NECESSARY TO COMPLETE GSI 65. REVIEW PWR RCP SEAL FAILURES CAUSED BY MECHANICAL FAILURES AND LOSS OF SEAL COOLING TO ASCERTAIN IF INCREASED NRC REQUIREMENTS ARE NECESSARY TO MINIMIZE SEAL FAILURES. EVALUATE THE EFFECTS OF SYSTEM DESIGN, MATERIALS, OPERATION, TESTING, AND MAINTENANCE ON SEAL FAILURE MECHANISMS. REVIEW THE RELIAB. OF SEAL COOLING SYSTEMS AND IDENTIFY METHODS TO MINIMIZE THE EFFECTS OF LOSS OF SEAL COOLING. PERFORM A RISK ASSESSMENT OF SEAL FAILURE FROM MAINTENANCE AND MECHANICAL EFFECTS.

THIS PROGRAM WILL DETERMINE WHAT STEPS THE NRC SHOULD TAKE, IF ANY, TO REDUCE RCP SEAL FAILURE TO AN ACCEPTABLE LEVEL COMMENSURATE WITH THE POTENTIAL SAFETY CONSEQUENCES OF SEAL FAILURE. THE PROGRAM CONSISTS OF THE FOLLOWING MAJOR TASKS:

TASK 1 - DETERMINE ACTUAL SEAL FAILURE MECHANISMS.

TASK 2 - EVALUATE RCP SEAL COOLING FOR ALL MODES OF OPERATION FOR WHICH SEAL COOLING IS REQ'D USING THE RELIABILITY OF TYPICAL RCP SEAL COOLING SYSTEM DESIGNS.

TASK 3 - REVIEW MECHANICAL AND MAINTENANCE-INDUCED FAILURE MECHANISMS.

TASK 4- FEASIBILITY STUDY TO IDENTIFY MOST EFFECTIVE FIX.

S T A T U S

AN NRR GENERIC LETTER FOR INFORMATION ONLY WAS ISSUED MAY 2, 1991, TO INFORM LICENSEES OF THE POTENTIAL IMPACT ON SBO ANALYSES. A FEDERAL REGISTER NOTICE WAS PUBLISHED APRIL 19, 1991, REQUESTING PUBLIC COMMENT ON DRAFT REGULATORY GUIDE DG-1008 AND THE DRAFT REGULATORY ANALYSIS, NUREG-1401. THE STAFF DECIDED TO PROCEED WITH A RULEMAKING FOR OFF-NORMAL CONDITIONS, WHILE THE NORMAL CONDITIONS HAVE BEEN ADDRESSED BY APPROPRIATE GENERIC COMMUNICATIONS.

NEW TASK MANAGER, J. JACKSON, ASSIGNED TO REPLACE K. SHAUKAT AFTER RES REORGANIZATION ON 03-28-99.

A TASK ACTION PLAN HAS BEEN DEVELOPED TO ADDRESS RECENT QUESTIONS REGARDING THE DEVELOPMENT OF RCP SEAL MODELS AND

ISSUE NUMBER: 23 TITLE: REACTOR COOLANT PUMP SEAL FAILURES

MILESTONES	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
TAP APPROVED BY DSI DIR	10/1983	--	10/1983
ISSUED REQUEST FOR PROPOSAL TO BNL FOR COMPONENT COOLING WATER PRA	11/1983	--	11/1983
TRANSFER RESOLUTION REPONSIBILITY FROM DSI TO DE	06/1984	--	06/1984
TRANSFER RESOLUTION RESPONSIBILTY FROM DE TO DST/GIB	05/1985	--	05/1985
COMPLETE DOCUMEN. OF BACKGROUND PAST SEAL FAILURES, INCLUDING LER SEARCH,REV OF INDUST. EXP.& REV.OF BWR SCR.(TASK1)	05/1985	--	12/1985
EVAL. RCP SEAL COOLING, INCL. DETERM. EFFECT OF LOSS COOLING, EVAL RELIAB. OF COOLING SYS. AND REV. RCP SEAL COOLING INSTR. (TASK 2)	12/1984	--	08/1986
EVALUATE MECHANICAL AND MAINTENANCE-INDUCED FAILURE MECHANISMS (TASK 3)	01/1985	--	01/1988
DRAFT SCIENTECH REPORTS TO IDENTIFY MOST EFFECTIVE FIX. (TASK 4)	03/1985	--	01/1989
REVIEW & FINALIZE SCIENTECH REPORTS	05/1989	--	03/1990
DTR PACKAGE COMPLETE	07/1985	--	04/1990
DIVISION DIRECTOR REVIEW COMPLETED	08/1985	--	05/1990
ACRS REVIEW OF DTR COMPLETE	11/1985	--	09/1990
CONCURRENCE BY OTHER OFFICES	09/1985	--	10/1990
DTR SENT TO CRGR	10/1985	--	10/1990
CRGR REVIEW OF DTR COMPLETE	11/1985	--	03/1991
FRN OF ISSUANCE OF PROPOSED RESOLUTION FOR PUBLIC COMMENT	12/1985	--	04/1991

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: 23 TITLE: REACTOR COOLANT PUMP SEAL FAILURES

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
CLOSE OUT ISSUE WITH MEMO TO THE EDO	09/1995	12/1999	--

ISSUE NUMBER: 145 TITLE: ACTIONS TO REDUCE COMMON CAUSE FAILURES

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
COMPLETION OF TAP	04/1992	--	04/1992
ISSUE CONTRACT FUND	01/1993	--	04/1993
CONTRACTOR'S DRAFT FINDINGS	06/1992	--	10/1993
CONTRACTOR'S FINAL FINDINGS	09/1992	--	12/1993
STAFF'S DRAFT FINDINGS	10/1992	--	11/1993
DECISION ON STAFF'S FINDINGS	11/1992	--	12/1993
DTR TO NRR FOR COMMENTS	04/1995	--	04/1995
CONTRACTOR PUBLICATION TO JOURNALS	05/1995	--	05/1995
NRR PROVIDES COMMENTS ON BNL'S FINDINGS	02/1995	--	07/1995
CONTRACTOR CORRECTIONS TO FINAL TS REPORTS	08/1995	--	09/1995
FTR COMPLETED BY TASK MANAGER	08/1995	--	11/1995
NEW TASK MANAGER ASSIGNED AFTER 03-28-99 RES REORGANIZATION	04/1999	--	04/1999
COMPLETE DRAFT OF FINAL RESOLUTION	06/1998	--	07/1999
FTR TO ACRS FOR REVIEW	09/1996	--	07/1999
MEETING WITH ACRS FULL COMMITTEE	07/1998	--	09/1999
RESPONSE FROM ACRS	09/1999	--	09/1999
ISSUE CLOSEOUT MEMO TO EDO	04/1996	10/1999	--

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: 148 TITLE: SMOKE CONTROL AND MANUAL FIRE-FIGHTING EFFECTIVENESS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
ISSUE FINAL STAFF REVIEW GUIDANCE	09/1993	--	03/1997
REVIEW CURRENT IPEEE TREATMENT AND RESOLUTION	05/1998	--	08/1997
DOCUMENT IPEEE RESOLUTION PROCESS/CRITERIA, PREPARE CLOSE OUT MEMORANDUM	06/1998	--	07/1998
ISSUE CLOSE OUT MEMO TO RES DIRECTOR	06/1995	--	07/1999

ISSUE NUMBER: 158 TITLE: PERF OF SAFETY-RELATED POWER-OPERATED VALVES UNDER DESIGN BASIS CONDIT

IDENTIFICATION DATE: 09/1991C PRIORITIZATION DATE: 01/1994C TYPE: GSI PRIORITY: M

TASK MANAGER: A. SERKIZ OFFICE/DIVISION/BRANCH: RES /DET /ERAB ACTION LEVEL: ACTIVE STATUS:

TAC NUMBERS: RESOLUTION DATE: 08/1999C

WORK AUTHORIZATION: MEMORANDUM FROM E. BECKJORD TO J. MURPHY ON JANUARY 26, 1994

FIN	CONTRACTOR	CONTRACT TITLE
W6354	INEL	POWER OPERATED VALVE PERFORMANCE

WORK SCOPE

AIR-OPERATED VALVES, SOLENOID-OPERATED VALVES, AND HYDRAULIC-OPERATED VALVES HAVE NOT BEEN SUBJECTED TO A COMPREHENSIVE RISK EVALUATION; HENCE, THESE VALVES LACK THE REGULATORY SCRUTINY APPLIED TO THE OTHER EQUIPMENT. THIS PROGRAM WILL DETERMINE IF THERE ARE SAFETY-RELATED PROBLEMS, THE SOLUTION OF THE PROBLEMS, AND WHETHER IMPLEMENTATION OF THE SOLUTIONS ARE COST BENEFICIAL.

STATUS

PRELIMINARY DRAFT REPORT WAS SUBMITTED BY THE CONTRACTOR AND REVIEWED BY NRR. DRAFT RESOLUTION SENT TO ACRS ON 04-05-99. QUESTIONS WERE RECEIVED FROM THE ACRS IN A LETTER TO THE EDO DATED 05/14/1999 RELATED TO CLOSURE OF THE GSI. THE STAFF RESPONSE TO THE 05/14/1999 ACRS LETTER WAS ISSUED ON 07/02/1999. THE STAFF AGREED THAT ADDITIONAL REGULATORY ACTION WAS NECESSARY BUT CONCLUDED THAT EXISTING REGULATIONS PROVIDE AN ADEQUATE FRAMEWORK FOR ANY NEEDED REGULATORY ACTION.

AFFECTED DOCUMENTS	PROBLEM/RESOLUTION
*** TO BE DETERMINED ***	NONE.

MILESTONES	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
QUICK REVIEW COMPLETE	04/1994	--	04/1994

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: 163 TITLE: MULTIPLE STEAM GENERATOR TUBE LEAKAGE

IDENTIFICATION DATE: 06/1992C PRIORITIZATION DATE: 01/1997C TYPE: GSI PRIORITY: H

TASK MANAGER: J. ANDERSEN OFFICE/DIVISION/BRANCH: NRR /DE /EMCB ACTION LEVEL: ACTIVE STATUS:

TAC NUMBERS: RESOLUTION DATE: _____

WORK AUTHORIZATION: JANUARY 17, 1997, MEMORANDUM FROM H. THOMPSON TO D. MORRISON

FIN	CONTRACTOR	CONTRACT TITLE

W O R K S C O P E

THIS ISSUE ADDRESSES THE SAFETY CONCERN ASSOCIATED WITH MULTIPLE STEAM GENERATOR TUBE LEAKS DURING A MAIN STEAM LINE BREAK THAT CANNOT BE ISOLATED. IT WAS OPENED IN RESPONSE TO A DPV FILED IN LATE 1991. THE DPV (AND LATER DPO) ISSUES ARE BEING CONSIDERED IN THE STAFF'S WORK ON STEAM GENERATOR TUBE INTEGRITY. THE NRC ORIGINALLY PLANNED TO DEVELOP A RULE PERTAINING TO STEAM GENERATOR TUBE INTEGRITY. THE PROPOSED RULE WAS TO IMPLEMENT A MORE FLEXIBLE REGULATORY FRAMEWORK FOR STEAM GENERATOR SURVEILLANCE AND MAINTENANCE ACTIVITIES THAT ALLOWS A DEGRADATION-SPECIFIC MANAGEMENT APPROACH. THE REGULATORY ANALYSIS CONCLUDED THAT THE MORE OPTIMAL REGULATORY APPROACH WAS TO UTILIZE A GENERIC LETTER. THE NRC STAFF SUGGESTED, AND THE COMMISSION SUBSEQUENTLY APPROVED, A REVISION TO THE REGULATORY APPROACH TO UTILIZE A GENERIC LETTER. FINALLY, IN LATE 1998, THE REGULATORY APPROACH WAS REVISED ONCE AGAIN. THE STAFF IS NOW WORKING TO RESOLVE CONCERNS WITH THE INDUSTRY INITIATIVE, NEI 97-06, IN LIEU OF A GENERIC LETTER. THE CURRENT FRAMEWORK PROVIDES REASONABLE ASSURANCE THAT OPERATING PWR'S ARE SAFE. HOWEVER, THE CURRENT REGULATORY FRAMEWORK HAS SHORTCOMINGS. TO RESOLVE THESE SHORTCOMINGS, THE STAFF IS WORKING WITH INDUSTRY TO REVISE THE REGULATORY FRAMEWORK TO UTILIZE A RISK-INFORMED AND PERFORMANCE-BASED APPROACH THAT WILL ENSURE COMPLIANCE WITH CURRENT REGULATIONS (I.E., GDC, APPENDIX B, ASME CODE, PART 100).

S T A T U S

THE STAFF COMPLETED A DRAFT RISK ASSESSMENT AND DRAFT REGULATORY ANALYSIS AND MET WITH ACRS ON MARCH 4, 5, AND APRIL 3, 1997, TO DISCUSS THE TWO EFFORTS. THE RESULTS OF THESE TWO EFFORTS CAUSED THE STAFF TO CONCLUDE THAT GENERIC REGULATORY ACTION IN THE FORM OF A RULE WAS NOT NECESSARY. THE STAFF SUBSEQUENTLY DRAFTED AND SENT TO THE COMMISSION COMSECY097-013 (05-23-1997) WHICH DISCUSSED THE BASIS FOR REVISING THE REGULATORY APPROACH TO UTILIZE A GENERIC LETTER. THE COMMISSION APPROVED THE REVISED REGULATORY APPROACH IN THE SRM DATED 06-30-1997.

THE DPO ISSUES DOCUMENT WAS COMPLETED AND SENT TO THE ACRS FULL COMMITTEE FOR REVIEW IN OCTOBER 1997. THE STAFF MET WITH CRGR ON 06-12-1998 FOR AN INFORMATION BRIEFING ON PACKAGE. THE STAFF MET WITH CRGR ON 07-21-1998 FOR A DETAILED REVIEW OF THE PROPOSED GL PACKAGE. THE STAFF ISSUED COMMISSION PAPER SECY-98-248 WITH THE RECOMMENDATION TO PUT A HOLD ON THE ISSUANCE OF A GL WHILE THE STAFF WORKS WITH THE INDUSTRY ON NEI 97-06 (THE PROPOSED ALTERNATIVE TO A GL). THE COMMISSION AGREED WITH THIS APPROACH IN AN SRM DATED 12-21-1998.

ON 01-20-99, THE STAFF ISSUED THE DPO RESOLUTION DOCUMENT FOR PUBLIC COMMENT. THE DPO RESOLUTION DOCUMENT HAS BEEN UPDATED TO REFLECT THE STATUS OF THE NEI 97-06 INDUSTRY INITIATIVE AND HAS BEEN FORWARDED TO THE EDO. RESOLUTION OF

ISSUE NUMBER: 168 TITLE: ENVIRONMENTAL QUALIFICATION OF ELECTRICAL EQUIPMENT

IDENTIFICATION DATE: 04/1993C PRIORITIZATION DATE: 06/1993C TYPE: GSI PRIORITY: H

TASK MANAGER: S. AGGARWAL OFFICE/DIVISION/BRANCH: RES /DET /MEB ACTION LEVEL: ACTIVE STATUS:

TAC NUMBERS: M85648 RESOLUTION DATE: 12/2000

WORK AUTHORIZATION: APRIL 1, 1993, MEMORANDUM FROM T. MURLEY TO J. SNIEZEK

FIN	CONTRACTOR	CONTRACT TITLE
A1818	SNL	LOCA TESTING OF CONNECTORS
A2336	ANL	RISK IMPACT OF EQ
E2097	SCIENTECH	EQ FOR OPERATING REACTORS
W6169	BNL	LITERATURE REVIEW
W6465	BNL	LOCA TESTING OF LOW VOLTAGE I&C CABLES

W O R K S C O P E

1. TO GATHER, REVIEW, AND EVALUATE OPERATING EXPERIENCE DATA AND EQUIPMENT REPLACEMENT SCHEDULES FOR NUCLEAR POWER PLANTS TO PROVIDE INSIGHT AS TO WHERE NRC SHOULD FOCUS ITS RESOURCES IN THE PERFORMANCE OF EQ AGING REVIEWS.
2. TO PERFORM A DETAILED ASSESSMENT OF THE RISK ASSOCIATED WITH EQ ISSUES.

IN ACCORDANCE WITH THE 5/5/94 MEMORANDUM TO THE NRR DIRECTOR FROM THE RES DIRECTOR, RESOLUTION WILL INCLUDE CONSIDERATION OF A LICENSE RENEWAL PERIOD OF 20 YEARS. THE CURRENT SCOPE OF THIS GSI IS LIMITED TO TWO REPRESENTATIVE GROUPS OF LOW-VOLTAGE I&C SAFETY-RELATED CABLES.

S T A T U S

DETAILED STATUS INFORMATION FOR GI-168 WAS SUBMITTED TO THE COMMISSION IN A MEMORANDUM DATED FEBRUARY 5, 1998. THE DRAFT PROGRAM REVIEW REPORT WAS ISSUED IN APRIL 1996 FOR DATA COLLECTION AND ANALYSIS. THE REVIEW OF EQ-RELATED PUBLISHED DOCUMENTS AND INDUSTRY REPORTS, REVIEW OF EQUIPMENT FAILURES ASSOCIATED WITH THE ACCIDENT AT TMI-2, AND THE DEVELOPMENT OF AN INTEGRATED DATABASE OF EQ TEST REPORTS, RESEARCH TESTS, AND OTHER TEST ACTIVITIES ARE COMPLETED. A FINAL REPORT, NUREG/CR-6384, ISSUED IN APRIL 1996, CONCLUDED THAT 19 UNRESOLVED ISSUES WITHIN THE SCOPE OF PLANNED RESEARCH CAN BE REDUCED TO SIX. PLANNED RESEARCH TO INVESTIGATE UNCERTAINTIES WITH ACCELERATED AGING AND TO REVIEW PROMISING CABLE CONDITION MONITORING METHODS BEGAN IN MAY 1996. WYLE LABORATORY WAS SELECTED TO PERFORM AGING AND LOCA TESTING. THREE LOCA TESTS HAVE BEEN COMPLETED. A TOTAL OF 6 LOCA TESTS ARE PLANNED (TO BE COMPLETED BY APRIL 2000). ISSUE TRANSFERRED FROM NRR TO RES IN FEBRUARY 1998.

ISSUE NUMBER: 170 TITLE: REACTIVITY TRANSIENTS AND FUEL DAMAGE CRITERIA FOR HIGH BURNUP FUEL

IDENTIFICATION DATE: 01/1995C PRIORITIZATION DATE: 01/1995C TYPE: GSI PRIORITY: H

TASK MANAGER: R. MEYER OFFICE/DIVISION/BRANCH: RES /DSARE/SMSAB ACTION LEVEL: ACTIVE STATUS:

TAC NUMBERS: M91256 RESOLUTION DATE: _____

WORK AUTHORIZATION: MEMO TO COMMISSIONERS FROM J. TAYLOR ON 11/9/94, "REACTIVITY TRANSIENTS AND FUEL DAMAGE CRITERIA FOR HIGH BURNUP FUEL"

FIN	CONTRACTOR	CONTRACT TITLE
W6194	INEL	
W6200	PNL	DATA REVIEW AND MODEL DEVELOPMENT FOR HIGH BURNUP FUEL
W6382	BNL	ANALYSIS OF REACTIVITY TRANSIENTS
W6483	ANL	CLADDING METALLURGY OF HIGH BURNUP FUEL
W6497	CEA	CABRI HIGH BURNUP FUEL TEST (COOPERATIVE PROGRAM, FRANCE)
W6500	RRC	IGR HIGH BURNUP FUEL TEST (COOPERATIVE PROGRAM, RUSSIA)

W O R K S C O P E

AFTER A PRELIMINARY SAFETY ASSESSMENT WAS PERFORMED, AN ACTION PLAN WAS DEVELOPED THAT ADOPTED AN EARLIER USER NEED LETTER AND DEFINED ADDITIONAL ACTIONS TO ASSESS THE ADEQUACY OF FUEL DAMAGE CRITERIA AT HIGH BURNUPS. RESEARCH WILL BE PERFORMED, AND RESULTS FROM INTERNATIONAL RESEARCH PROGRAMS WILL BE ANALYZED. DISCUSSIONS WILL BE HELD WITH INDUSTRY REPRESENTATIVES TO OBTAIN THEIR COOPERATION IN ASSESSING SAFETY EVALUATIONS. CONFIRMATION OR REVISION OF FUEL DAMAGE CRITERIA WILL BE OBTAINED AS A RESULT OF THIS WORK, AND THE NEED FOR ANY REGULATORY ACTIONS WILL BE DETERMINED.

S T A T U S

A COMPREHENSIVE PROGRAM PLAN FOR ADDRESSING ISSUES RELATED TO HIGH-BURNUP FUEL WAS REVIEWED WITH THE ACRS IN APRIL AND JUNE 1998 AND ISSUED IN A MEMORANDUM TO THE COMMISSIONERS ON JULY 6, 1998. NO BACKFIT ANALYSES ARE BEING SOUGHT. NRC RESEARCH WILL BE PERFORMED TO CONFIRM PREVIOUS DECISIONS TO PERMIT BURNUPS TO 62 GWD/T. NO EXTENSIONS BEYOND THAT BURNUP WILL BE CONSIDERED UNTIL SUBSTANTIALLY MORE RESEARCH IS COMPLETED BY THE INDUSTRY.

AFFECTED DOCUMENTS

REGULATORY GUIDE 1.77
SRP SECTION 4.2
10 CFR 50.46

PROBLEM/RESOLUTION

TESTS IN FRANCE AND JAPAN HAVE SHOWN THAT ACCEPTANCE CRITERIA IN REGULATORY GUIDE 1.77 FOR PWR REACTIVITY ACCIDENTS ARE NOT ADEQUATE FOR HIGH-BURNUP FUEL. OTHER

ISSUE NUMBER: 170 TITLE: REACTIVITY TRANSIENTS AND FUEL DAMAGE CRITERIA FOR HIGH BURNUP FUEL

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
ASSESS EFFECTS ON DESIGN BASIS ACCIDENTS OF REDUCED FAILURE THRESHOLD FOR HIGH BURNUP FUEL	09/1995	--	09/1995
WATER REACTOR SAFETY INFORMATION MEETING ON HIGH BURNUP	10/1995	--	10/1995
COMMISSION PAPER ON HIGH BURNUP FUEL	03/1996	--	03/1996
ACRS BRIEFING BY RES	04/1996	--	04/1996
CSNI EXPERTS MEETING ON TRANSIENT BEHAVIOR OF HIGH BURNUP FUEL	05/1996	--	05/1996
INITIATE RESEARCH PROGRAM ON LOCA CRITERIA AT HIGH BURNUP	09/1996	--	09/1996
WATER REACTOR SAFETY INFORMATION MEETING ON HIGH BURNUP FUEL	10/1996	--	10/1996
CORE PERFORMANCE ISSUES WORKSHOP WITH INDUSTRY	10/1996	--	10/1996
COMMISSION PAPER ON HIGH BURNUP FUEL ISSUES	11/1996	--	11/1996
RIL WITH TECH BASIS FOR INTERIM RIA ACCEPTANCE CRITERIA	12/1996	--	03/1997
COMMISSION BRIEFING ON HIGH BURNUP FUEL	03/1997	--	03/1997
PARTIAL RESPONSE TO SRM FROM COMMISSION BRIEFING OF 3/97 (POINT OF CONTACT)	04/1997	--	04/1997
PARTIAL RESPONSE TO SRM FROM COMMISSION BRIEFING OF 3/97 (REG CRITERIA)	06/1997	--	07/1997
ISSUE LETTER TO NEI ASSESSING INDUSTRY ACTIONS	09/1996	--	08/1997

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: 170 TITLE: REACTIVITY TRANSIENTS AND FUEL DAMAGE CRITERIA FOR HIGH BURNUP FUEL

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
RECEIVE FIRST SHIPMENT OF COMMERCIAL HIGH BURNUP FUEL RODS FOR LOCA TESTING	10/1998	09/1999	--
BEGIN 2-YEAR TEST PROGRAM ON LOCA-RELATED OXIDATION KINETICS	12/1999	12/1999	--
SIGN FRENCH AGREEMENT FOR 5-YEAR TEST PROGRAM ON RIA CRITERIA IN CABRI TEST REACTOR	12/1999	12/1999	--
BEGIN CONSTRUCTION OF WATER LOOP IN FRENCH CABRI TEST REACTOR FOR REACTIVITY ACCIDENT TESTING	09/1999	09/2000	--

ISSUE NUMBER: 172 TITLE: MULTIPLE SYSTEM RESPONSES PROGRAM

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
ISSUE REPORT ON HOW MSRP ISSUES WERE ADDRESSED IN IPE PROGRAM	12/1998	12/1999	--
ISSUE REPORT ON HOW MSRP ISSUES WERE ADDRESSED IN IPEEE PROGRAM	12/1998	--	--
MEET WITH ACRS	12/1998	--	--
CLOSE OUT ISSUE WITH MEMO TO THE EDO	12/1998	--	--

ISSUE NUMBER: 173.A TITLE: SPENT FUEL STORAGE POOL: OPERATING FACILITIES

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
REVIEW EXISTING NRC GUIDANCE AND REQUIREMENTS	08/1994	--	08/1994
IDENTIFY SIGNIFICANT SFP CONCERNS	12/1994	--	12/1994
REPORT SIGNIFICANT SFP PROBLEMS TO NRR MANAGEMENT	12/1994	--	12/1994
DEVELOP AN SFP INSPECTION PLAN	01/1995	--	01/1995
CONDUCT INSPECTIONS OF SELECTED PLANTS	06/1995	--	06/1995
EVALUATE AND REPORT RESULTS OF INSPECTIONS	09/1995	--	09/1995
COMMISSION BRIEFING ON SFP ISSUES	02/1996	--	02/1996
COMMISSION BRIEFING ON SFP ISSUES	04/1996	--	04/1996
ASSESS RISK/SIGNIFICANCE OF INDIVIDUAL CONCERNS	06/1996	--	07/1996
ASSESS MONITORING OF POTENTIAL OFFSITE RELEASES	06/1996	--	07/1996
ASSESS RADIOACTIVE MATERIAL STORAGE PRACTICES	06/1996	--	07/1996
PROPOSED COURSE OF ACTION SUBMITTED TO COMMISSION	06/1996	--	07/1996
COMMISSION MEETING FOR RESOLUTION	08/1996	--	08/1996
ACRS MEETING	08/1996	--	08/1996
REG. ANALYSIS FOR RULE	05/1997	--	07/1997
PLANT-SPECIFIC REG. ANALYSIS FOR HARDWARE BACKFITS AT SELECTED SITES	05/1997	--	12/1997
REVISE R.G. 1.13	10/1998	08/2000	--

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: 190 TITLE: FATIGUE EVALUATION OF METAL COMPONENTS FOR 60-YEAR PLANT LIFE

IDENTIFICATION DATE: 08/1996C PRIORITIZATION DATE: 08/1996C TYPE: GS1 PRIORITY: H

TASK MANAGER: K. SHAUKAT OFFICE/DIVISION/BRANCH: RES /DET /MEB ACTION LEVEL: ACTIVE STATUS:

TAC NUMBERS: RESOLUTION DATE: 12/1999

WORK AUTHORIZATION: MEMO TO A. THADANI FROM T. SPEIS ON 08/26/96, "GSI-166, 'ADEQUACY OF FATIGUE LIFE OF METAL COMPONENTS'"

FIN	CONTRACTOR	CONTRACT TITLE
W6671	PNL	FATIGUE ANALYSIS OF SELECTED COMPONENTS FOR 60-YEAR PLANT LIFE

W O R K S C O P E

EVALUATE FATIGUE ADEQUACY OF OPERATING PLANT COMPONENTS. IN ACCORDANCE WITH 05/05/94 MEMORANDUM TO THE NRR DIRECTOR FROM THE RES DIRECTOR, RESOLUTION WILL INCLUDE CONSIDERATION OF A 60-YEAR PROJECTED PLANT LIFE.

S T A T U S

FATIGUE ANALYSIS OF COMPONENTS FOR 60-YEAR PLANT LIFE IS UNDERWAY. THIS WORK COMPLETES THE ONGOING WORK UNDER GI-78, AND THE RESIDUAL WORK ON GI-166 WILL BE CARRIED ON UNDER THIS GENERIC ISSUE.

A REPORT, "FATIGUE ANALYSIS OF COMPONENTS FOR 60-YEAR PLANT LIFE" IS COMPLETED MAKING USE OF UPDATED FATIGUE DESIGN CURVES FOR STAINLESS STEELS DEVELOPED BY ARGONNE NATIONAL LABORATORY. THIS REPORT PROVIDED THE RESULTS IN THE FORM OF PROBABILITY OF THROUGH-WALL CRACKING FOR SELECTED COMPONENTS, AND ITS EFFECT ON CORE DAMAGE FREQUENCY. AN ACRS MEETING IS PLANNED IN COOPERATION WITH NRR. FOLLOWING THE ACRS MEETING AND ITS RECOMMENDATIONS, THE ISSUE IS EXPECTED TO BE CLOSED OUT.

AFFECTED DOCUMENTS	PROBLEM/RESOLUTION
NUREG/CR-6260	NONE.

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
RES DECISION TO ESTABLISH NEW GSI	08/1996	--	08/1996

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: 191 TITLE: ASSESSMENT OF DEBRIS ACCUMULATION ON PWR SUMP PERFORMANCE

IDENTIFICATION DATE: 09/1996C PRIORITIZATION DATE: 09/1996C TYPE: GSI PRIORITY: H

TASK MANAGER: M. MARSHALL OFFICE/DIVISION/BRANCH: RES /DET /ERAB ACTION LEVEL: ACTIVE STATUS:

TAC NUMBERS: RESOLUTION DATE: 09/2001

WORK AUTHORIZATION: MEMO TO D. MORRISON FROM W. RUSSELL, "THIRD SUPPLEMENTAL USER NEED REQUEST...ACCIDENT GENERATED DEBRIS," 12/07/95

FIN	CONTRACTOR	CONTRACT TITLE
W6650	SEA	TECHNICAL ASSISTANCE IN RESOLVING GENERIC SAFETY ISSUES
Y6041	LANL	ASSESSMENT OF DEBRIS ACCUMULATION ON PRESSURIZED WATER REACTORS SUMP PERFORMANCE

W O R K S C O P E

THE GOALS OF THE NRC'S REASSESSMENT ARE TO (1) DETERMINE IF THE TRANSPORT AND ACCUMULATION OF DEBRIS IN CONTAINMENT FOLLOWING A LOCA WILL IMPEDE THE OPERATION OF THE ECCS IN OPERATING PWRS, (2) IF IT IS SHOWN THAT DEBRIS ACCUMULATION WILL IMPEDE ECCS OPERATION, DEVELOP THE TECHNICAL BASIS FOR REVISING NRC'S REGULATIONS OR GUIDANCE TO ENSURE THAT DEBRIS ACCUMULATION IN CONTAINMENT WILL NOT PREVENT ECCS OPERATON, AND (3) IF IT IS SHOWN THAT DEBRIS ACCUMULATION WILL IMPEDE ECCS OPERATION, PROVIDE NRC TECHNICAL REVIEWERS WITH SUFFICIENT INFORMATION ON PHENOMENA INVOLVED IN DEBRIS ACCUMULATION AND HOW IT EFFECTS ECCS OPERATION TO FACILITATE THE REVIEW OF ANY CHANGES TO PLANTS THAT MAY BE WARRANTED.

S T A T U S

PLANS ARE UNDERWAY TO BEGIN DEBRIS TRANSPORT TESTS AND TO APPLY DEBRIS TRANSPORT MODEL (APPROACH) TO A REFERENCE PLANT. COLLECTION AND REVIEW OF INFORMATION ON CONTAINMENT AND SUMP DESIGNS WILL CONTINUE.

AFFECTED DOCUMENTS

(1) REGULATORY GUIDE 1.82, REV. 2
(2) NUREG-0800
(3) GENERIC LETTER 85-22

PROBLEM/RESOLUTION

NONE.

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: B-17 TITLE: CRITERIA FOR SAFETY-RELATED OPERATOR ACTIONS

IDENTIFICATION DATE: 06/1978C PRIORITIZATION DATE: 11/1983C TYPE: GSI PRIORITY: M

TASK MANAGER: P. LEWIS OFFICE/DIVISION/BRANCH: RES /DSARE/REAHFB ACTION LEVEL: ACTIVE STATUS:

TAC NUMBERS: RESOLUTION DATE: 03/2000

WORK AUTHORIZATION: NRR FY-84 OPERATING PLAN (APPENDIX G)

FIN	CONTRACTOR	CONTRACT TITLE
A0465	LLNL	INTEGRATION OF HUMAN PERFORMANCE FACTORS FULLY INTO THE RELIABILITY EVALUATION PROCESS (TASKS 5 AND 6)

W O R K S C O P E

THE WORK SCOPE INCLUDES: (1) DEVELOP TIME CRITERION FOR SAFETY RELATED OPERATOR ACTIONS; (2) DETERMINE WHETHER OR NOT AUTOMATIC ACTUATION WILL BE REQUIRED; AND (3) DEVELOP GUIDANCE AND REVIEW CRITERIA FOR LICENSEE PROPOSALS TO USE MANUAL OPERATOR ACTION OR AUTOMATIC ACTION. THE SCOPE OF THIS GENERIC ISSUE COVERS DESIGN BASIS ACCIDENTS OR EVENTS. CRITERIA FOR SAFETY RELATED OPERATOR ACTIONS DURING SEVERE ACCIDENTS WILL BE COVERED UNDER THE SEVERE ACCIDENT PROGRAM.

S T A T U S

THIS ISSUE WAS TRANSFERRED FROM NRR TO RES WITH THE 4/87 NRC REORGANIZATION (4/10/87 MEMO FROM W. RUSSELL TO T. SPEIS). NO WORK HAD BEEN COMPLETED ON THE ISSUE PRIOR TO TRANSFER.

DURING 1983 AND 1984, RES WORKED ON THIS ISSUE. DURING THAT TIME DATA WERE COLLECTED BY RES CONCERNING OPERATOR RESPONSE TIMES. ANSI/ANS 58.8 WAS PUBLISHED IN SEPTEMBER 1984. THE GENERIC ISSUE WAS LATER TRANSFERRED TO NRR. NO WORK WAS COMPLETED BY NRR ON THIS ISSUE. WORK HAS BEGUN WHICH SHOULD LEAD TO RESOLUTION OF THIS GSI BY ENDORSING ANSI/ANS 58.8. THIS STANDARD IS BEING REVISED BASED ON NEW INDUSTRY DATA. LLNL BEGAN WORK TO MANAGE THE TECHNICAL EFFORT INVOLVING EPRI AND A SUBCONTRACTOR TO EPRI IN MARCH 1989.

THIS ITEM HAD A PROPOSED FINAL RESOLUTION (I.E., RG 1.164 THAT ENDORSED ANS-58.8) THAT WAS NOT ACCEPTED BY ACRS, AS STATED IN THEIR LETTER OF NOVEMBER 14, 1995. TO HELP RESOLVE THE ACRS COMMENTS, AN ADDITIONAL ROUND OF PUBLIC COMMENT WAS DONE IN 1996 AND, BASED UPON FURTHER DISCUSSIONS WITH ACRS AND NRR, A NEW APPROACH (CONSIDERING ALTERNATIVES TO ENDORSING ANS-58.8) IS BEING TAKEN.

A DRAFT SOW WAS PLACED IN CONCURRENCE IN SEPTEMBER 1998. BNL BEGAN WORK ON 04-01-99. A PROPOSAL FOR CLOSING OUT THE ISSUE WAS DRAFTED BY THE TASK MANAGER IN 09/1999.

AFFECTED DOCUMENTS

TO BE PROVIDED LATER

PROBLEM/RESOLUTION

NONE.

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: B-17 TITLE: CRITERIA FOR SAFETY-RELATED OPERATOR ACTIONS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
ACRS REVIEW	08/1999	12/1999	--
FINAL NUREG	12/1999	03/2000	--
CLOSE OUT ISSUE WITH MEMO TO THE EDO	09/1994	03/2000	--

ISSUE NUMBER: B-55 TITLE: IMPROVE RELIABILITY OF TARGET ROCK SAFETY-RELIEF VALVES

AFFECTED DOCUMENTS	PROBLEM/RESOLUTION
NONE	NONE.

MILESTONES	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
MEETING WITH BWR OWNERS SETPOINT DRIFT COMMITTEE	12/1983	--	11/1983
BWR OWNERS REPORT SUBMITTED	07/1984	--	06/1984
BOSTON EDISON REPORT SUBMITTED	--	--	08/1984
COMPLETION OF NRR EVALUATION OF OWNERS GROUP AND BOSTON EDISON REPORTS	10/1984	--	10/1984
COMMUNICATION TO OG RE: NRC COMMENTS/CONCERNS	--	--	03/1985
OG PRESENTATION TO THE STAFF	10/1985	--	10/1985
STAFF RECEIVES OWNERS' PROGRAM FOR ELIMINATING SET POINT DRIFT	10/1985	--	10/1985
OWNERS BEGIN TO INSTALL NEW PILOT DISKS TO TEST IN PLANT FOR ONE FUEL CYCLE	01/1986	--	01/1986
ISSUE STATUS REPORT ON INDUSTRY EFFORTS	12/1987	--	01/1988
ISSUE PROGRESS REPORT ON INDUSTRY EFFORTS	11/1988	--	11/1988
BWROG APPROVAL FOR FUNDING CATALYST/PRESSURE SWITCH PROGRAM	06/1990	--	06/1990
BWROG SELECT/DESIGN CATALYST	03/1991	--	02/1991
NRC STAFF ISSUE STATUS REPORT ON INDUSTRY PROGRESS	06/1991	--	08/1991
BWROG COMPLETES FINALIZING DESIGN AND LABORATORY TESTING	04/1992	--	07/1992

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: NMSS-0007 TITLE: CRITICALITY BENCHMARKS GREATER THAN 5% ENRICHMENT

IDENTIFICATION DATE: 05/1998C PRIORITIZATION DATE: 05/1998C TYPE: GSI PRIORITY: L

TASK MANAGER: D. DAMON OFFICE/DIVISION/BRANCH: NMSS/FCSS/ ACTION LEVEL: ACTIVE STATUS:

TAC NUMBERS: RESOLUTION DATE: _____

WORK AUTHORIZATION:

FIN	CONTRACTOR	CONTRACT TITLE
W6479	ORNL	DEVELOPMENT AND APPLICABILITY OF CRITICALITY SAFETY SOFTWARE FOR LICENSING REVIEW

W O R K S C O P E

THE IMPORTANCE OF SOFTWARE (METHODS AND DATA) IN ESTABLISHING THE CRITICALITY SAFETY OF SYSTEMS WITH FISSILE MATERIAL IS INCREASING AS LICENSEES WORK TO OPTIMIZE FACILITIES AND STORAGE/TRANSPORT PACKAGES AT THE SAME TIME THAT ACCESS TO EXPERIMENTAL DATA IS DECREASING. AVAILABLE EXPERIMENTAL DATA ARE INSUFFICIENT TO VALIDATE NUCLEAR CRITICALITY SAFETY EVALUATIONS FOR ALL REQ'D CONFIGURATIONS AT UR-235 ENRICHMENTS IN THE RANGE OF 5-20%. THE PURPOSE OF THIS PROJECT IS TO DEVELOP AND CONFIRM THE ADEQUACY OF METHODS, ANALYTICAL TOOLS, AND GUIDANCE FOR CRITICALITY SAFETY SOFTWARE TO BE USED IN LICENSING NUCLEAR FACILITIES. THE CONTRACTOR WILL DEVELOP AND TEST METHODS TO ESTIMATE TRENDS IN CALCULATIONAL BIAS AND UNCERTAINTY (THUS EXTENDING THE RANGE OF APPLICABILITY) USING SENSITIVITY ANALYSIS TECHNIQUES THAT RELATE THE IMPORTANCE OF THE SYSTEM PARAMETERS TO THE CALCULATED NEUTRON MULTIPLICATION FACTOR; PROVIDE EXPERT GUIDANCE ON ASSESSING THE ADEQUACY OF THE PARAMETER PHASE SPACE USED IN THE VALIDATION PROCESS AND THE RESULTING BIAS AND UNCERTAINTY; AND ILLUSTRATE USE OF THE GUIDANCE BY APPLICATION TO A REGIME OF EXPERIMENTAL PHASE SPACE (SUCH AS 5-10% U-235 AND DEGREE OF MODERATION) THAT HAS LIMITED MEASURED DATA BUT EXTENSIVE INTEREST IN TERMS OF CURRENT AND PLANNED SAFETY EVALUATIONS.

A NEW STATEMENT OF WORK MAY NEED TO BE DEVELOPED FOR OTHER CONTRACT WORK (E.G., APPLYING THE METHODS DEVELOPED TO DETERMINE MARGINS OF SAFETY FOR ACTUAL SCENARIOS, TRAINING NRC PERSONNEL ON THE CONTRACT METHODS, REVISING NATIONAL STANDARDS, OR PERFORMING BENCHMARK EXPERIMENTS).

S T A T U S

THE PART OF THE ORNL RESEARCH PROJECT DEALING WITH THE ISSUE OF VALIDATION OF PROCESS CALCULATIONS WHERE APPLICABLE BENCHMARKS ARE QUESTIONABLE, SUCH AS THE CASE FOR ENRICHMENTS AT 5 TO 20 %, IS COMPLETE.
ORNL COMPLETED THE DEVELOPMENT AND APPLICATION OF METHODS TO ADDRESS THIS SITUATION. ORNL HAS PRODUCED FOUR TECHNICAL REPORTS DOCUMENTING THE METHOD AND APPLYING IT TO A NUMBER OF SPECIFIC CASES. FCSS CRITICALITY SAFETY STAFF REVIEWED THESE REPORTS AND PROVIDED COMMENTS ON THE TWO MAJOR METHODS DOCUMENTS TO ORNL. REVISED (FINAL) REPORTS HAVE BEEN SUBMITTED AND ARE BEING PREPARED FOR PUBLICATION. THE FOUR REPORTS COVERED THE FOLLOWING SUBJECTS: (1) METHODOLOGY FOR DEFINING RANGE OF APPLICABILITY INCLUDING EXTENSIONS OF ENRICHMENTS FROM 5% TO 11%; (2) TEST APPLICATIONS AND RESULTS OF THE METHOD; (3) TEST APPLICATION FOR HIGHER ENRICHMENTS USING FOREIGN EXPERIMENTS; (4) FEASIBILITY STUDY FOR EXTENDING THE METHOD TO MULTI-DIMENSIONAL ANALYSES, SUCH AS TRANSPORT CASKS AND REACTOR FUEL. THE CURRENT METHOD CAN ONLY BE USED FOR ONE-DIMENSIONAL GEOMETRIES.

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: NMSS-0007 TITLE: CRITICALITY BENCHMARKS GREATER THAN 5% ENRICHMENT

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
DEVELOPMENT OF GENERALIZED SENSITIVITY METHODS	12/1997	--	12/1997
ACQUISITION AND DOCUMENTATION OF RUSSIAN DATA	05/1998	--	05/1998
DEVELOPMENT OF GUIDANCE FOR DEFINING RANGES OF APPLICABILITY	07/1998	--	11/1998
APPLICATION OF GUIDANCE TO EXTEND LOW-ENRICHMENT RANGE	09/1998	--	11/1998
TECHNICAL ASSISTANCE AND PROJECT PLANNING	03/1999	--	03/1999
RECEIVE FINAL ORNL CONTRACT REPORT	03/1999	--	10/1999
EVALUATE FINAL ORNL CONTRACT REPORT	10/1999	02/2000	--
DETERMINE IF USER NEEDS HAVE BEEN MET BY ORNL CONTRACT	11/2000	11/2000	--
CLOSE OUT ISSUE	--	--	--

ISSUE NUMBER: NMSS-0008 TITLE: YEAR 2000 COMPUTER PROBLEM - NONREACTOR LICENSEES

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
INFORMATION NOTICE 96-70	12/1996	--	12/1996
INFORMATION NOTICE 97-61	07/1997	--	07/1997
NMSS NEWSLETTER ARTICLE	09/1997	--	09/1997
ESTABLISH Y2K TEAM	09/1997	--	09/1997
MEMO TO INSPECTORS (ASK Y2K QUESTIONS)	11/1997	--	11/1997
COMPLETE SITE VISITS	12/1997	--	12/1997
HPS ARTICLE	01/1998	--	01/1998
GENERIC LETTER REQUESTING RESPONSE FROM FUEL CYCLE LICENSEES (GL 98-03)	05/1998	--	06/1998
REVISE INSPECTION PROCEDURES IP87111/IP87119	05/1998	--	06/1998
INFORMATION NOTICE 98-30	06/1998	--	06/1998
INFORMATION NOTICE UPDATING LICENSEES ON NRC ACTIVITIES (IN 99-18)	06/1999	--	06/1999
INFORMATION NOTICE RECOMMENDING Y2K CONTINGENCY PLANS (IN 99-20)	06/1999	--	06/1999
PARTICIPATE IN INTERNATIONAL Y2K MEETING SPONSORED BY IAEA	07/1999	--	07/1999
FOLLOW-UP INSPECTION OF GASEOUS DIFFUSION PLANTS	09/1999	--	09/1999
Y2K CONTINGENCY PLAN EXERCISE	10/1999	--	10/1999
CRITIQUE CONTINGENCY PLAN EXERCISE	11/1999	11/1999	--
CLOSE OUT ISSUE	01/2000	01/2000	--

AS OF FY: 1999 QUARTER: 4

ISSUE NUMBER: NMSS-001.0 TITLE: TROXLER GAUGE SOURCE ROD WELD FAILURES

AFFECTED DOCUMENTS

PROBLEM/RESOLUTION

TROXLER'S PRACTICE OF LABELING ALL SUBMITTALS "PROPRIETARY" HAS DELAYED THE SHARING OF THE METALLURGICAL REPORT AND OTHER INFORMATION WITH NRC BECAUSE OF THE ANALYSIS BY NORTH CAROLINA LEGAL STAFF FOR RELEASING PROPRIETARY INFORMATION.

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
NORTH CAROLINA ORDER ISSUED	04/1998	--	04/1998
TROXLER ISSUES CUSTOMER BULLETIN	07/1998	--	07/1998
TROXLER COMPLETES METALLURGICAL TESTING	07/1998	--	07/1998
TROXLER PROVIDES TESTING RESULTS TO STATE	07/1998	--	07/1998
TROXLER PROVIDES RESPONSE STATUS	10/1998	--	05/1999
IDENTIFY NRC LICENSEES THAT FAILED TO RESPOND	12/1999	12/1999	--
FOLLOW-UP WITH NRC LICENSEES THAT FAILED TO RESPOND	01/2000	01/2000	--
CONFIRM TROXLER REVISION OF OPERATING MANUAL	08/1999	01/2000	--
CLOSE OUT ISSUE	08/1999	03/2000	--

ISSUE NUMBER: NMSS-0014 TITLE: SURETY ESTIMATES FOR GROUNDWATER RESTORATION AT IN-SITU LEACH FIELDS

AFFECTED DOCUMENTS	PROBLEM/RESOLUTION
	TERMINATED.

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
PORE VOLUME - DATA EVALUATION (TASK 1)	12/1997	--	06/1998
COMMISSION RESPONSE TO SECY-99-013	08/1999	11/1999	--
DEVELOP PORE VOLUME METHODOLOGY (TASK 2)	06/1998	--	--
DEVELOP SURETY METHODOLOGY (TASK 3)	12/1998	--	--
SURETY TECHNOLOGY TRANSFER (TASK 4)	08/1999	--	--
STABILITY MONITORING - DATA EVALUATION (TASK 5)	06/1999	--	--
TESTING OF STABILITY METHODOLOGY (TASK 6)	09/1999	--	--
DEVELOP STABILITY METHODOLOGY (TASK 7)	11/1999	--	--
MONITOR TECHNOLOGY TRANSFER (TASK 8)	01/2000	--	--
CLOSE OUT ISSUE	--	--	--

ISSUE NUMBER: NMSS-0015 TITLE: ADEQUACY OF 10 CFR 150 CRITICALITY REQUIREMENTS

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
ISSUE REQUEST FOR BIDS	04/1998	--	05/1998
RECEIVE BIDS	05/1998	--	06/1998
AWARD CONTRACT	06/1998	--	06/1998
RECEIVE DRAFT REPORT	12/1998	--	03/1999
OBTAIN PUBLIC COMMENTS ON AGREEMENT STATE COMPATIBILITY	09/1999	10/1999	--
INFORM COMMISSION	10/1999	11/1999	--
CHANGE COMPATIBILITY	01/2000	01/2000	--
ACCEPT FINAL REPORT	--	--	--
IMPLEMENT NECESSARY REGULATORY CHANGES	--	--	--
CLOSE OUT ISSUE	--	--	--

ISSUE NUMBER: NMSS-0016 TITLE: ADEQUACY OF 0.05 WEIGHT PERCENT LIMIT IN 10 CFR 40

M I L E S T O N E S	ORIGINAL DATE	CURRENT DATE	ACTUAL DATE
ISSUE OPTIONS PAPER	07/1998	10/1999	--
RECEIVE SRM	--	--	--
RESPOND TO SRM	--	--	--
CLOSE OUT ISSUE	--	--	--