

James A. FitzPatrick
Nuclear Power Plant
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Michael J. Colomb
Site Executive Officer

November 24, 1999
JAFP-99-0312

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Mail Station P1-137
Washington, DC 20555

SUBJECT: James A. FitzPatrick Nuclear Power Plant
Docket No. 50-333
Additional Information Regarding Table 1 of Relief Request 17 (TAC M72418)

Reference: NYPA Letter, James Knubel to the NRC (JPN-99-025), "Proposed Alternatives in Accordance with 10CFR50.55a(a)(3)(i) for Reactor Pressure Vessel Circumferential Shell Weld Examinations," dated August 5, 1999.


Dear Sir:

This letter provides additional information to the referenced relief request. Table 1 of the proposed relief request listed values for $\Delta RT_{NDT}^{\circ F}$ and Mean $RT_{NDT}^{\circ F}$ that were based on $\frac{1}{4}t$ (one quarter of weld thickness) and not the inner diameter (ID) of the weld. This additional information provides the ID and $\frac{1}{4}t$ weld values for $\Delta RT_{NDT}^{\circ F}$ and Mean $RT_{NDT}^{\circ F}$ listed in Table 1. Attachment 1 contains the revised table.

If you have any questions, please contact Mr. Robert Steigerwald at (315) 349-6209.

There are no commitments made in this letter.

Very truly yours,


MICHAEL J. COLOMB
Site Executive Officer

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cc: Regional Administrator
U.S. Nuclear Regulatory Commission
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ATTACHMENT 1 to JAFP-99-0312

TABLE 1

Parameter Description	FitzPatrick Comparative Data at 32 EFPY (Bounding Circ Weld)	USNRC Limiting Plant Specific Analysis Data at 32 EFPY, SE Table 2.6-4	
		CE "BWRVIP"	CE "CEOG"
Fluence, n/cm ²	1.61E+18 (ID)	2.0E+18	2.0E+18
Initial RT _{NDT} °F	-50 (Ref. 6)	0	0
Chemistry Factor	* 209.1(Ref. 6)	151.7	172.2
Cu %	.337 (Ref. 6)	0.13	0.183
Ni %	.609 (Ref. 6)	0.71	0.704
ΔRT _{NDT} °F (ID/¼t)	* 108.5/93.9 (Ref. 6)	86.4	98.1
Mean RT _{NDT} °F (ID/¼t)	58.5/43.9	86.4	98.1

*Per Regulatory Guide 1.99, Revision 2