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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 26, 1999

Mr. Garry L. Randolph  
Vice President and Chief Nuclear Officer  
Union Electric Company  
Post Office Box 620  
Fulton, MO 65251

SUBJECT: RELIEF REQUEST TO ALLOW USE OF THE 1998 EDITION OF ASME CODE SECTION XI, SUBSECTION IWE - CALLAWAY PLANT, UNIT 1 (TAC NO. MA4598)

Dear Mr. Randolph:

By letters dated January 11, July 9, and September 10, 1999 (ULNRC-3938, -4063, and -4109, respectively), you requested relief from 10 CFR 50.55a to use the 1998 edition of Section XI, Subsection IWE, of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code (ASME Code) in lieu of the 1992 Edition and 1992 Addenda of the ASME Code, the code incorporated by reference in 10 CFR 50.55a. You stated that use of the 1998 Edition of Subsection IWE to perform inspections of the metallic containment and penetration liners and their integral attachments will reduce costs, radiation exposure, and personnel risk, but will not reduce component safety, quality, or integrity.

The staff was assisted in its review of the subject relief request by the Idaho National Engineering and Environmental Laboratory (INEEL). INEEL's technical evaluation report (TER) is attached to the enclosed safety evaluation and describes the bases for your request for relief, and discusses the implication of the alternative in terms of quality and safety related to the inspection of the Callaway containment.

Based on the review of your submittals, the staff concludes that the use of the 1998 Edition of the Code, supplemented by your commitment in your September 10, 1999, letter to document and perform an engineering evaluation, on a case-by-case basis, of any defect or deterioration that exceeds a depth of 10 percent the nominal wall thickness will provide an acceptable level of quality and safety for ensuring the integrity of the Callaway containment.

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PDR ADOCK

Mr. Garry L. Randolph

- 2 -

November 26, 1999

Therefore, relief is granted pursuant to 10 CFR 50.55a(a)(3)(i) in that the proposed alternative would provide an acceptable level of quality and safety and contingent upon the commitment discussed above. The NRC staff's evaluation and conclusions are contained in the enclosed safety evaluation.

Sincerely,

ORIGINAL SIGNED BY: S. Dembek

Stephen Dembek, Chief, Section 2  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-483

Enclosure: Safety Evaluation

cc w/encl: See next page

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Mr. Garry L. Randolph

- 2 - November 26, 1999

Therefore, relief is granted pursuant to 10 CFR 50.55a(a)(3)(i) in that the proposed alternative would provide an acceptable level of quality and safety and contingent upon the commitment discussed above. The NRC staff's evaluation and conclusions are contained in the enclosed safety evaluation.

Sincerely,



Stephen Dembek, Chief, Section 2  
Project Directorate IV & Decommissioning  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

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Enclosure: Safety Evaluation

cc w/enc: See next page

**Callaway Plant, Unit 1**

cc:

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