



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

November 26, 1999

Mr. John H. Mueller
Chief Nuclear Officer
Niagara Mohawk Power Corporation
Nine Mile Point Nuclear Station
Operations Building, Second Floor
Lycoming, NY 13093

SUBJECT: COMPLETION OF LICENSING ACTION FOR GENERIC LETTER 98-04, NINE MILE POINT NUCLEAR STATION, UNITS 1 AND 2 (TAC NOS. MA4070 AND MA4071)

Dear Mr. Mueller:

On July 14, 1998, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter (GL) 98-04, "Potential for Degradation of the Emergency Core Cooling System and the Containment Spray System After a Loss-of-Coolant Accident Because of Construction and Protective Coating Deficiencies and Foreign Material in Containment," to all holders of operating licenses or construction permits. The NRC issued GL 98-04 to determine the status of containment coating programs.

In GL 98-04, the NRC staff requested that licensees provide, for each of their facilities, the following information:

- (1) *A summary description of the plant-specific program or programs implemented to ensure that Service Level 1 protective coatings used inside the containment are procured, applied, and maintained in compliance with applicable regulatory requirements and the plant-specific licensing basis for the facility. Include a discussion of how the plant-specific program meets the applicable criteria of 10 CFR Part 50, Appendix B, as well as information regarding any applicable standards, plant-specific procedures or other guidance used for (a) controlling the procurement of coatings and paints used at the facility; (b) the qualification testing of protective coatings; and (c) surface preparation, application, surveillance, and maintenance activities for protective coatings. Maintenance activities refer to rework of degraded coatings, removing degraded coatings to sound coatings, correctly preparing the surfaces, applying new coating, and verifying the quality of coatings.*
- (2) *Information demonstrating compliance with item (i) or item (ii):*
 - (i) *For plants with licensing-basis requirements for tracking the amount of unqualified coatings inside the containment and for assessing the impact of potential coating debris on the operation of safety-related systems, structures, and components (SSCs) during a postulated design basis loss-of-coolant accident (DB LOCA), the following information shall be provided to demonstrate compliance:*
 - (a) *The date and findings of the last assessment of coatings and the planned date of the next assessment of coatings.*

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- (b) *The limit for the amount of unqualified protective coatings allowed in the containment and how this limit is determined. Discuss any conservatism in the method used to determine this limit.*
 - (c) *If a commercial-grade dedication program is being used at your facility for dedicating commercial-grade coatings for Service Level 1 applications inside the containment, discuss how the program adequately qualifies a coating for Service Level 1. Identify what standards or other guidance are currently being used to dedicate containment coatings at your facility.*
- (ii) *For plants without the above licensing-basis requirements, information shall be provided to demonstrate compliance with the requirements of 10 CFR 50.46b(5), "Long-term cooling" and the functional capability of the safety-related containment spray system (CSS) as set forth in your licensing basis. If a licensee can demonstrate this compliance without quantifying the amount of unqualified coatings, this is acceptable. The following information shall be provided:*

If a commercial-grade dedication program is not being used at your facility for qualifying and dedicating commercial-grade coatings for Service Level 1 applications, provide the regulatory and safety basis for not controlling these coatings in accordance with such a program. Additionally, explain why the facility's licensing basis does not require such a program.

Niagara Mohawk Power Corporation responded to GL 98-04 in a letter dated November 10, 1998, for Nine Mile Point Nuclear Station, Unit Nos. 1 and 2. This submittal provided the information requested by GL 98-04. Clarifications were provided during phone calls that took place on November 9 and 10, 1999, with Mr. A. Abbassi and others of your organization. The NRC staff has reviewed your response and has concluded that all requested information has been provided. Therefore, our review is considered complete and associated TAC Nos. MA4070 and MA4071 are closed. Thank you for your prompt and complete response.

If you have any questions regarding this matter, please contact me by phone on (301) 415-3049 or by electronic mail at dsh@nrc.gov.

Sincerely,



Darl S. Hood, Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-220 and 50-410

cc: See next page

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Unit Nos. 1 and 2**

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