

Dave Morey
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November 22, 1999

Docket Nos.: 50-348
50-364

NEL-99-0414

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555-0001

Joseph M. Farley Nuclear Plant
Response to Request for Additional Information
Steam Generator Replacement Related Technical Specifications Change Request

Ladies and Gentlemen:

In accordance with the provisions of 10 CFR 50.90, Southern Nuclear Operating Company (SNC) submitted a proposed amendment to the Farley Nuclear Plant (FNP) Unit 1 and Unit 2 Technical Specifications (TS) in a letter dated December 1, 1998. The amendment request addressed the required Technical Specifications changes associated with the replacement of the current Westinghouse Model 51 steam generators with the Westinghouse Model 54F. Revision 1 to the December 1, 1998 letter was submitted in a letter dated April 21, 1999. Supplemental information was provided on the Reactor Coolant System (RCS) Leak Before Break calculation results in a letter dated October 18, 1999. Revision 2 to the December 1, 1998 letter was submitted on November 11, 1999 and provided the steam generator replacement TS marked-up and clean typed pages on the final version of the Farley Improved Technical Specifications (ITS). Your November 19, 1999 letter requested additional information in order to complete your review of our submittal. In the Attachment, SNC provides the additional information requested.

There are no new commitments in this response. If you have any questions, please advise.

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U. S. Nuclear Regulatory Commission

Mr. D. N. Morey states that he is a Vice President of Southern Nuclear Operating Company and is authorized to execute this oath on behalf of Southern Nuclear Operating Company and that, to the best of his knowledge and belief, the facts set forth in this letter and attachments are true.

Respectfully submitted,

SOUTHERN NUCLEAR OPERATING COMPANY

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Dave Morey

Sworn to and subscribed before me this 22nd day of November 1999.

Martha Gayle Dow
Notary Public

My Commission Expires: November 1, 2001

CHM/maf: Letter-RAI 2.doc
Attachment: Joseph M. Farley Nuclear Plant Response to Request for Additional Information
Steam Generator Replacement Related Technical Specifications Change Request
(NRC Letter Dated November 19, 1999)

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U. S. Nuclear Regulatory Commission

cc: Southern Nuclear Operating Company
Mr. L. M. Stinson, General Manager – Farley

U. S. Nuclear Regulatory Commission, Washington, D. C.
Mr. L. M. Padovan, Licensing Project Manager – Farley

U. S. Nuclear Regulatory Commission, Region II
Mr. L. A. Reyes, Regional Administrator
Mr. T. P. Johnson, Senior Resident Inspector – Farley

Attachment

**Joseph M. Farley Nuclear Plant
Response to Request for Additional Information
Steam Generator Replacement Related Technical
Specifications Change Request
(NRC Letter Dated November 19, 1999)**

**Joseph M. Farley Nuclear Plant
Response to Request for Additional Information
Steam Generator Replacement Related Technical Specifications Change Request
(NRC Letter Dated November 19, 1999)**

NRC Question

Do you have an ongoing Farley/Westinghouse process to specify large break and small break loss-of-coolant-accident analysis peak cladding temperature-sensitive parameters and their analysis values that conservatively bound the as-operated plant values?

SNC Response

Farley/Westinghouse have processes in place which ensure that the peak cladding temperature (PCT)-sensitive parameters used as input to the best estimate large break loss of coolant accident (BELOCA) and small break loss of coolant accident (SBLOCA) analyses bound the as-operated plant values for Farley. The BELOCA methodology was established such that the PCT-sensitive parameters envelope the range of operating conditions. The SBLOCA analysis employs Appendix K methodology and requires the use of the most conservative value for parameters which are PCT-sensitive. As a result, the SBLOCA analysis is based on conservative, bounding input parameters relative to where the plant will operate.