



Northern States Power Company

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10 CFR 50.54(f)
Generic Letter 92-01

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Comprehensive Revised Response to Generic Letter 92-01

Preparation of the License Amendment Request dated March 6, 1998, which requested to update the reactor coolant system (RCS) pressure/temperature limit curves and low temperature overpressure protection (LTOP) system setpoints, provided an occasion to re-examine a large amount of the material previously submitted in response to Generic Letter 92-01. During this process it was discovered that the W2 weld in each reactor vessel is actually below the top of the active fuel (TAF). This discovery of the four materials, which were not previously accounted for as beltline materials, as well as minor discrepancies discovered during the review of capsule specimen test results and the development of temperature/pressure limit curves, resulted in Prairie Island (1) holding a conference call on January 29, 1998, with NRC staff from Region III and NRR to provide notification under the requirements of 10 CFR 50.9 that the reactor vessel W2 weld in each reactor was not included in our Generic Letter 92-01 submittals, (2) initiating a comprehensive review of the previous GL 92-01 submittals and subsequent reactor vessel test data to clarify all details related to the Prairie Island reactor vessels' structural integrity, and (3) informing the NRC staff in a letter dated September 16, 1998, that a comprehensive review of the previous GL 92-01 submittals and subsequent reactor vessel test data was in progress.

The comprehensive review of previous Generic Letter 92-01 submittals and subsequent reactor vessel material test data found numerous minor discrepancies in information and data previously reported. The discrepancies in reported material property values affected the non-limiting reactor vessel materials. None of these discrepancies are significant to safety or had any impact on the reported material properties that were the basis for the updated reactor vessel heatup and cooldown

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limit curves implemented in the Pressure and Temperature Limits Report (PTLR) that was submitted for NRC staff approval in the License Amendment Request Dated March 6, 1998.

This comprehensive review was conducted by an independent firm with extensive experience in this area, Structural Integrity Associates, Inc. and internal independent review of the final report was performed by Structural Integrity Associates in accordance with their QA program. In addition, the final report received a rigorous verification by Prairie Island staff in accordance with a commitment made by Prairie Island in response to a Notice of Violation (Inspection Report 97008) resulting from a Safety System Operational Performance Inspection. This commitment was that training would be conducted to ensure that all personnel responsible for the preparation, checking and verification of calculations are aware of the requirements and expectations with regards to the rigorous approach to use in the conduct of each of these activities.

All discovered discrepancies have been identified and corrected in the attached report. None of these discrepancies were found to be significant to safety.

If you have any questions related to this matter, please contact John Stanton at 651-388-1121 x4083.



Joel P. Sorensen
Site General Manager
Prairie Island Nuclear Generating Plant

Attachments:

Affidavit

"Update of the Response to Generic Letter 92-01 Reactor Vessel Structural Integrity for Prairie Island Units 1 and 2", Structural Integrity Associates, Inc., SIR-99-075 Rev. 2, approved 11/8/99.

c: Regional Administrator - Region III, NRC
Senior Resident Inspector, NRC
NRR Project Manager, NRC
J E Silberg
State of Minnesota, Attn: Kris Sanda

UNITED STATES NUCLEAR REGULATORY COMMISSION

NORTHERN STATES POWER COMPANY
PRAIRIE ISLAND NUCLEAR GENERATING PLANT DOCKET Nos. 50-282
50-306

Comprehensive Revised Response to Generic Letter 92-01

Northern States Power Company, a Minnesota corporation, with this letter is submitting information to revise information previously submitted in response to revisions, supplements and requests for additional information related to Generic Letter 92-01, Reactor Vessel Structural Integrity. This letter and its attachments contain no restricted or other defense information.

NORTHERN STATES POWER COMPANY

By Joel P. Sorensen
Joel P. Sorensen
Site General Manager
Prairie Island Nuclear Generating Plant

On this 10th day of November 1999 before me a notary public in and for said County, personally appeared, Joel P. Sorensen, Site General Manager, Prairie Island Nuclear Generating Plant, and being first duly sworn acknowledged that he is authorized to execute this document on behalf of Northern States Power Company, that he knows the contents thereof, and that to the best of his knowledge, information, and belief the statements made in it are true and that it is not interposed for delay.

Dale M. Vincent

