



Public Service Electric and Gas Company P.O. Box 236 Hancocks Bridge, New Jersey 08038-0236

Nuclear Business Unit

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Attn: Document Control Desk

**MONTHLY OPERATING REPORT
SALEM UNIT NO. 1
DOCKET NO. 50-272**

Gentlemen:

In compliance with Section 6.9.1.6, Reporting Requirements for the Salem Technical Specifications, the original Monthly Operating report for October 1999 is attached.

Sincerely,

M. B. Bezilla
Vice President - Operations

/rbk
Enclosures

C Mr. H. J. Miller
Regional Administrator USNRC, Region 1
475 Allendale Road
King of Prussia, PA 19046

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The power is in your hands.

PDR ADOCK 0500272 R

DOCKET NO.: 50-272
 UNIT: Salem 1
 DATE: 11/15/99
 COMPLETED BY: R. Knieriem
 TELEPHONE: (856) 339-1782

Reporting Period: October 1999

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)
 Maximum Dependable Capacity (MWe-Net)

No. of hours reactor was critical
 No. of hours generator was on line (service hours)
 Unit reserve shutdown hours
 Net Electrical Energy (MWH)

1115		
1106		
Month	Year-to-date	Cumulative
161	6260	117081
133	6200	112787
0	0	0
81718	6394389	113006552

UNIT SHUTDOWNS

NO	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/COMMENT
3	10/1/99 - 10/26/99	S	612	C	1	Refueling Outage

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

Summary:

Salem Unit 1 began the month of October 1999, in its 13th refueling outage. This outage continued until October 26, 1999 when the unit returned to service. Salem Unit 1 completed the month in power ascension at 95% power.

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE SALEM UNIT 1 GENERATING STATION

MONTH: October 1999

The following items completed during **October 1999** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant; nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes - Summary of Safety Evaluations

Modification 1EC-3388, Pkg. 1, Rev. 01, Feed Pump Recirculation System Upgrade

This modification replaced the existing single stage Feed Pump Recirculation Control Valve with a multi-stage drag valve, replaced the existing on-off recirculation flow control design with a modulating flow control, and installed strainers upstream of the Recirculation Control Valves.

Review of this modification under 10CFR50.59 was required because the upgrade of the Feed Pump Recirculation System constituted a change to the facility as described in the SAR. This upgrade did not impact the design, operation, or response time of the feedwater isolation functions, containment isolation of the safety-related portion of the feed water system or the Steam Generator Level Control System. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally,

this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Modification 1EC-3729, Pkg. 1, Rev. 0, Replacement of Pressurizer Safety Valves 1PR3, 1PR4, and 1PR5

This modification replaced the existing seismic category I, safety-related Salem Unit 1 Pressurizer Safety Valves with new seismic category I, safety-related valves meeting the requirements of ASME Section III, Class A, 1968 Edition through S68 Addendum. The replacement valves will maintain the same internals and performance characteristics as the previous design and differ only with regard to the valve body and bolting arrangement.

Review of this modification under 10CFR50.59 was required because the replacement of Pressurizer Safety Valves constituted a change to the facility as described in the SAR. With regard to performance, the replacement valves are essentially an equivalent replacement of existing valves, and differ only with regard to the valve body design and the bolting arrangement. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Modification 1EE-0405, Elimination of Reactor Coolant System (RCS) Valve Stem Leakoff Lines

This modification eliminated the valve stem leak-off lines from four valves used to isolate the Pressurizer Spray Valves, and two valves used to isolate the Pressurizer Power Operated Relief Valves. The modification also installed a single live-load packing set in lieu of the double packing set used with the leak-off line. Because of the improved performance of single live-load packing, the double packing, and leak-off lines to the PRT are no longer required to meet the design leakage requirements to the containment atmosphere.

Review of this modification under 10CFR50.59 was required because the elimination of the valve stem leak-off lines and the installation of single live-load packing constituted a change to the facility as described in the SAR. This evaluation concluded that this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or

malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Temporary Modifications - Summary of Safety Evaluations

Temporary Modification S99-023, 11 Switchgear Penetration Area (SPAV) Exhaust Fan Blank

This temporary modification installed a blank flange in the Switchgear Area exhaust plenum so that the 12 SPAV Exhaust Fan could continue to maintain ventilation to the Switchgear Area while the 11 SPAV Exhaust Fan was removed for preventive maintenance.

Review of this temporary modification under 10CFR50.59 was required because the installation of blank flange in the Switchgear Area exhaust plenum constituted a change to the facility as described in the SAR, and changed procedures as described in the SAR. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Procedures - Summary of Safety Evaluations

Procedure SC.MD-CM.CAV-0005(Q), Rev 0, Temporary Blank Installation for Switchgear Penetration Area Ventilation System (SPAV) Supply Fans

This procedure was developed to support the installation of a blank flange to isolate a SPAV Supply Fan from the remainder of the system to facilitate maintenance to that fan. During the installation of the blank flange, only one of three SPAV Supply Fans will be in service, vice two of three as required by the SAR.

Review of this procedure under 10CFR50.59 was required because the installation of a blank flange to isolate a SPAV Supply Fan constitutes a change to the facility as described in the SAR, and a change to procedures described in the SAR. During the installation of the blank flange, temperatures of areas cooled by the SPAV system will be monitored so that corrective actions can be taken, if necessary, to prevent exceeding temperature limits in those areas. Therefore, this change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new

failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

UFSAR Change Notices - Summary of Safety Evaluations

There were no changes in this category implemented during October 1999.

Deficiency Reports - Summary of Safety Evaluations

Deficiency Report 80003864, Inoperative Roughing Filter Damper on 14 Containment Fan Coil Unit (CFCU)

This evaluation addressed actions taken in response to the 14 CFCU Roughing Filter Damper that was found to be partially inoperative. The actions will result in a portion of the damper remaining closed and without any automatic operation capability until repairs are completed at a later date.

Review of this evaluation under 10CFR50.59 was required because the actions taken to address the partially inoperative 14 CFCU Roughing Filter Damper constituted a change to the facility as described in the SAR, and a change to procedures described in the SAR. The actions taken will not affect the ability of the CFCU to carry out its design function. Since the inoperative damper vanes are fixed in their accident configuration, and the remaining damper vanes are capable of automatic operation. Therefore the 14 CFCU can satisfy its design basis accident heat removal function. This change would not increase the probability or consequences of an accident previously analyzed. Additionally, this change did not increase the probability or consequences of a malfunction of equipment important to safety. This change would not create any new accidents or malfunctions since no new failure modes were introduced. In addition the Technical Specification Bases were not affected and no changes to the Technical Specifications were required.

Other - Summary of Safety Evaluations

There were no changes in this category implemented during October 1999.