

**Carolina Power & Light Company**

Robinson Nuclear Plant
3581 West Entrance Road
Hartsville SC 29550

Serial: RNP-RA/99-0222

NOV 15 1999

United States Nuclear Regulatory Commission

Attn: Document Control Desk

Washington, DC 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 50-261/LICENSE NO. DPR-23

MONTHLY OPERATING REPORT


Ladies and Gentlemen:

In accordance with Technical Specifications (TS) Section 5.6.4 for the H. B. Robinson Steam Electric Plant (HBRSEP), Unit No. 2, Carolina Power & Light (CP&L) Company submits the enclosed report of operating statistics and shutdown experience for the month of October 1999. In accordance with TS Section 5.6.4, this report is being submitted to the NRC by the 15th day of the month following the calendar month covered by the report.

Attachments I, II, and III provide the Monthly Operating Report in accordance with Technical Specifications Section 5.6.4. Attachment IV provides the Steam Generator Tube Inspection Report Information required by TS Section 5.6.8.b.

If you have any questions concerning this matter, please contact Mr. H. K. Chernoff.

Sincerely,



R. L. Warden
Manager - Regulatory Affairs

PMY/pmy

Attachments

c: NRC Resident Inspector, HBRSEP

L. A. Reyes, NRC, Region II

R. Subbaratnam, NRC, NRR

IE24

CP&L CO
 RUN DATE 11/02/99
 RUN TIME 07:55:56

PLANT PERFORMANCE DATA SYSTEM
 OPERATING DATA REPORT
 H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2

DOCKET NO. 050-0261
 COMPLETED BY TOM FREEMAN
 TELEPHONE (843)857-1403

OPERATING STATUS

1. UNIT NAME: H. B. ROBINSON STEAM ELECTRIC PLANT (HBRSEP),
UNIT NO. 2
2. REPORTING PERIOD: OCTOBER 1999
3. LICENSED THERMAL POWER (MWT): 2300
4. NAMEPLATE RATING (GROSS MWE): 739.0
5. DESIGN ELECTRICAL RATING (NET MWE): 700.0
6. MAX DEPENDABLE CAPACITY (GROSS MWE): 700.0
7. MAX DEPENDABLE CAPACITY (NET MWE): 683.0
8. IF CHANGES OCCUR IN CAPACITY RATING (ITEMS 3 THROUGH 7) SINCE LAST REPORT,
GIVE REASONS:
9. POWER LEVEL TO WHICH RESTRICTED IF ANY (NET MWE): None
10. REASONS FOR RESTRICTION, IF ANY:

NOTES:

	THIS MONTH	YR TO DATE	CUMUL ATIVE
11. HOURS IN REPORTING PERIOD	745.00	7296.00	251328.00
12. NUMBER OF HOURS REACTOR CRITICAL	198.77	6576.05	185648.26
13. REACTOR RESERVE SHUTDOWN HRS	.00	.00	3314.65
14. HOURS GENERATOR ON LINE	173.67	6546.35	182383.48
15. UNIT RESERVE SHUTDOWN HOURS	.00	.00	23.20
16. GROSS THERMAL ENERGY GEN. (MWH)	359523.12	14793793.20	381836406.24
17. GROSS ELEC. ENERGY GEN. (MWH)	114983.00	4850397.00	124147847.00
18. NET ELEC. ENERGY GENERATED (MWH)	104813.00	4620045.00	117514793.00
19. UNIT SERVICE FACTOR	23.21	89.73	72.57
20. UNIT AVAILABILITY FACTOR	23.31	89.73	72.58
21. UNIT CAP. FACTOR (USING MDC NET)	20.60	92.71	69.73
22. UNIT CAP. FACTOR (USING DER NET)	20.10	90.46	66.80
23. UNIT FORCED OUTAGE RATE	.00	.00	12.55

24. SHUTDOWNS SCHED. OVER NEXT 6 MONTHS (TYPE DATE AND DURATION OF EACH): None
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF START UP:
26. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): FORECAST ACHIEVED

INITIAL CRITICALITY
 INITIAL ELECTRICITY
 COMMERCIAL OPERATION

CP&L CO
RUN DATE 11/02/99
RUN TIME 07:55:55PLANT PERFORMANCE DATA SYSTEM
APPENDIX B - AVERAGE DAILY POWER LEVEL
H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2PAGE 1 of 1
RPD39-000DOCKET NO. 050-0261
COMPLETED BY: THOMAS FREEMAN
TELEPHONE (843)857-1403

OCTOBER 1999

DAY	AVG. DAILY POWER LEVEL (MWE-NET)	DAY	AVG. DAILY POWER LEVEL (MWE-NET)
1	-2	17	-6
2	-2	18	-7
3	-2	19	-11
4	-2	20	-13
5	-2	21	-19
6	-2	22	-23
7	-2	23	-24
8	-3	24	1
9	-4	25	322
10	-5	26	556
11	-5	27	723*
12	-5	28	726*
13	-5	29	725*
14	-5	30	724*
15	-5	31	723*
16	-5		

* The maximum dependable capacity is 683 MWE. The Plant operated above the Max Dependable Capacity (MDC) due to the condenser water inlet temperature < 95° F.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-0261
UNIT NAME HBRSEP, UNIT NO. 2
DATE 11/02/1999
COMPLETED BY THOMAS FREEMAN
TELEPHONE (843) 857-1403

REPORT MONTH: OCTOBER 1999

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
99-002	9/25/99	S	571.3	C	1				Scheduled Shutdown - RO - 19

1

2

3

4

F: Forced
S: Scheduled

Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Other (Explain)

Exhibit G - Instructions
For Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File
(NUREG-0161)

⁵ Exhibit 1 - Same Source

SUMMARY: Continuation of RO-19. Outage ended on 10/24/99. There were no challenges to the pressurizer power operated relief valves or the pressurizer safety valves during the month. Steam generator tube inspections performed during the month are summarized in Attachment IV to this letter.

STEAM GENERATOR TUBE INSPECTION REPORT

SUMMARY

This document summarizes the Steam Generator (S/G) inspection program and the results of the examination that was performed at the H. B. Robinson Steam Electric Plant, Unit No. 2 during Refueling Outage (RO) 19 from September 25, 1999 to October 24, 1999.

The eddy current examinations were performed utilizing Zetec bobbin coil probes and Rotating Pancake and/or Plus Point Coil (RPC) probes. The site specific eddy current detection and sizing techniques used to perform the examinations were based on EPRI PWR Steam Generator Examination Guidelines, TR-107569-V1R5, Revision 5 qualified techniques. RPC examinations were performed in S/G "A" on approximately 20% of the inlet (Hot) side historical manufacturing buff marks, dents, and benign bobbin indications. RPC examinations were also performed on 100% of the U-bend region of Row 1 and 2 tubes utilizing a single Plus Point Coil probe. S/G "A" had a total of 50% of the tubes tested with the Bobbin Coil from the Cold Leg and 60% tested from the Hot Leg in the top-of-tubesheet area.

There were no repairable tube indications reported for the 1999 RO19 examination. Five tubes had signal responses that indicated a volumetric/geometric condition at the tubesheet interface. Three tubes indicated a loose part that was dispositioned by comparing the data to the results of loose part indications detected during RO18. The loose part indications represented signal responses that were less than those reported in RO18. There were no other tubes affected by the loose part or any degradation detected on any of those tubes. A site-qualified technique was implemented for sizing of indications that were a result of loose part or other mechanical part wear. The technique used rotating coil technology and a special sizing standard that represented several different types of wear patterns. The site-qualified techniques was based on EPRI qualified techniques for sizing wear.

EXAMINATION RESULTS

The following summarizes the inspections of the recorded indications.

There were no wear indications found in the anti-vibration bars by the bobbin coil examination.

There were a total of 54 indications recorded in 34 tubes with the Plus Point coil. The indications were located in the periphery of the tube bundle in both Hot and Cold Leg plenums. A site-qualified technique utilizing the Plus Point Coil determined the size of these indications. These indications were small in volume. No tube indications exceeded the repair criteria of 40% depth.

There were 5 tubes that were considered to have imperfections due to the manufacturing irregularities at the tubesheet interface. No associated tube degradation was detected in these tubes.

Three tubes examined contained indications that were detected with low frequency that indicated the presence of a loose part. These indications were smaller in nature than recorded in RO18. No degradation was associated with those tubes.

The majority of the remaining indications recorded during the RO19 examination were from signal responses that represented manufacturing buff marks. The primary and secondary analyses record the signals as non-quantifiable indications, if it met the recording criteria. The non-quantifiable signal indications were determined to be present during the baseline examination or next available inspection.

EXAMINATION RESULTS Continued

The following table lists the other types of indications recorded, their quantity, and a description/comment on each.

# of Indications	3 Letter Code	Comment
4	CUD	Copper Deposits recorded from past outages
1	DDI	Distorted Dent Indication - No degradation
3	DDS	Distorted Dent Signal
116	DNT	Dents ≥ 2 Volts on P1
5	DSS	Distorted Support Signal
4	INF	Indication not found - previous indication was not present. Note: Investigation of past history determined incorrect locations were reported.
40	NQI	Non-quantifiable Indication - resolved by RPC examinations and recorded as % measurements.
1	NXP	One tube in cold leg has no tube expansion in tube sheet.
1	OXP	One tube has a slight over expansion at the tube sheet interface.
3	PLP	Possible Loose Part
7	PRC	Previously recorded rotating coil indication
11	RBD	Retest, bad data
2	RIC	Retest, incomplete test
5	VOL	Volumetric type indication at tube sheet interface
1120	NQS	Non Quantifiable Signal - these were NQI's that were looked up in history during the resolution process and determined to be present on the baseline data.