



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

STP NUCLEAR OPERATING COMPANY

DOCKET NO. 50-499

SOUTH TEXAS PROJECT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 105
License No. NPF-80

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by STP Nuclear Operating Company* acting on behalf of itself and for Houston Lighting & Power Company (HL&P), the City Public Service Board of San Antonio (CPS), Central Power and Light Company (CPL), and the City of Austin, Texas (COA) (the licensees), dated May 7, 1998, as supplemented by letters dated May 20, June 16, September 30, October 20, and October 21, 1999, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

*STP Nuclear Operating Company is authorized to act for Houston Lighting & Power Company (HL&P), the City Public Service Board of San Antonio, Central Power and Light Company, and the City of Austin, Texas, and has exclusive responsibility and control over the physical construction, operation, and maintenance of the facility.

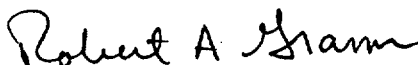
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and Paragraph 2.C.(2) of Facility Operating License No. NPF-80 is hereby amended to read as follows:

2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 105 , and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 30 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Robert A. Gramm, Chief, Section 1
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Attachment: Changes to the Technical
Specifications

Date of Issuance: November 8, 1999

ATTACHMENT TO LICENSE AMENDMENT NOS. 117 AND 105

FACILITY OPERATING LICENSE NOS. NPF-76 AND NPF-80

DOCKET NOS. 50-498 AND 50-499

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

3/4 2-11
B 3/4 2-6

INSERT

3/4 2-11
B 3/4 2-6

POWER DISTRIBUTION LIMITS

3/4.2.5 DNB PARAMETERS

LIMITING CONDITION FOR OPERATION

- 3.2.5 The following DNB-related parameters shall be maintained within the limits following:
- Reactor Coolant System T_{avg} , \leq the limit as specified in the Core Operating Limits Report
 - Pressurizer Pressure, $>$ the limit as specified in the Core Operating Limits Report
 - Thermal Design Reactor Coolant System Flow for:
 - Model E Steam Generators, $\geq 370,000$ gpm
 - Model $\Delta 94$ Steam Generators, $\geq 392,000$ gpm

APPLICABILITY: MODE 1.

ACTION:

With any of the above parameters exceeding its limit, restore the parameter to within its limit within 2 hours or reduce THERMAL POWER to less than 5% of RATED THERMAL POWER within the next 4 hours.

SURVEILLANCE REQUIREMENTS

- 4.2.5.1 Each of the parameters shown above shall be verified to be within its limits at least once per 12 hours. The provisions of Specification 4.0.4 are not applicable for verification that RCS flow is within its limit.
- 4.2.5.2 The RCS flow rate indicators shall be subjected to a channel calibration at least once per 18 months.

NOTE

SR 4.2.5.3 is required at beginning-of-cycle with reactor power $\geq 90\%$ RTP.

- 4.2.5.3 The RCS total flow rate shall be determined by precision heat balance or elbow tap ΔP measurements at least once per 18 months. The provisions of Specification 4.0.4 are not applicable.

POWER DISTRIBUTION LIMITS

BASES

3/4.2.5 DNB PARAMETERS (continued)

The value for thermal design RCS flow rate presented in Technical Specification 3.2.5 is the thermal design reactor coolant system flow rate used in the analysis approved by the Nuclear Regulatory Commission in Amendments 97 and 84 on September 29, 1998. This flow rate is an analytical limit consistent with 10% tube plugging of the steam generator tubes and Departure from Nucleate Boiling requirements. Thermal design RCS flow rate with Model E Steam Generators is 370,000 gpm, and with Model Δ 94 Steam Generators it is 392,000 gpm. These values include 2.8% measurement uncertainty.

The RCS flow measurement uncertainty of 2.8% bounds the precision heat balance and the elbow tap Δp measurement methods. The elbow tap Δp measurement uncertainty presumes that elbow tap Δp measurements are obtained from either QDPS or the plant process computer. Based on instrument uncertainty assumptions, RCS flow measurements using either the precision heat balance or the elbow tap Δp measurement methods are to be performed at greater than or equal to 90% RTP at the beginning of a new fuel cycle. The elbow tap Δp RCS flow measurement methodology is described in ST-HL-AE-5707, "Proposed Amendment to Technical Specification Table 2.2-1 and 3/4.2.5 for Reactor Coolant System Flow Monitoring – Revised," dated August 6, 1997, and in ST-HL-AE-5752, "Amended Response to Request for Additional Information on the Proposed Elbow Tap Technical Specification Change (Table 2.2-1 and Section 3/4.2.5)," dated September 18, 1997.

The 12-hour periodic surveillance of these parameters through instrument readout is sufficient to ensure that the parameters are restored within their limits following load changes and other expected transient operation.