

November 9, 1999

Mr. R. P. Powers
Senior Vice President
Nuclear Generation Group
American Electric Power Company
500 Circle Drive
Buchanan, MI 49107-1395

SUBJECT: DONALD C. COOK NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF THE SECOND DRAFT RISK ASSESSMENT

Dear Mr. Powers:

By letter dated August 12, 1999, the NRC transmitted the draft preliminary results of an analysis of the risk significance of issues that were identified at D. C. Cook Nuclear Plant (D. C. Cook), Units 1 and 2, between August 1997 and January 1999. The analysis is being performed to support our ongoing accident sequence precursor analysis program, which evaluates operational events at nuclear power plants to determine the risk-significance of events or conditions. This analysis is consistent with the new reactor oversight process for analyzing the safety significance of inspection findings.

In our August 12, 1999, letter, we asked you to provide comments and also stated that the analysis was ongoing, and as parts were completed, they would be transmitted to you for additional comments. We have completed the second phase of the risk analysis and have enclosed the draft preliminary results. In this portion of the analysis, the staff has identified the following five issues to be potentially risk significant:

- Vortexing in the containment sump due to diversion of sump water from the active areas of the sump to the inactive areas of the sump (Issue #26);
- Failure of high pressure injection pumps or throttle valves due to debris ingestion during the sump recirculation function (Issue #27);
- Vortexing in the refueling water storage tank (RWST) and resulting failure of the residual heat removal (RHR) pumps (Issue #36);
- Debris accumulation on recirculation sump screen causing head losses which may result in RHR pump cavitation (Issue #28); and
- A postulated crack in a Unit 2 main steamline degrading the ability of component cooling water pumps of both units to perform their function (Issue #53).
- A postulated crack in a Unit 2 main steamline degrading the ability of component cooling water pumps of both units to perform their function (Issue #53).

We are again providing to you the analysis methods and results of our continuing review effort. We will be pleased to receive and address any comments that you may have. A final report that contains the analyses of all issues is scheduled to be transmitted to you in December of 1999 for your review and comment.

The Manual Chapter (MC) 0350 Restart Panel will be using the risk analysis to aid in determining the risk significance of the issues contained on the NRC Restart Action Matrix (RAM). This will give the MC 0350 panel another factor in determining which issues contained on the RAM will require further detailed inspection prior to restart of a unit at D. C. Cook.

Please contact me at 630-829-9700 if you have any questions regarding this letter.

Sincerely,

Original Signed By John A. Grobe

John A. Grobe, Director
Division of Reactor Safety
Region III

Docket Nos. 50-315; -316

Enclosure: Draft Preliminary Analysis of
Potential Risk Significant Issues

cc w/encl: A. C. Bakken III, Site Vice President
T. Noonan, Plant Manager
M. Rencheck, Vice President, Nuclear Engineering
R. Whale, Michigan Public Service Commission
Michigan Department of Environmental Quality
Emergency Management Division
MI Department of State Police
D. Lochbaum, Union of Concerned Scientists

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