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Nuclear Business Unit

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U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Gentlemen:

**MONTHLY OPERATING REPORT
HOPE CREEK GENERATING STATION UNIT 1
DOCKET NO. 50-354**

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for **October 1999** are being forwarded. Also being forwarded, pursuant to the requirements of 10CFR50.59(b), is the summary of changes, tests, and experiments that were implemented during **October 1999**.

Sincerely,

Mark B. Bezilla
Vice President - Operations

RAR
Attachments

C Distribution

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The power is in your hands.

PDR ADDIC

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DOCKET NO.: 50-354
 UNIT: Hope Creek
 DATE: 11/8/99
 COMPLETED BY: F. Todd
 TELEPHONE: (609) 339-1316

Reporting Period October 1999

OPERATING DATA REPORT

Design Electrical Rating (MWe-Net)
Maximum Dependable Capacity (MWe-Net)

No. of hours reactor was critical
No. of hours generator was on line (service hours)
Unit reserve shutdown hours
Net Electrical Energy (MWH)

	1067	
	1031	
Month	Year-to-date	Cumulative
745	6134	95263
745	6074	93683
0	0	0
767291	6182948	94780469

UNIT SHUTDOWNS

NO.	DATE	TYPE F=FORCED S=SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR (2)	CORRECTIVE ACTION/ COMMENT

(1) Reason

- A - Equipment Failure (Explain)
- B - Maintenance or Test
- C - Refueling
- D - Regulatory Restriction
- E - Operator Training/License Examination
- F - Administrative
- G - Operational Error (Explain)
- H - Other

(2) Method

- 1 - Manual
- 2 - Manual Trip/Scram
- 3 - Automatic Trip/Scram
- 4 - Continuation
- 5 - Other (Explain)

DOCKET NO.: 50-354
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DATE: 11/8/99
COMPLETED BY: R. Ritzman
TELEPHONE: (609) 339-1445

Summary Of Monthly Operating Experience

- Hope Creek entered the month of October at approximately 100% reactor power.
- On October 6 there was an automatic runback due a 3B, 4B, 5B Feedwater Heater trip due to a fuse problem. Power was restored to approximately 100% reactor power on October 8.
- On October 12, power was reduced for a control rod pattern adjustment. Power was restored to approximately 100% reactor power later that day.
- At the end of the month, Hope Creek completed 60 days of continuous power operation.

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SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION

MONTH October 1999

The following items completed during **October 1999** have been evaluated to determine:

1. If the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or
2. If a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or
3. If the margin of safety as defined in the basis for any technical specification is reduced.

The 10CFR50.59 Safety Evaluations showed that these items did not create a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These items did not change the plant effluent releases and did not alter the existing environmental impact. The 10CFR50.59 Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Design Changes Summary of Safety Evaluations

There were no reportable changes in this category implemented during October 1999.

Temporary Modifications Summary of Safety Evaluations

There were no reportable changes in this category implemented during October 1999.

Procedures Summary of Safety Evaluations

There were no reportable changes in this category implemented during October 1999.

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS
FOR THE HOPE CREEK GENERATING STATION – Cont'd

UFSAR Change Notices Summary of Safety Evaluations

There were no reportable changes in this category implemented during October 1999.

Other Summary of Safety Evaluations

Safety Evaluation H99-053, Service Water Pump Troubleshooting at Different Flow Rates. This 10CFR50.59 Safety Evaluation was written to support troubleshooting of the Service Water pumps. The Service Water pumps have failed Inservice Tests (IST) due to low differential pressure across the pump. This troubleshooting consists of adjusting a heat exchanger discharge valve to achieve various flow rates, and gathering data at these pre-set flow rates. The information gathered from the troubleshooting will assist in determining the solution to the low differential pressure issue. In order to gather the desired data, the Service Water system will be operated in configurations that are not discussed in detail in the UFSAR. Therefore, the troubleshooting was considered to be a "test or experiment not described in the UFSAR."

Although the Service Water system will be operated in a configuration that is not discussed in the UFSAR, the Service Water system will be able to perform its design function at all times. In addition, this proposal does not increase the probability or consequences of an accident previously evaluated in the SAR, does not increase the probability or consequences of a malfunction of equipment important to safety previously evaluated in the SAR, does not create the possibility of an accident or malfunction of a different type from any previously evaluated in the SAR, and does not reduce the margin of safety as defined in the basis for any Technical Specification. Therefore, this proposal does not involve an Unreviewed Safety Question.

Safety Evaluation H99-055, Emergency Sump Pump Discharge Valves. This 10CFR50.59 was written to support a revision to the normal position of two Emergency Sump Pump Discharge Valves from open to normally locked closed. Maintaining these valves in a normally locked closed position will provide additional positive control that the sumps will not be pumped down prior to being sampled.

The implementation of additional administrative controls to provide additional positive control that the sumps will not be pumped down prior to being sampled does not increase the probability or consequences of an accident previously evaluated in the SAR, does not increase the probability or consequences of a malfunction of equipment important to safety previously evaluated in the SAR, does not create the possibility of an accident or malfunction of a different type from any previously evaluated in the SAR, and does not reduce the margin of safety as defined in the basis for any Technical Specification. Therefore, this proposal does not involve an Unreviewed Safety Question.