

November 2, 1999

Mr. Oliver D. Kingsley
President, Nuclear Generation Group
Commonwealth Edison Company
ATTN: Regulatory Services
Executive Towers West III
1400 Opus Place, Suite 500
Downers Grove, IL 60515

SUBJECT: NRC INSPECTION REPORT 50-373/99020(DRS); 50-374/99020(DRS)

Dear Mr. Kingsley:

On October 6, 1999, the NRC completed an inspection at your LaSalle Nuclear Generating Station, Units 1 and 2. The enclosed report presents the results of that inspection. The inspection examined activities related to the identification and resolution of problems within the corrective action program.

Areas examined are identified in the report. Within those areas, the inspection consisted of a selective examination of procedures and representative records, and interviews with personnel. The objective of the inspection effort was to determine whether activities authorized by the license were conducted safely and in accordance with NRC requirements.

Based on the results of this inspection, the NRC has determined that two violations of NRC requirements occurred. These violations are being treated as Non-Cited Violations (NCVs), consistent with Appendix C of the Enforcement Policy. These NCVs are described in the subject inspection report. If you contest the violation or severity level of the NCVs, you should provide a response within 30 days of the date of this inspection report, with the basis for your denial, to the Nuclear Regulatory Commission, ATTN: Document Control Desk, Washington, DC 20555-0001, with a copies to the Regional Administrator, Region III, and the Director, Office of Enforcement, United States Nuclear Regulatory Commission, Washington, DC 20555-0001.

Although the violations are not being cited, they are of concern because they are the result of inadequate corrective actions. The corrective actions for the fillet weld design deficiency were narrowly focused and other similar deficiencies were identified by the inspector. Also during review of the actions to resolve a pipe support modeling concern, the inspector identified that the technical bases for a potentially non-conservative design assumption involving anchor bolt stiffness were not adequately demonstrated. We are concerned about the possible generic nature of that design assumption and would like to meet with you to discuss this in detail. The meeting would be conducted by the Office of Nuclear Reactor Regulation and coordinated by Ms. Donna Skay (301-415-1322). To support this meeting, we request that you submit information describing the basis for the anchor bolt stiffness values used in your design, your resolution of the modeling concern, and the impact of your assumptions on the pipe support design margins.

PDR ADOCU

O. Kingsley

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In accordance with 10 CFR 2.790 of the NRC's "Rules of Practice," a copy of this letter, its enclosure, and your response, if you choose to provide one, will be placed in the NRC Public Document Room.

We will gladly discuss any questions you have concerning this inspection.

Sincerely,

Original Sign By J. Grobe

John A. Grobe, Director
Division of Reactor Safety

Docket Nos. 50-373; 50-374
License Nos. NPF-11; NPF-18

Enclosure: Inspection Report 50-373/99020(DRS); 50-374/99020(DRS)

cc w/encl: D. Helwig, Senior Vice President, Nuclear Services
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