



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

October 28, 1999

LICENSEE: Duke Energy Corporation (Duke)
FACILITY: Oconee Nuclear Station, Units 1, 2, and 3
SUBJECT: SUMMARY OF MEETING BETWEEN THE U.S. NUCLEAR REGULATORY COMMISSION (NRC) STAFF AND DUKE REPRESENTATIVES REGARDING ISSUES ASSOCIATED WITH REACTOR VESSEL INTERNALS FOR THE OCONEE LICENSE RENEWAL APPLICATION (LRA)

On September 29, 1999, representatives of Duke met with the NRC staff at Rockville, Maryland, to discuss issues associated with reactor vessel internals for the LRA for Oconee Units 1, 2, and 3. A list of meeting attendees is provided in the Enclosure. The purpose of the meeting was for the staff to provide feedback to Duke for the following issues:

- plant specific action items related to BAW-2248, "Demonstration of the Management of Aging Effects for the Reactor Vessel Internals"
- draft responses to Oconee safety evaluation report (SER) open items related to reactor vessel internals
- general discussion on how to capture commitments related to the reactor vessel internals aging management program

The following background material was provided at the meeting by the staff to aid in the discussion: 1) a meeting summary dated August 2, 1999, for a July 19, 1999, meeting between the Babcock and Wilcox Owners Group (B&WOG) and the NRC, and 2) a B&WOG response dated August 16, 1999, to the staff regarding void swelling.

The meeting summary dated August 2, 1999, contains Duke's proposed plant-specific action items related to BAW-2248. The summary also contains Duke's proposed responses for Oconee regarding the SER open items related to reactor vessel internals.

The staff stated that, in general, it needed the following information to resolve the open items in BAW-2248 and the SER:

- The critical locations for the identified aging mechanisms need to be described as well as the basis for the critical location (e.g., for void swelling the critical location may be based on high gamma locations and locations where design basis accidents assume a minimum flow)
- The tests that have been or will be used to resolve the issue need to be described
- The engineering analyses that have been or will be used to resolve the issue need to be described
- The inspections that have been or will be used to resolve the issue need to be described

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The staff stated that commitments that relate to future actions may be captured by license conditions. To the extent that the aging mechanism is the subject of on-going research (e.g., void swelling) the staff would expect that licensees commit to follow the on-going research programs and to implement corrective actions as appropriate. The staff could capture this type of commitment in a more broadly worded license condition with respect to schedules than the type of license condition that might be used to capture a specific commitment.

The staff stated that it could proceed with issuing the final safety evaluation (SE) for BAW-2248 with renewal applicant action items identified for any issue that was not fully resolved. Alternatively, the staff stated that it could attempt to resolve some of the issues in the draft SE for BAW-2248, which would limit the number of renewal applicant action items that would need to be addressed. However, the latter option would require more time and interaction between the staff and the B&WOG. Duke stated that it would discuss the options with the B&WOG and let the staff know which option the B&WOG preferred. Subsequent to the meeting, Duke stated that it and the members of the B&WOG preferred the staff to issue the final SE with renewal applicant action items identified for any issue that was not fully resolved.

At the end of the meeting the staff informed Duke that it would be asking the B&WOG a question based on discussions that it was currently having with Baltimore Gas and Electric Company regarding the Calvert Cliffs LRA. Specifically, the issue involves the reactor vessel flange leakoff line. In the final SE for BAW-2251, "Demonstration of the Management of Aging Effects for the Reactor Vessel" the staff agreed with the B&WOG that the reactor vessel flange leakoff line was outside the scope of license renewal based on the reactor vessel flange O-rings being credited with performing a pressure boundary function. The staff stated that it has reevaluated this conclusion because O-rings do not typically serve a pressure boundary function. The staff stated that it would pursue this issue further with the B&WOG.

A draft of this meeting summary was provided to Duke to allow them the opportunity to comment on the summary prior to issuance.

Original Signed By

Joseph M. Sebrosky, Project Manager
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Docket Nos. 50-269, 50-270, and 50-287

Enclosures: As stated (1)
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NRC MEETING WITH DUKE ENERGY CORPORATION TO DISCUSS
LICENSE RENEWAL ISSUES ASSOCIATED WITH REACTOR VESSEL INTERNALS
ATTENDANCE LIST
SEPTEMBER 29, 1999

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Enclosure

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