



October 29, 1999
RC-99-0209

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Gentlemen:

Subject: VIRGIL C. SUMMER NUCLEAR STATION
DOCKET NO. 50-395
OPERATING LICENSE NO. NPF-12
LICENSEE EVENT REPORT (LER 1999-012-00)
Inadvertent Start of 'B' RHR Pump

Attached is Licensee Event Report No. 1999-012-00, for the Virgil C. Summer Nuclear Station (VCSNS). This report describes the inadvertent start of the 'B' Residual Heat Removal Pump during removal of another breaker in the same switchgear. This issue is being reported per 10 CFR 50.73(a)(2)(iv).

Should you have any questions, please call Mr. Jeffrey W. Pease at (803) 345-4124.

Very truly yours,

Gary J. Taylor

GJT/JWP
Attachment

- | | |
|------------------------------|----------------------|
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LICENSEE EVENT REPORT (LER)	

FACILITY NAME (1) Virgil C. Summer Nuclear Station	DOCKET NUMBER (2) 05000395	PAGE (3) 1 of 3
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TITLE (4)
 Inadvertent Start of 'B' Residual Heat Removal (RHR) Pump

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
09	30	1999	1999	-- 012	-- 00	10	29	1999		05000
									FACILITY NAME	DOCKET NUMBER

OPERATING MODE (9) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10) 100	<input type="checkbox"/>	20.2201(b)	<input type="checkbox"/>	20.2203(a)(2)(v)	<input type="checkbox"/>	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)
	<input type="checkbox"/>	20.2203(a)(1)	<input type="checkbox"/>	20.2203(a)(3)(i)	<input type="checkbox"/>	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(x)
	<input type="checkbox"/>	20.2203(a)(2)(i)	<input type="checkbox"/>	20.2203(a)(3)(ii)	<input type="checkbox"/>	50.73(a)(2)(iii)	<input type="checkbox"/>	73.71
	<input checked="" type="checkbox"/>	20.2203(a)(2)(ii)	<input type="checkbox"/>	20.2203(a)(4)	<input type="checkbox"/>	X 50.73(a)(2)(iv)	<input type="checkbox"/>	OTHER 21.21(a)(1)
	<input type="checkbox"/>	20.2203(a)(2)(iii)	<input type="checkbox"/>	50.36(c)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	<input type="checkbox"/>	Specify in Abstract below or in NRC FORM 366A
<input type="checkbox"/>	20.2203(a)(2)(iv)	<input type="checkbox"/>	50.36(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	<input type="checkbox"/>		

LICENSEE CONTACT FOR THIS LER (12)

NAME A. R. Rice Manager, Nuclear Licensing & Operating Experience	TELEPHONE NUMBER (include Area Code) (803) 345-4232
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO EPIX

SUPPLEMENTAL REPORT EXPECTED (14)				EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE).	NO X							

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

At 1410 on September 30, 1999, during performance of Electrical Maintenance Procedure EMP-405.002 on a spare breaker in cubical XSW1DB1 05C, the four wheel roll around cart utilized for breaker removal came into contact with the manual close lever for the bottom breaker in the Motor Control Center (MCC), XSW1DB1 05D. This breaker (XSW 1DB1 05D) supplies power to the 'B' train Residual Heat Removal (RHR) pump. The breaker closed causing the 'B' train RHR pump to start.

Due to the high noise level in the switchgear room, the electrician performing the maintenance task was wearing earplugs and did not hear the breaker close. The electrician exited the area with the breaker from XSW1DB1 05C. The pump ran for approximately six minutes prior to being stopped by the control room operators. At 1416 the pump was secured and placed in pull-to-lock after verifying that no apparent demand signal was present. The pump remained in Pull-to-Lock while the investigation continued.

At 1445 on September 30, 1999, it was determined that this was an inadvertent actuation of the pump caused by human error. At this time, the pump was restored to standby which is the normal operating condition.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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		1999	--- 012 ---	00	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

PLANT IDENTIFICATION

Westinghouse - Pressurized Water Reactor

EQUIPMENT IDENTIFICATION

EIIS Code: EA

IDENTIFICATION OF EVENT

This report is based on the initiation of CER 99-1295

EVENT DATE

September 30, 1999.

REPORT DATE

October 29, 1999

CONDITIONS PRIOR TO EVENT

Mode 1 – Normal Operations (100%)

**LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION**

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TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

DESCRIPTION OF EVENT

During planned maintenance activities on cubicle XSW1DB1 05C, the breaker in cubicle XSW1DB1 05D supplying the 'B' Residual Heat Removal (RHR) Pump was accidentally closed. Breaker XSW1DB1 05D is below XSW 1DB1 05C in the same Motor Control Center (MCC). The pump ran for approximately six minutes until control room operators, after verifying that no apparent demand signal was present, secured the pump and placed it in Pull-to-Lock. The pump remained in Pull-to-Lock for 29 minutes while the investigation continued.

CAUSE OF EVENT

The VCSNS electrician removing the breaker from cubicle XSW 1DB1 05C, inadvertently allowed the roll around cart utilized for removal of these breakers to contact the manual close lever for the breaker in cubicle XSW1DB1 05D.

ANALYSIS OF EVENT

This event did not impact the ability of RHR to perform its Emergency Core Cooling System functions.

INTERIM CORRECTIVE ACTIONS

A stand down was held for the Electrical Maintenance group on the morning of October 4, 1999. During this time all electrical maintenance personnel were briefed on the event of September 30, 1999 and its impact on plant operations.

ADDITIONAL CORRECTIVE ACTIONS

The review of this event will be placed on tape for training of shift personnel. Alterations to transport cart are under investigation to protect against reoccurrence.

PRIOR OCCURRENCES

None