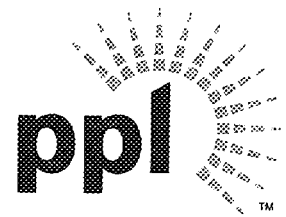


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NOV 05 1999

U.S. Nuclear Regulatory Commission
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SUSQUEHANNA STEAM ELECTRIC STATION
LICENSEE EVENT REPORT 50-387/99-001-02
PLA - 5130 FILE R41-2

Docket No. 50-387
License No. NPF-14

Attached is Licensee Event Report 50-387/99-001-02. This supplement is being provided to update the safety significance of the failure of the Low Pressure Coolant Injection (LPCI) throttle valve (HV-151F017B). As stated in the original submittal of this Licensee Event Report, the exact date of the failure of the LPCI throttle valve cannot be determined. For the reportability of the event, the failure date was assumed to be February 11, 1999, since this was the date when the first indications that there was a potential degradation in the 'B' Residual Heat Removal (RHR) Loop keep-fill system were noted. There are no other indications that the valve failed prior to February 11, 1999. A conservative assessment of safety significance of having the 'B' loop of RHR out of service from the date of the performance of the last stroke time test of the valve (November 19, 1998) was performed. This assessment used the new NRC event evaluation methodology. The assessment included a consideration of the most significant system outages coupled with failure of valve HV-151F017B. In all cases, the significance color was green. Therefore, failure of valve HV-151F017B is a low safety consequence event.

A handwritten signature in cursive script that reads 'R F Saunders'.

Robert F. Saunders
Vice President - Nuclear Site Operations

Attachment

cc: Mr. H. J. Miller
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cc: Mr. S. L. Hansell
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