



**Northeast
Nuclear Energy**

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Millstone Nuclear Power Station
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The Northeast Utilities System

NOV - 3 1999

Docket No. 50-245
B17912

U. S. Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555

Millstone Nuclear Power Station, Unit No. 1
Revised Fuel Handling Accident Analysis - Additional Information

The purpose of this letter is to provide the NRC Staff with additional information on the Millstone Unit No. 1 revised fuel handling accident analysis assumptions and results.

On April 19, 1999,⁽¹⁾ Northeast Nuclear Energy Company (NNECO) submitted a request for revised technical specifications that reflected the permanently shutdown and defueled condition of Millstone Unit No. 1. This information was supplemented by NNECO's letters of August 25, 1999,⁽²⁾ and October 14, 1999.⁽³⁾ Recently, telephone conversations have been conducted with the Staff on the NNECO submittals. The NRC questions and NNECO responses are contained in the attachment to this letter.

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- (1) Letter from R. P. Necci to U. S. NRC, B17621, "Proposed Revision to Technical Specifications, Permanently Defueled Technical Specifications," dated April 19, 1999.
 - (2) Letter from R. P. Necci to U. S. NRC, B17829, "Supplement No. 1 to the Permanently Defueled Technical Specifications, (TSCR 1-1-99)," dated August 25, 1999.
 - (3) Letter from F. C. Rothen to U. S. NRC, B17900, "Revised Fuel Handling Accident Analysis Assumptions and Results," dated October 14, 1999.

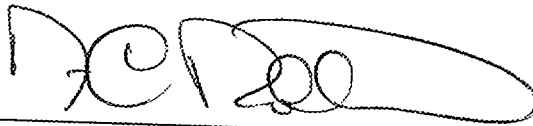
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There are no regulatory commitments contained within this letter.

If you should have any questions, please contact Mr. Bryan S. Ford at
(860) 437-5895.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY



F. C. Rothen
Vice President - Nuclear Work Services

Subscribed and sworn to before me

this 3 day of November 1999

Donna Lynne Williams
Notary Public

Date Commission Expires: Nov 30, 2001

Attachment: Revised Fuel Handling Accident Analysis - Additional Information

cc: H. J. Miller, Region I Administrator
L. L. Wheeler, NRC Project Manager, Millstone Unit No. 1
P. C. Cataldo, NRC Inspector

Director
Bureau of Air Management
Monitoring and Radiation Division
Department of Environmental Protection
79 Elm St.
Hartford, CT 06106-5127

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Attachment

Millstone Nuclear Power Station, Unit No. 1

Revised Fuel Handling Accident Analysis - Additional Information

November 1999

Question 1

Please identify what code was used in calculating core inventories.

Response 1

The ORIGEN-2 code was used in calculating core inventories.

Question 2

Please provide the whole body doses following a postulated fuel handling accident at Millstone Unit No. 1.

Response 2

Please refer to the table presented below.

Location	Ventilation Flows / Unfiltered Inleakage	Doses (REM)		Skin dose (REM)	
		Thyroid	Whole Body*		
EAB	NA	5.443E-04	1.027E-03	4.963E-02	
LPZ	NA	1.695E-05	3.198E-05	1.546E-03	
MP-1 Control Room	2600 cfm 300 cfm	0 - 5 sec 5 sec - 30 days	9.374E-02	1.073E-01	2.665E+01
MP-2 Control Room	800 cfm 130 cfm	0 - 5 sec 5 sec - 30 days	7.647E-02	8.673E-02	2.194E+01

* The regulatory acceptance criterion is stated in terms of whole body dose. The doses shown are those values calculated as TEDE values. Use of the TEDE values for presenting data on whole body doses is conservative.

Note: This clarification to the table does not change the fuel handling accident summary of results submitted with the October 14, 1999, letter.