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October 26, 1999

2CAN109905

U. S. Nuclear Regulatory Commission
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Subject: Arkansas Nuclear One - Unit 2
Docket No. 50-368
License No. NPF-6
Revised Leak Rate for Use During Special Steam Generator Inspection
(TAC NO. MA6224)

Gentlemen:

By letter dated July 29, 1999 (2CAN079903), Entergy Operations submitted a proposed technical specification change to support the special steam generator (SG) inspection to be performed at Arkansas Nuclear One, Unit 2 (ANO-2), during the 2P99 mid-cycle outage scheduled to begin on November 5, 1999. The proposed inspection scope and expansion criteria were submitted for review on August 6, 1999 (2CAN089901). Based upon a review of these submittals, the NRC Staff issued a request for additional information on October 8, 1999 (2CNA109901), to which Entergy Operations' responded by letter dated October 14, 1999 (2CAN109903).

Based upon a teleconference with the Staff held on October 20, 1999, Entergy Operations agreed to modify the leak rate to be used as an expansion criterion for circumferential indications. The total leakage rate from in-situ pressure tested circumferential cracks has been established at 0.04 gpm. This rate is more conservative than the 0.046 gpm leak rate calculated for the ANO-2 Cycle 14 Operational Assessment. Thus, the expansion criterion, as modified, is:

- Failure of the circumferential indications in the "A" SG that meet the testing criteria to pass in-situ testing requirements of less than 0.04 gpm total leakage at main steam line break pressure (~2788 psig at room temperature*) or burst of any single flaw at less than 3ΔP (~4515 psig at room temperature*) will cause an expansion. The expansion will be to inspect the top-of-tubesheet region of all inservice non-sleeved tubes in the "A" SG and a random 20% of the inservice non-sleeved tubes in the "B" SG.

* Pressure is adjusted for room temperature by 11.5%.

A001

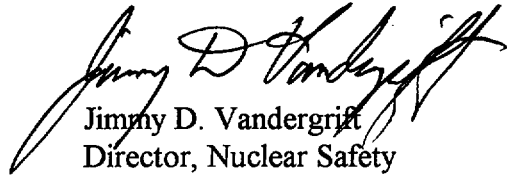
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If you have any questions concerning this submittal, please contact me.

Very truly yours,



Jimmy D. Vandergriff
Director, Nuclear Safety

JDV/jjd

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