

November 26, 1997

SECY 97-272

For: The Commissioners

From: James L. Blaha, Assistant for Operations, Office of  
the EDO

Subject: WEEKLY INFORMATION REPORT - WEEK ENDING NOVEMBER  
21, 1997

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\*No input this week.

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Office of Nuclear Reactor Regulation  
Items of Interest  
Week Ending November 21, 1997

November 6, 1997, Meeting with Westinghouse Owners Group RE: Fuel Cladding Oxidation/ 10 CFR 50.46(b)

On October 28, 1997, Westinghouse (W) notified NRC (by telecon) and its licensees (36 plants, including Fort Calhoun, a CE design) that it had discovered a non-conservatism in the oxidation model for high burnup Zr-4 clad integral fuel burnable absorber (IFBA) fuel. The model was non-conservative when compared to data for IFBA fuel with greater than about 39-40K MWD/MTU burnup. Information provided by W indicates that this is not a safety issue, but there is a question of compliance with 10 CFR 50.46 (b)(2) (> 17% local cladding oxidation). On November 6, 1997, the staff met with W and the W Owners Group (WOG) in a public meeting to verify its safety determination and discuss the compliance issue (present compliance for each plant, continued near term compliance, including compensatory actions for those plants reaching burnups at which non-compliance was projected, and long-term resolution). At the meeting, the WOG identified that one plant was probably about to enter into non-compliance but was shutting down for a scheduled refueling outage. For the remainder of the plants, W presented assessment criteria to identify plants and times at which it could no longer assure compliance.

The WOG committed to provide formally to the staff information presented at the meeting including a list of affected plants, the projected dates of potential noncompliance, and plans for resolution. The WOG also indicated that each affected plant (the next plants would be affected in about one month to six weeks) would take appropriate actions per 10 CFR 50.46 (a)(3)(ii) when it became affected and indicated that compensatory actions could include peaking limitations or derating. W would continue to do more detailed assessments for plants and make timely assessments with regard to the compliance issue. In the longer term, W plans to correct its model and begin implementing it by August 1998.

The staff concluded that the W licensees are in present compliance with 10 CFR 50.46 requirements and that the overall plan offered the licensees a means to show continued compliance with both the reporting and the analytical requirements of 10 CFR 50.46.

Palo Verde Unit 2

Steam Generator Tube Eddy Current Inspection Results

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ENCLOSURE A

Arizona Public Service Company (APS), the licensee for Palo Verde Unit 2 (PV-2), has completed its steam generator (SG) tube eddy current (EC) inspections for the current refueling outage (end-of-cycle 7). Through its bobbin coil probe inspections, APS identified 54 pluggable wear indications in the batwing-stay cylinder area. (The batwings are diagonal straps that provide support for the SG tubes. The stay cylinder region is an untubed area located in the center of the tube bundle.) The structural limit for SG tube wear is 75% throughwall, but the licensee implements a conservative plugging limit of 20% throughwall to account for high growth rates in this region. PV-2 reported 51 pluggable batwing wear indications in the #22 SG and 3 pluggable indications in the #21 SG. The licensee attributed the difference in number of pluggable indications in each SG to the shroud modifications made in the #22 SG. These modifications increased the flow rates through the SG resulting in increased wear rates. APS used an Electric Power Research Institute (EPRI) Appendix H qualified technique to size the wear indications; the largest of the batwing wear indications was sized at a depth of 70% throughwall.

In a Request for Additional Information (RAI) dated October 2, 1997, the staff requested the licensee to discuss APS' decision to not *in situ* pressure test or pull the tube with the 70% throughwall indication to ensure adequate leakage and structural integrity over the past operating cycle was maintained. In developing its response to the RAI, the licensee identified several significant issues associated with the EPRI qualification of the technique used to size wear indications. An assessment of these issues resulted in APS concluding the sizing technique used in past inspections may have been nonconservative. The licensee plans to document its position in the RAI response including an operability determination for PV-2 as well as similarly affected SGs in the other Palo Verde units. APS also notified EPRI and the Combustion Engineering Owners Group of its findings.

#### Boiling Water Reactor (BWR) Fall Outage Issues

##### Hope Creek Jet Pump Sensing Line Cracking and Core Spray Inlet Nozzle Cracking

During RFO7, the licensee found cracking indications on the jet pump sensing line (JPSL) lower brackets for Jet Pumps 8 and 15 and found a failure of the same weld on Jet Pump 9. Cracks or failures of the sensing lines were not observed. The sensing lines are 0.25-inch pipes which are welded to the jet pump diffuser by two 0.5-inch square support brackets; the lower of which were found to be cracked and failed. The licensee has installed semi-circular ring clamps to restrain the JPSL against their support brackets (replacing the original welds). Similar cracks were

observed at Susquehanna Unit 2 in 1994, and similar clamps were installed at that time. Subsequent inspections of the clamps have shown that the clamps are still in place.

Additionally, the licensee found a stress corrosion cracking (SCC) crack in the weld butter of the core spray inlet nozzle. The licensee expanded its inspection to the other core spray nozzle and four other reactor coolant system nozzles, and did not find any additional cracking. The examinations were performed by qualified automated ultrasonic testing systems.

The staff is continuing to evaluate the licensee's evaluations.

#### Peach Bottom Unit 3 Jet Pump Riser Elbow Cracking

The licensee has found cracks on the thermal sleeve to jet pump riser elbow side of weld RS-1 in the heat affected zone on three jet pump risers (jet pumps 1 and 2, 9 and 10, and 13 and 14). The cracks are 10.8, 1.7, and 12.7 inches, respectively. All other jet pump assemblies have been inspected at jet pump riser locations, and no further cracking was detected. The other welds on the risers were also inspected.

The licensee has completed a fracture mechanics evaluation of the cracking utilizing industry topical report BWRVIP-41, "BWR Jet Pump Assembly Inspection and Flaw Evaluation Guidelines," which is presently under staff review. The licensee has determined that continued operation (The unit is presently at 100% power.) is justified for several months and that if flow is reduced to 80% of rated (approximately 92% power), the licensee has stated that fatigue crack growth will be bounded for operation as long as 8 months. Further, the licensee has stated that it plans on installing a repair to jet pumps 1 and 2 and to 13 and 14 within the time frame of the analysis.

The staff is evaluating the licensee's evaluation, especially the technical significance of combining fatigue and SCC crack growth rates, and is reviewing if any additional regulatory actions may be necessary.

#### Hatch Unit 1 Core Plate Support Ring and Jet Pump Riser Cracking

The licensee found a 0.3-inch long vertical crack on the outside diameter of the core plate support ring segment weld. The inspection scope was expanded to one other weld (The additional four welds in this segment are

inaccessible.), and no other indications were found. The licensee also found cracking in the heat affected zones of the elbow to thermal sleeve weld in two of the ten jet pump inlet nozzles (N2B and N2D). The cracks are 1.625 inches at the 12:00 position on the sleeve side of nozzle N2B and 2.178 inches at the 12:00 position on the elbow side of nozzle N2D. The licensee expanded its inspection scope to include the accessible portions of welds at the elbow to riser, riser to transition piece, riser brace, jet pump diffuser, and jet pump adaptor locations on the associated jet pumps, and the licensee found no additional cracking.

The licensee performed an evaluation of the above cracking and determined that an additional cycle of operation was acceptable. Further, the licensee committed to re-inspecting during the next outage to determine crack growth.

The staff is evaluating the licensee's evaluation and is continuing discussions with the licensee on this issue.

#### Crystal River Unit 3 Restart Progress

##### Licensing Submittals

Florida Power Corporation (FPC or the licensee) continues to maintain its target for completing the corrective actions and restarting the facility by December 1997. FPC identified 37 potential licensing submittals that would require NRC approval prior to restart in December 1997. Additionally, the staff, during its recent Safety System Functional Inspection (SSFI), identified several unresolved issues, and the region submitted four Task Interface Agreements (TIA) for NRR determination whether the issues must be resolved prior to restart. The current status of these issues is as follows:

##### Status of Licensing Submittals That are Required for Restart

Total Identified (including the four TIAs)	41
Awaiting initial submittal	6
Received and review in progress	18
Completed / Closed	17

On November 6, 1997, in a telephone conference with the staff, FPC indicated that it has made preliminary assessments using the Generic Letter 91-18, Revision 1, guidance to determine operability for several identified Unreviewed Safety Questions (USQ). Based on the licensee's preliminary assessments, FPC believes that its amendment requests for six USQs would not require staff approval prior to restart. The staff plans to independently review FPC's position on these six issues.

### Salem Unit 1

Operational Status - Salem Unit 2 is operating at 100% power. Salem Unit 1 is shut down and defueled. The licensee's target date for restart is February 1998.

Salem Assessment Panel - The next meeting of the Salem Assessment Panel will be held at the site on November 20, 1997. Following the meeting, the Panel will meet with the licensee to review the items that have to be closed prior to restart of Salem Unit 1.

Licensing Actions - One amendment is required to support restart of Salem Unit 1 to incorporate parameters from the Margin Recovery Program. These parameters have been used in the Chapter 15 accident analyses in the 10 CFR 50.59 evaluations that were developed by the licensee for the replacement of the steam generators.

### Peach Bottom Unit 3

On November 10, 1997, the staff (NRR, Region I, and Consultant) discussed the information submitted regarding the indications in three jet pump riser welds with PECO Energy and their consultants. During the conference call, PECO Energy committed to provide the staff the following additional information:

- General Electric's analysis of a design base accident-loss of coolant accident with jet pump riser failure;
- a submittal date for their reply to our RAIs of November 7, 1997, regarding the jet pump crack analysis (the timing for PECO's response to our August 14, 1997, RAIs on ECCS pump NPSH will be determined after the staff's review of the above stated GE analysis); and
- a letter discussing their operational strategy regarding power, flow, and time limits (keeping the plant operation within the bounds of the piping analysis).

### Inoperable Safe Shutdown Paths and Early Unit 2 Maintenance Outage

On September 26, 1997, the licensee entered a 67-day limiting condition for operation (LCO) because all safe shutdown (SSD) paths were determined to be inoperable. In the Quad Cities SSD analysis, all cables within a fire area are postulated to fail in the event of a fire, except for those cables provided with passive fire protection. The SSD analysis relies on the opposite unit's equipment to help place and maintain the affected unit in a safe shutdown condition. This assumes that the SSD equipment is not affected by the fire and that the breakers for the non-SSD

loads on the same bus as the SSD equipment would be opened or shed from the bus. If the non-SSD load breakers are not shed from the bus and the cables for these loads pass through the fire zone, damage to these cables could cause loss of power to the SSD equipment on that bus for the unaffected unit.

Upon review of the Fire Protection Report (FPR) the station discovered two issues with the implementation of the SSD analysis. The first issue discovered was that even though the SSD analysis assumes shedding non-SSD loads, the station procedures do not implement all of the assumed load shedding. The second discovery was that the analysis that addresses the ability to shutdown the unaffected unit after shedding of non-SSD loads could not be located. The licensee did not have assurance that the SSD paths were operable. There is no analysis in place that ensures that equipment used to safely shutdown the unaffected unit will not be affected by fire in the affected unit; therefore, the licensee declared all SSD paths inoperable.

Update: As of November 12, 1997, the licensee is still completing review of the SSD procedures for Unit 1. Unit 1 is presently operating in day 47 of the 67-day administrative LCO while Unit 2 is shutdown. Unit 2 has completed Maintenance Outage Q2P01; however, startup is not planned until the procedures for Unit 2 have been reviewed/ revised as necessary. Instead of revising the current SSD procedures to include load shedding on the opposite unit, the licensee is now revising the SSD procedures to include the station blackout diesel as an alternate power source. The licensee projects completing the Unit 1 work prior to the expiration of the 67-day LCO. Unit 2 startup is projected to be in late December.

There was a meeting at the site on November 20, 1997, to address these issues with the licensee. NRC staff participation included Plant Systems, Projects, Region III, and the resident inspector staff.

### Braidwood Units 1 and 2

#### Waterhammer at Braidwood, Unit 2

Braidwood, Unit 2 is in startup following completion of a refueling outage. With the unit in Mode 3, the licensee was attempting to open an air-operated control valve in a 6-inch feedwater tempering line to the upper steam generator nozzle on the "D" steam generator. The nozzle is also used for auxiliary feedwater. The valve did not respond, so the licensee increased the air pressure. The valve suddenly opened. The piping downstream of the valve had been voided, and a resulting water hammer occurred. The event happened at about 2:00 a.m. (CST) on November 10, 1997.

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ENCLOSURE A



Upon immediate visual inspection of the system, the licensee reported the following findings: (1) a spring can pipe support was damaged, (2) a box guide pipe support was damaged and pulled out of the wall, and (3) two snubbers on the 16 inch main feedwater line were challenged.

So far, the licensee has used magnetic particle and ultrasonic testing to inspect the steam generator nozzle. ComEd found no damage to the nozzle. ComEd is presently inspecting the welds in the tempering line. The spring can has been repaired. Most likely, the licensee will have to drill new holes to reattach the box guide. The two snubbers on the main feedwater line were tested and were found to be operable. The licensee also tested two additional snubbers downstream of the two that were challenged, and they were also found to be operable. A root cause analysis is underway. The licensee is expecting to have all actions related to this event completed within the next few days.

Region III has sent an specialist inspector to the site and plans to send one or two on November 13, 1997, to monitor and evaluate the licensee's activities.

Office of Nuclear Material Safety and Safeguards  
Items of Interest  
Week Ending November 21, 1997

Workshop on the Revision of 10 CFR Part 35

On November 12-14, 1997, the NRC staff conducted the second in a series of public workshops to discuss the fundamental approaches and issues that need to be addressed in revision of 10 CFR Part 35. This workshop, held in Chicago, Illinois, was attended by representatives of the Part 35 Working Group and Steering Group. Invited participants included nuclear medicine physicians; physician specialists (such as cardiologists, endocrinologists, and radiologists); medical and health physicists; technologists; nurses; radiopharmaceutical manufacturers; hospital administrators; patients' rights advocate; and Agreement State regulators. Topics discussed included risk assessment; the NRC's 1979 Medical Policy Statement; and rule alternatives for the Quality Management Program, patient notification, threshold for reportable events, radiation safety committee, and training and experience criteria.

Joint Guidance on Mixed Waste Testing

On November 7, 1997, the NRC and the Environmental Protection Agency (EPA) completed development of final guidance on the testing requirements for mixed waste. Mixed waste is defined as hazardous waste that also contains source, special nuclear, or byproduct material. Because mixed waste contains both hazardous waste and radioactive material, it is regulated by both the NRC or an NRC Agreement State and the EPA or an EPA authorized State. In the past, mixed waste generators have expressed concern that EPA's testing requirements for the hazardous component of the mixed waste could result in increased radiation exposures.

The new guidance emphasizes the use of process knowledge to determine if waste is hazardous in order to avoid unnecessary exposures to radioactivity, and it provides guidelines to be used by generators wishing to rely on process knowledge as the basis for evaluating their waste. The guidance also discusses using small volume samples and surrogate materials as well as alternative sampling and testing procedures, such as representative drum sampling or sampling from drums containing lower concentrations of radioactive material.

Completion of this guidance is the culmination of a multi-year effort by the two agencies to provide mixed waste managers with practical methods for complying with the regulatory requirements of the NRC and EPA. The agencies plan to publish the final guidance in the Federal Register.

### International Physical Protection Advisory Service Mission to Poland

The International Atomic Energy Agency (IAEA) has requested that a member of the Division of Fuel Cycle Safety and Safeguards staff represent the U.S. in an "International Physical Protection Advisory Service" mission to Poland from November 24-December 2, 1997. These missions were established to provide advice and assistance to IAEA member states to strengthen and enhance the effectiveness of the state's system for physical protection of radioactive materials. The mission team will also include experts from Germany, France, and the United Kingdom with broad experience in design and implementation of physical protection systems and some knowledge of regulatory oversight.

### Work Stoppage at Westerman Companies

On November 11, 1997, Lockheed Martin Utility Services (LMUS), a contractor to U.S. Enrichment Corporation (USEC), informed the NRC that they had stopped all fabrication activities at Westerman Companies, a fabricator of UF6 cylinders for USEC. The stop work order was issued by LMUS, not by NRC, based on findings identified during an August 25-29, 1997, NRC inspection at Westerman. LMUS also informed the NRC that they were reviewing Westerman's corrective actions and anticipated restart of fabrication activities at Westerman in the near future. Prior to restarting fabrication, LMUS stated that they would assess the current conditions to ensure that there were no outstanding issues.

### Inspection of Transportation Packaging Test Activities at Amersham Corporation

On November 17-18, 1997, members of the Spent Fuel Project Office staff inspected test activities associated with the Model-660 transportation packaging at Amersham Corporation in Burlington, Massachusetts. The inspectors reviewed test plan implementation and test data for accident condition tests performed as described in the NRC Confirmatory Action Letter dated June 10, 1997. There were no inspection findings, and an exit meeting was held with Amersham on November 18, 1997, to present the results of the inspection.

Amersham performed tests on four test specimens (units). Each unit was drop tested from 30 feet in different orientations onto an essentially unyielding surface. One of the units, test unit D, experienced some damage. Unit C passed the drop test and, due to minimal damage, was not thermal tested. Units A, B, and D were thermal tested. Units A and B passed the test, but unit D did not. As a result of the failure, Amersham sent a letter containing a Justification for Continued Operation (JCO) for the

Model-660 packaging to NRC on November 17, 1997. The NRC staff is currently reviewing the JCO.

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ENCLOSURE B

Office of Nuclear Regulatory Research  
Items of Interest  
Week Ending November 21, 1997

Workshop on Review of Dose Modeling Methods for Demonstration of Compliance with Radiological Criteria for License Termination

The NRC staff is developing guidance for implementing the final rule on radiological criteria for license termination, which includes identification of criteria for evaluating acceptability of dose models and parameter values for demonstrating compliance with the final rule. In support of this effort, RES and NMSS staff jointly organized a 2-day public workshop on the review of dose modeling methods. The workshop, which was announced in the Federal Register (62 FR 51706), was held in the auditorium at the NRC Headquarters on November 13-14, 1997. Presenters at the workshop included NMSS and RES staff who discussed dose modeling needs for licensing reviews and development of guidance related to dose modeling and parameter selection needs; Department of Energy (DOE) national laboratory scientists who provided responses to earlier NRC staff-developed questions and discussed their various Federally sponsored dose models (i.e., D&D, RESRAD, and MEPAS codes); and an Environmental Protection Agency (EPA) scientist who presented details on the EPA dose assessment model (i.e., PRESTO code). During the lunch hour and afternoon break on November 14, 1997, computer demonstrations of the dose codes were provided to the attendees. Over 120 individuals attended the workshop including those from NRC Headquarters and the regions; those from the Agreement States; industry representatives and consultants; staff members from EPA, Department of Defense, Defense Nuclear Facility Safety Board, and DOE; national laboratory scientists; and interested members of the public. A transcript of the meeting will be placed in the Public Document Room in one week. Under preparation is a NUREG/CP report of the workshop proceedings that will outline the presentations made during the workshop and address questions raised during the panel discussions. This document is designed to provide technical bases and discussions to support development of the decommissioning guidance.

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ENCLOSURE C

Office for Analysis and Evaluation of Operational Data  
Items of Interest  
Week Ending November 21, 1997

Visit of Spanish Regulatory Staff

On November 18, 1997, Mr. Javier Zarzuela, head of the Operational Experience and Training section of the Spanish Nuclear Safety Commission (CSN), and Dr. Marta Marono, a contractor to the CSN, visited AEOD to discuss the NRC's Performance Indicator (PI) and Accident Sequence Precursor (ASP) programs. The CSN uses NRC PI software and methodology for their PI program. They include the Spanish plants in the appropriate U.S. peer groups to compare the performance of their plants to that of similar U.S. plants. (The Spanish operate six Westinghouse plants, two General Electric plants, and one Kraftwerk Union plant.) The AEOD staff discussed with Mr. Zarzuela and Dr. Marono several issues related to PIs, including Licensee Event Report coding, future PI developments, reliability data and risk-based PIs, and the INPO EPIX system. The CSN is also planning to undertake an ASP program next year that will be patterned after the AEOD program. AEOD staff discussed several options that the CSN is considering for beginning their ASP program and made recommendations to them based upon the NRC's experience.

Visit of Hungarian Atomic Energy Authority Personnel

On November 18, 1997, Dr. Ferenc Kolonits, Department Head in the Hungarian AEA Nuclear Safety Directorate, and Dr. Gabor Redey of his staff, visited AEOD's Safety Programs Division to discuss nuclear plant aging. The AEOD staff described the results of the study on aging which they performed for the Nuclear Energy Agency. The staff also provided them a copy of the study report entitled, *Evidence of Aging Effects on Certain Safety-Related Components*, NUREG/CR-6642.

Air-Operated Valves

As part of its study on air-operated valves (AOV), AEOD, its contractor, INEEL, (two engineers), and a staff member from RES visited Palisades Nuclear Plant on November 4 and 5, 1997.

As part of the AOV program at Palisades, the AOVs were ranked or categorized by an expert panel using insights from the maintenance rule evaluations of systems importance, and from a review of the plant's inservice testing (IST) program. The plant's probabilistic risk assessment (PRA) was not used for the first cut of determining which AOVs are to be included in the plant's AOV program. However, insights from the PRAs are being

used to further define the important valves which are to be included in the program.

The Palisades plant has a large number of important AOVs which are included in the AOV program. The purpose of the AOV program is to verify the design of AOVs to assure that they have adequate margins when they are required to function during design basis events.

The Palisades plant has performed static diagnostic testing and is planning to perform dynamic and additional static testing early next year. Presently, the licensee is planning to run comparative tests among three different manufacturers' AOV diagnostic equipment.

Independent of the AOV design, AEOD found significant air system deficiencies which were discussed with the licensee at the exit meeting.

#### Calvert Cliffs Exercise

Two members of Incident Response Division participated with Region I staff as an ingestion team in the Calvert Cliffs ingestion exercise on November 19 and 20, 1997. This effort was part of the AEOD State Outreach program intended to gain more Federal participation with States in licensee intermediate phase exercises. At the same time, AEOD escorted two members of the Nuclear Safety Directorate of the Hungarian Atomic Energy Authority, who observed both the plume and ingestion phases of the exercise.

#### PRELIMINARY NOTIFICATIONS

1. PNO-I-97-066A, Cabot Corporation, UPDATE - RETURN OF INTERMODAL CONTAINERS TO CABOT CORPORATION
2. PNO-III-97-092, Shannon and Wilson, Inc., NOTIFICATION OF DAMAGED MOISTURE/DENSITY GAUGE AND INABILITY TO RETURN SEALED SOURCES TO SHIELDED POSITION
3. PNO-IV-97-066, Texas Utilities Electric Co. (Comanche Peak 2), NEW FUEL DAMAGE
4. PNO-IV-97-067, Texas Utilities Electric Co. (Comanche Peak 1), ELECTROCUTION AT COMANCHE PEAK UNIT 1

Office of Administration  
Items of Interest  
Week Ending November 21, 1997

OWFN Security Camera Upgrades

Division of Facilities and Security representatives met with the General Services Administration and their contractor to coordinate the exterior site work needed to add additional cameras on the White Flint Complex. The work consists of new concrete foundations for three new poles and additional wiring to connect the new color cameras to our existing network of closed-circuit television cameras. The existing black and white cameras will be also be replaced with color cameras which will provide clearer enhanced pictures. The exterior work is expected to begin the week of November 24, 1997, and be completed within 60 days. The camera replacement is expected to begin in February 1998 and be completed in May 1998.

Pilot Course

On November 20, 1997 representatives from AEOD, NRR, and RES attended an in-house pilot course entitled, "Contract Administration/Property Management." This course which was taught by representatives from the Division of Contracts and Property Management, RES, and NRR is one of ten course modules to be offered to NRC Project Managers. At the end of the pilot, the attendees were able to track project costs against a spending plan, properly approve or disapprove vouchers, remedy performance problems, and modify or terminate a contract or agreement. The course was well received by participants who attended the pilot. The course will be finalized, based upon the comments received from the class participants and will be made available on a regular basis starting in calendar year 1998.

Procedures Applicable to Proceedings for the Issuance of Licenses for the Receipt of High-Level Radioactive Waste at a Geologic Repository (Part 2)

A proposed rule that would amend the Commission's Rules of Practice for the licensing proceeding on the disposal of high-level radioactive waste at a geologic repository was published in the Federal Register on November 13, 1997 (62 FR 60789). The proposed rule would allow the application of technological developments that have occurred since the original rule was issued in 1989 while retaining the goal of facilitating the Commission's ability to comply with the schedule for a decision on the construction authorization established by the Nuclear Waste Policy Act. The comment period for this action closes January 27, 1998.



Chief Information Officer  
Items of Interest  
Week Ending November 21, 1997

Freedom of Information and Privacy Act Requests Received during  
the 5-Day Period of November 14-20, 1997:

Records on self. (FOIA/PA-97-447)

Western Nuclear Corp., uranium-bearing ores. (FOIA/PA-97-448)

Executive Order 12898, environmental justice strategy.  
(FOIA/PA-97-449)

Correspondence by named individual re performance of named  
employee. (FOIA/PA-97-450)

Commonwealth Edison Co., plant issues matrix for Braidwood,  
Byron, Dresden, LaSalle, Quad Cities, and Zion. (FOIA/PA-97-451)

OIG report, 97-40G. (FOIA/PA-97-452)

Request for Quotation 60780016, Training for Environmental Impact  
Statements & Assessments. (FOIA/PA-97-453)

Correspondence by named individual re performance of named  
employee. (FOIA/PA-97-454)

Crystal River, radiological effluent technical specifications  
implementation (EGG-PHYS-6171), specific exhibits.  
(FOIA/PA-97-455)

Contract, NRC-33-95-173, performance reviews. (FOIA/PA-97-456)

Office of Human Resources  
Items of Interest  
Week Ending November 21, 1997

Eye Disease Education and Vision Screening Program Conducted

During the month of November, the Employee Health Center conducted an Eye Disease Education and Vision Screening program. Thirty-six employees attended the video presentation on November 4, 1997, which focused on glaucoma and diabetic eye disease. Sixty-three people signed up for the vision screening to be conducted by the Lions Club on November 18 and 19, 1997. The program was presented in conjunction with the National Diabetes Month Eye Initiative sponsored by the National Eye Health Education Program Partnership.

Employee Assistance Program Association (EAPA) Conference Attended

On November 16-19, 1997, the Employee Assistance Program Manager attended the National EAPA Conference held in Baltimore. Topics covered at the workshops included chemical dependency, mental health treatment and prevention issues, work/family programs, violence in the workplace, and organizational change.

<b>Arrivals</b>		
BLUMBERG, William	HEALTH PHYSICIST (PFT)	NRR
KHAN, Tariq	REACTOR SYSTEMS ENGINEER (PFT)	NRR
LIEBERMAN, Jodi	INT'L POLICY ANALYST (PFT)	OIP
SUTTORA, Linda	ENVIRONMENTAL SCIENTIST (PFT)	NMSS
<b>Retirements</b>		
None		
<b>Departures</b>		
None		

Office of Small Business & Civil Rights  
Items of Interest  
Week Ending November 21, 1997

Minority Enterprise Development Week

The Office of Small Business and Civil Rights participated in the 15th Annual Minority Enterprise Development Week (MED Week) held at the Sheraton Washington Hotel on November 21-22, 1997. Featured speakers were the Honorable Richard Gephardt, Minority Leader, U.S. House of Representatives, and the Honorable Aida Alvarez, Administrator, U.S. Small Business Administration. Approximately 15 Federal agencies were represented. Von Deloatch attended the conference and manned the NRC exhibit booth providing information about potential procurement opportunities to approximately 350 minority firms. Seminar topics included Opportunities in the Technology and Communications Industries, Federal Procurement Opportunities in a Post-Adarand Environment, and Emerging International Markets for the Minority Entrepreneur.

8(a) Program Workshop

On November 18, 1997, Mr. Deloatch participated in a one-day workshop for businesses that recently joined the 8(a) Program. The workshop was sponsored by the Small Business Administration (SBA) and focused on marketing strategies that 8(a) firms might use to become more competitive in the Federal Procurement Program. Approximately 200 8(a) firms were in attendance.

Office of State Programs  
Items of Interest  
Week Ending November 21, 1997

Environmental Justice Subcommittee Meeting

Rosetta Virgilio attended the November 20, 1997, meeting of the Interagency Working Group (IWG) on Environmental Justice (EJ) Subcommittee on Policy and Coordination held in Washington, DC. A representative of the White House, Council on Environmental Quality (CEQ), provided members of the IWG with a copy of the pre-publication draft of the CEQ "Guidance for Considering Environmental Justice Under the National Environmental Policy Act (NEPA)" dated November 19, 1997. The availability of the Final CEQ EJ NEPA guidance is expected to be announced mid-December via press conference.

As a result of a November 19, 1997, meeting between the White House, CEQ, and African-American environmental leaders, a White House Conference on Environmental Justice (WHCEJ) will be scheduled on or about the 4th anniversary of Executive Order 12898, "Federal Actions to Address Environmental Justice in Minority Populations and Low-Income Populations" (February 11, 1998). Members of the IWG were asked to provide by December 2, 1997, an appropriate listing of any additional agency representatives and any particular environmental justice constituency to be invited to attend the WHCEJ as well as any new initiatives for achievement of the goals of the Executive Order. On or about December 12, 1997, an appropriate White House reaffirmation of agency commitments under the Executive Order will take place. If details of the WHCEJ are available at that time, the communication will include a request for Cabinet-secretary-level participation in the conference.

The second "Report to the President on Environmental Justice," although finalized, was never submitted to the President for approval. Because the contents of the draft report have become somewhat stale and should include new issues raised at the aforementioned November 19 meeting, it will be circulated through the Office of Management and Budget for revision. The CEQ will provide specific direction to agencies on new areas to be addressed and estimates having the final report to the President by December 31, 1997.

Office of Public Affairs  
Items of Interest  
Week Ending November 21, 1997

Media Interest

HQ provided information and arranged an interview for a Knight-Ridder reporter with an NRR official concerning license renewal at Oconee.

There were media inquiries in Region IV after Union Electric declared an Unusual Event because of a security threat at Callaway.

The Walnut Creek Field Office coordinated a news conference in Phoenix for Commissioner Diaz and assisted with a public meeting on the employee concerns program at San Onofre.

Press Releases	
Headquarters:	
97-169	Note to Editors: ACRS Meeting on December 3
97-170	Note to Editors: ACRS Reports
97-171	Note to Editors: ACRS Meeting on December 2
97-172	Note to Editors: ACRS Meeting on December 11-12
97-173	NRC Advisory Committee on Reactor Safeguards to Meet in Rockville, Maryland, on December 3-6
97-174	Nuclear Regulatory Commission and Department of Energy Sign Memorandum of Understanding on Pilot Program for External Regulation
Regions:	
I-97-149	Note to Editors: Millstone Meetings on November 20 and December 2
I-97-150	NRC Recognizes Improvements at Haddam Neck; Modifies Some Restrictions on Radiological Controls
II-97-81	NRC Staff to Hold Predecisional Enforcement Conference to Discuss Robinson Nuclear Power Plant Issues with CP&L
III-97-98	NRC Staff to Discuss Inspection Findings on November 19 at Palisades Nuclear Power Station
III-97-99	NRC Staff Proposes \$16,000 Fine Against U.S. Army

III-97-100	NRC to Hold Predecisional Enforcement Conference with Michigan Department of Transportation on Apparent Violations
III-97-101	NRC Staff Proposes \$100,000 Fine for Two Violations of NRC Requirements at Perry Nuclear Power Plant
IV-97-69	NRC Proposes \$5,000 Fine Against the Terracon Companies for Violation of Radiation Safety Requirements
IV-97-70	NRC Commissioner Diaz to Meet With News Media Following Visit to Palo Verde Nuclear Plant

Office of the Secretary  
 Items of Interest  
 Week Ending November 21, 1997

<b>Decision Documents Released to the Public</b>		
Document	Date	Subject
1.                   SECY- 97-146	7/11/9 7	NRC's Mandated and Non-Mandated Activities and Revisions to Agency Policy on Performing Reimbursable Work for Others
- SRM on 97-146	11/5/9 7	(same)
- Commission Voting Record on 97- 146	11/5/9 7	(same)
2.                   SECY- 97-188	8/18/9 7	Formerly Licensed Sites Identified for Further Investigation in Agreement States
- SRM on 97-188	11/7/9 7	(same)
- Commission Voting Record on 97- 188	11/7/9 7	(same)
3.                   SECY- 97-228	10/3/9 7	Final Amendments to 10 CFR Part 73, "Changes to Nuclear Power Plant Security Requirements"
- Commission Voting Record on 97- 228	11/5/9 7	(same)
4.                   SECY- 97-232	10/9/9 7	Final Rule on Exempt Distribution and Use of a Radioactive Drug Containing One Microcurie of Carbon 14 Urea (Parts 30 and 32)
- Commission Voting Record on 97- 232	11/5/9 7	(same)

<b>Negative Consent Papers Released to the Public</b>			
1.	SECY- 97-214	9/24/9 7	Changes to 10 CFR Part 72, Expand Applicability to Include Certificate Holders and Applicants and Their Contractors and Subcontractors
	- SRM on 97-214	11/4/9 7	(same)
2.	SECY- 97-224	10/1/9 7	Creation of a Research Effectiveness Review Board
	- SRM on 97-224	11/4/9 7	(same)
<b>Information Papers Released to the Public</b>			
1.	SECY- 97-261	11/5/9 7	Weekly Information Report - Week Ending October 31, 1997
<b>Memoranda Released to the Public</b>			
1.	SRM on 97-193	11/5/9 7	Assistance to the U.S. Department of Energy's Fissile Materials Disposition Program
2.	M97101 5	11/18/ 97	Briefing on PRA Implementation Plan; Wednesday, October 15, 1997

#### **Commission Correspondence Released to the Public**

1. Letter to Dr. John H. Gibbons, The White House Office of Science and Technology, dtd 11/14/97 provides a report on classified research projects involving human subjects
2. Letter to the President of the United States dtd 11/14/97 concerns the proposed Agreement for Cooperation with Kazakhstan
3. Letter to the Vice President of the United States dtd 11/13/97 provides the FY 1997 annual report on the Welfare-to-Work initiative

#### **Federal Register Notices Issued**

1. Advisory Committee on Reactor Safeguards; Subcommittee Meeting on Thermal-Hydraulic and Severe-Accident Phenomena; Notice of Meeting on December 9 and 10, 1997



2. Advisory Committee on Reactor Safeguards; Subcommittee Meeting on Thermal-Hydraulic and Severe-Accident Phenomena; Notice of Meeting on December 11 and 12, 1997

NOVEMBER 21, 1997  
ENCLOSURE O

Region I  
Items of Interest  
Week Ending November 21, 1997

Pilgrim

On November 21, 1997, NRC staff met with Boston Edison Company in King of Prussia, Pennsylvania, for a predecisional Enforcement Conference. The apparent violations which are discussed in Inspection Report No. 50-293/97-05, include failure to maintain the licensing and design basis, improper design control, inadequate and/or untimely corrective actions, and not reporting conditions outside the design basis.

Indian Point Units 2 and 3

On November 19-20, 1997, the NRC Executive Director for Operations; Associate Director for Projects, NRR; Director, Project Directorate, I-1, NRR; EDO Regional Coordinator; Region I Administrator; Director, Division of Reactor Projects; Deputy Director, Division of Reactor Safety; and Region I staff from Division of Reactor Projects met with officials from the New York Power Authority and Consolidated Edison at the sites to tour the plants and hold discussions with plant personnel.

Region II  
Items of Interest  
Week Ending November 21, 1997

Florida Power and Light Company - St. Lucie

On November 17, 1997, the Regional Administrator accompanied Chairman Jackson to Florida Power & Light Company's (FP&L) St. Lucie nuclear power plant. Chairman Jackson met at St. Lucie with the NRC's resident inspectors, held discussions with FP&L officials on current improvement initiatives at the plant, and took a working tour of the facility including areas where replacement of the Unit 1 steam generators is currently in progress. At the conclusion of the visit, Chairman Jackson held a news conference with members of the media and the public.

Tennessee Valley Authority - Sequoyah

Representatives from the Tennessee Valley Authority met with the NRC on November 19, 1997, for a predecisional enforcement conference to discuss the causal factors that led to an inoperable vital battery bus at Sequoyah in July 1997. The licensee presented its analysis of the safety significance and its completed and planned corrective action to preclude recurrence.

BWX Technologies

On November 20, 1997, representatives from BWX Technologies were in the Regional Office to attend a predecisional enforcement conference. The meeting was closed to the public due to the potential for discussion of classified information. The issues discussed involved exceeding the nuclear criticality safety limits for an accountability transfer cart on July 26, 1997. The licensee presented an analysis of the safety significance to the incident and its completed and planned corrective actions to preclude recurrence.

Florida Power Corporation - Crystal River

On November 18, 1997, Crystal River 3 reached a milestone in their station recovery plan and established a vacuum in the main condenser. This evolution was conducted using auxiliary steam from an adjacent fossil plant. On November 20, 1997, a pressurizer steam bubble was established using the pressurizer heaters. Temperature inside the pressurizer is being controlled at approximately 297 degrees Fahrenheit to maintain approximately 50 pounds per square inch gauge pressure in the reactor coolant system (RCS). The RCS temperature is 81 degrees Fahrenheit. Major problems or concerns were not identified during the two evolutions.

Work on the planned upgrades for the B Emergency Diesel Generator (EDG) was completed, and functional testing is in progress. Following return to operation of the B EDG, additional testing will be conducted on the power upgrades for the A EDG. Appendix R modifications, along with the EDG work, are the present critical path hardware activities.

NOVEMBER 21, 1997  
ENCLOSURE P

Region III  
Items of Interest  
Week Ending November 21, 1997

Special Inspection Team at Palisades Nuclear Power Plant --  
Public Exit

On November 19, 1997, NRC staff conducted a public exit meeting at the Palisades Nuclear Power Plant in Michigan to discuss the findings of a special inspection at the Palisades plant. The inspection covered an October 17, 1997, incident in which the entire reactor control rod drive system was taken out of service for maintenance of one control rod drive while the plant was in operation. The utility used a procedure that was intended for use while the plant was shut down. The use of the incorrect procedure was discovered by the utility during a review of the maintenance on October 20, 1997.

Duane Arnold Nuclear Power Plant

A public meeting between NRC staff and IES Utilities, Inc., was held at the Duane Arnold Nuclear Power Plant in Iowa on November 18, 1997, to discuss the plant's Systematic Assessment of Licensee Performance report. The plant received performance ratings of "superior" in operations, engineering, and plant support and "good" in maintenance.

Predecisional Enforcement Conference with Detroit Edison Company  
- Fermi Plant

On November 18, 1997, a Predecisional Enforcement Conference was conducted in the Region III Office in Lisle, Illinois, between management representatives of Detroit Edison Company and members of the NRC staff. The conference was held to discuss the utility's apparent inadequate evaluation of the safety implications when it changed procedures affecting the emergency equipment cooling system of the Fermi Nuclear Power Plant.

Predecisional Enforcement Conference with B.P. Chemicals, Inc.

On November 19, 1997, a Predecisional Enforcement Conference was conducted in the Region III Office in Lisle, Illinois, between management representatives of B.P. Chemicals and members of the NRC staff. The conference was held to discuss the company's apparent failure to oversee decommissioning activities adequately to assure compliance with federal regulations.

From 1964 to 1971, the company used depleted uranium as catalyst material for chemical production. Some of the resulting contaminated waste was placed into settling ponds and other

localized places on site. Remediation of the facility began in 1991. Final decommissioning activities now under way include permanent on-site disposal of the low level contaminated sludge and soil in specially designed cells.

Predecisional Enforcement Conference with the Michigan Department of Transportation

On November 21, 1997, a Predecisional Enforcement Conference was conducted in the Region III Office in Lisle, Illinois, between management representatives of the Michigan Department of Transportation and members of the NRC staff. The conference was held to discuss two incidents in September and October in which the Michigan Department of Transportation apparently failed to maintain control of two nuclear gauges.

Management Meeting with Commonwealth Edison Company - LaSalle

On November 21, 1997, a management meeting was conducted in the Region III Office in Lisle, Illinois, between management representatives from Commonwealth Edison Company and members of the NRC staff. The meeting discussion focused on the status of LaSalle Nuclear Power Station's restart action plan and the effectiveness of plant performance improvement actions.

Management Meeting with Commonwealth Edison Company - Byron Nuclear Power Station

On November 20, 1997, a management meeting was conducted in the Region III Office in Lisle, Illinois, between management representatives from Commonwealth Edison Company and members of the NRC staff. The meeting discussion focused on the current status of the Byron Nuclear Power Station.

Region IV  
Items of Interest  
Week Ending November 21, 1997

China Atomic Energy Authority Representatives Visit WNP-2

On November 19, 1997, the WNP-2 Senior Resident Inspector met with members of the China Atomic Energy Authority and the China National Nuclear Corporation as part of a visit to the WNP-2 facility. The members were invited to visit the facilities by the U.S. Department of Energy (DOE) under the auspices of mutual cooperation to enhance China's nuclear material and control accountability programs. WNP-2 was one of several diverse facilities toured by the delegation to review safeguards and material controls. The members of the delegation were as follows:

Dr. Chen Fangliu, Director of the National Office of Nuclear Material Control

Ms. Ma Yanqin, Deputy Director of the Office of Nuclear Material Control

Mr. Qiao Shengzhong, Deputy Director of the Bureau of Nuclear Fuel, China National Nuclear Corporation

Mr. Shen Lixin, Project Officer, China National Nuclear Corporation

Ms. Dong Chaojie, Project Officer, Bureau of Legislative Affairs, State Council, PRC

Accompanying the members were as follows:

James Busse, International Safeguards Division, DOE

Robert Sorenson, Senior Program Manager, International Safeguards, PNNL

Commissioner Diaz Visit to Palo Verde

On November 19, 1997, Commissioner Diaz visited the Palo Verde Nuclear Generating Station. He toured the site with Region IV Regional Administrator, met with the Palo Verde resident inspectors, discussed industry initiatives with licensee senior managers, and held a question and answer session with a representative group of licensee employees. In addition, Commissioner Diaz toured a neutrino experiment being conducted by Stanford, the California Institute of Technology, and the University of Alabama at the Palo Verde site.