

July 30, 1998

SECY 98-184

For: The Commissioners

From: James L. Blaha, Assistant for Operations, Office of
the EDO

/s/

Subject: WEEKLY INFORMATION REPORT - WEEK ENDING JULY 24,
1998

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*No input this week.

James L. Blaha
Assistant for Operations, OEDO

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Office of Nuclear Reactor Regulation
Items of Interest
Week Ending July 24, 1998

Status of Risk-Informed Regulatory Review Efforts Workshop

The NRC sponsored a workshop on July 22, 1998, at the Bethesda Marriott on the topic of Risk-Informed Regulation Efforts at the NRC. Sam Collins provided the opening remarks, and Gary Holahan, DSSA, and Tom King, RES, gave presentations. This workshop was an opportunity for industry and the public to provide NRC with feedback on how its risk-informed regulation efforts are perceived outside the NRC.

Risk-Informed Diesel Generator Reviews

On July 17, 1998, the Probabilistic Safety Assessment Branch (SPSB) provided its risk input to EELB completing its assessment of the risk associated with the proposed Oyster Creek emergency diesel generator (EDG) technical specification (TS) change request that would allow biennial inspection of EDGs at power instead of during shutdown. Oyster Creek had originally requested an allowed-outage time (AOT) extension from 7 to 14 days for on-line maintenance in 1996 but submitted a revised request on June 29, 1998, that the AOT extension be removed from the original request. SPSB used the existing information, provided by the licensee for the original application, to perform the risk evaluation. SPSB found that the proposed TS modification would have only a small quantitative impact on plant risk and the licensee has in place controls on equipment to reduce the likelihood of risk significant plant configurations during the proposed AOT. Therefore, SPSB concluded that PRA analysis supported the proposed on-line inspection of EDGs at power.

On July 17, 1998, SPSB conducted a conference call with the San Onofre licensee regarding the proposed EDG AOT extension from 3 to 14 days. Specifically, the call focused on the licensee's shutdown risk argument that performing EDG inspections at power would result in overall risk reduction. The licensee had provided quantitative results calculated using its shutdown PRA, which included the risk analyses from both internal and external events. The shutdown risk argument is believed to be a viable approach, but requires additional staff review. Before the call, a list of RAI questions were transmitted to the licensee for its review regarding modeling assumptions, data, quality assurance, etc. in its shutdown PRA. The main purpose of the call was to have a mutual understanding of the set of RAI questions between

the staff and the licensee staff. After some discussion, the licensee indicated that it would provide the information needed for staff review.

Risk-Informed Review of Monticello Extended Power Uprate Program

On June 8, 1998, the Probabilistic Safety Assessment Branch (SPSB) supported the final ACRS briefing on the staff's review of Monticello Extended Power Uprate program. The licensee's submittal included a section on the PRA performed for the bounding power level of 112%. SPSB's review concluded that, in general, the licensee's PRA for the uprated power level meets the criteria outlined in Reg. Guide 1.174.

Probabilistic Analysis Supports Relief From Inservice Inspection Requirements of BWR Circumferential RPV Welds

On June 9, 1998, SPSB co-presented the results of the staff SER on BWRVIP-05, "BWR Reactor Pressure Vessel Shell Weld Inspection Recommendations." The safety evaluation report (SER) included a probabilistic analysis which estimated the frequency of cold overpressurization of the vessel which was considered the most limiting transient and the resulting conditional failure probability of the vessel. Based on the results, the staff plans to issue a Generic Letter which permits licensees permanent relief from Inservice Inspection Requirements for volumetric examination of BWR circumferential RPV welds. ACRS plans to issue a letter prior to August 1, 1998, endorsing the staff's SER.

Three Mile Island Unit 1

GPU Nuclear, Inc. issued a press release Friday, July 17, 1998, that it had reached an "agreement in principle" to sell its Three Mile Island Unit 1 nuclear generating facility near Harrisburg, PA, to AmerGen Energy Company owned jointly by PECO Energy Company (51%) and British Energy (49%). There are several steps that need to be completed prior to finalization of the sale including a 90-day on-site "due diligence" review by British Energy and completion of all regulatory approvals. GPUN intends to submit its license application for transfer of ownership after the due diligence review is completed and would probably request a quick turnaround, but has anticipated that all regulatory approvals (including the licensing review) could take 12-24 months. This is the first nuclear plant sale in the United States and the first purchase of a nuclear plant by AmerGen. PECO's Limerick and Peach Bottom facilities are not part of AmerGen holdings and are not planned to be at this time. GPU's

TMI-2 and Oyster Creek facilities are not part of the sale. GPU indicated it is considering early shutdown of Oyster Creek in the fall 2000. The initial agreement sets aside \$100 million, \$23 million for the reactor and \$77 million over five years for the nuclear fuel. Additional future payments are possible. TMI-1 decommissioning costs will be pre-funded by GPU. Some local public interest in the sale is anticipated.

AmerGen is headquartered near Philadelphia, PA.

Lasalle Units 1 and 2

In preparation for restart of LaSalle Unit 1 on or about July 28, 1998, the NRC Restart Panel is in the process of completing items in its MC 0350 action plan. The NRC's Restart Readiness Inspection Team exited the site on July 21, 1998. The team found no significant issues that would preclude restart. The public restart meeting for LaSalle Unit 1 was held near the site on July 24, 1998. At this meeting the licensee presented its assessment of the corrective actions taken to address the problems that caused the shutdown in September 1996 and its readiness to restart. Following the public meeting, the NRC restart panel provided its recommendation to the Regional Administrator. The corrective action letter (CAL) was lifted on July 24, 1998. The NRC is developing a plan for round-the-clock site coverage during restart.

ComEd is in the process of forming a restart organization for Unit 2 (scheduled to restart approximately six months following Unit 1). Tony Dobbs, of ComEd's corporate office, has been named the Unit 2 Restart Manager. The organization will also include restart managers for Engineering, Maintenance, and Licensing.

Fermi Unit 2 - Manual Scram Because of a Main Turbine Control Valve Problem

On July 19, 1998, the licensee was operating the Fermi 2 plant at roughly 62 percent power for a scheduled control rod pattern adjustment and maintenance. At approximately 5:28 p.m., the operators noticed a disturbance in reactor pressure, reactor power, and main steam flow to one of the moisture separator/reheaters. Reactor power decreased to 50 percent, increased to 75 percent, and then decreased to 50 percent again. As power began to increase again, the operators manually scrambled the reactor, with all control rods inserting properly and all safety systems functioning as expected.

Based on the data collected during the event, the licensee

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determined that the event had been caused by a problem with the No. 4 turbine control valve (TCV). Although the exact nature of the problem could not be determined, it appeared that some movement of valve internal components had occurred independent of motion from the actuator. During stroke testing of the valve after the scram, the licensee found that the length of the stroke had increased. This is another indicator of a problem internal to the valve.

The licensee is scheduled to begin a refueling outage in early September. Because disassembly and repair of the TCV would be expected to take more than a week, the licensee plans to restart the unit and operate the main turbine with the No. 4 TCV closed. Power will initially be limited to roughly 75 percent in this configuration. However, more detailed analyses may allow operation at higher power levels. Region III and NRR staff have monitored the licensee's event evaluation and its plans for restart.

The plant entered Operational Condition 2 (Startup) at approximately 4:54 a.m. on July 23, 1998. The reactor should be critical by mid-morning, with turbine roll in the afternoon.

Management Changes

Mr. C. K. McCoy, Vice President of Vogtle Project, has been temporarily replaced by Mr. J. Beasley, the Vogtle Plant General Manager. A search for a permanent replacement is underway.

Office of Nuclear Material Safety and Safeguards
Items of Interest
Week Ending July 24, 1998

Participation in the Health Physics Society Annual Meeting

During the week of July 13-16, 1998, staff from the Division of Industrial and Medical Nuclear Safety participated in several committee meetings and workshops at the Health Physics Society (HPS) Annual meeting held in Minneapolis, Minnesota. On July 15, 1998, staff participated in the HPS Legislation and Regulation Committee to provide insights into Nuclear Regulatory Commission activities such as decommissioning, comments on the Environmental Protection Agency's Federal Guidance Report 13, and the revisions to 10 CFR Part 35. In addition, staff also chaired the HPS Publications Committee meeting on July 13, 1998. A major topic of discussion was the need for journal articles on practical health physics issues.

International Safeguards Subgroup Meeting

On July 20, 1998, a meeting of the Interagency Subgroup on International Atomic Energy Agency (IAEA) Safeguards in the U.S. was held at the Nuclear Regulatory Commission (NRC) Headquarters. The Subgroup is chaired by the Section Chief of the International Safeguards Section and includes representatives of the Departments of State, Energy, and Defense, and the Arms Control and Disarmament Agency. Discussions focused primarily on preparations for a July 22, 1998, bilateral coordination meeting with IAEA in Washington, DC. Topics addressed in the coordination meeting included the stabilization of excess weapons material, return of spent research reactor fuel to the U.S., downblending of high-enriched uranium deemed excess to the weapons program, and the status of the downblending verification exercise at an NRC regulated facility.

U.S. Enrichment Corporation/Atomic Vapor Laser Isotope Separation Nuclear Criticality Safety Validation Report

On July 21, 1998, staff from the Division of Fuel Cycle Safety and Safeguards met with staff from the U.S. Enrichment Corporation (USEC) and their contractor to discuss the nuclear criticality safety validation report, submitted to support the licensing of the Atomic Vapor Laser Isotope Separation (AVLIS) technology, and responses to staff questions on the report. USEC is requesting the Nuclear Regulatory Commission to license the AVLIS facility up to a uranium enrichment of 10 percent. The nuclear criticality safety validation report was submitted to

support this enrichment level. USEC intends to submit an AVLIS license application in February 1999; however, non-site-specific information is being submitted for staff review before the application is submitted. The criticality safety validation report and this meeting were part of the ongoing pre-license application activities.

Hanford Tank Waste Remediation Systems Contract Signed

On July 21, 1998, the Department of Energy (DOE) announced that it reached an agreement with British Nuclear Fuels Limited (BNFL), Inc., to continue development of a facility to treat and immobilize radioactive waste at the Hanford site in Washington. A new contract will be awarded pending a 30-day Congressional review period. Under the new contract, work will continue under two parts: (1) an initial 24-month period during which BNFL, Inc., will continue facility design to an approximately 30% completion level and develop a firm fixed-price for the remainder of the project; and (2) a construction and a minimum-order-quantity period of 10 years at a fixed cost targeted at about \$6.9 billion. During this period, approximately 10 percent of the Hanford Tank waste by mass is expected to be treated, and the facility will be constructed to have a 30-year design life and be expandable with operations beginning no earlier than 2005. DOE has suggested an eventual transition of nuclear safety regulation of the privatized facilities for processing the Hanford high-level tank waste to the Nuclear Regulatory Commission (NRC); and, to smooth the transition, NRC has been advising DOE in its safety reviews during the initial development phase of the project under an interagency memorandum of understanding.

Staff Visit to Maybell, Colorado, Uranium Recovery Site

On July 21, 1998, Division of Waste Management (DWM) staff visited the Maybell, Colorado, uranium mill site being reclaimed under Title I of the Uranium Mill Tailings Radiation Control Act. At the request of the Department of Energy (DOE), DWM staff met with DOE contractors regarding potential problems with rock size and gradations placed for long-term erosion control. Staff toured the site and evaluated the rock placement in several areas of the diversion channels and on the slopes of the disposal cell. After assessing the problem and reviewing construction records, staff suggested several methods that could provide acceptable solutions to the problem. DOE intends to provide formal documentation of the proposed changes in the near future.

Department of Energy's Proposed Revision to the Burnup Credit Topical Report

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On July 22, 1998, a meeting was held between representatives of the Spent Fuel Project Office and the Department of Energy (DOE) to discuss DOE's proposed revision to the burnup credit topical report and the staff's resources and scheduler availability. DOE plans to submit Revision 2 to the burnup credit topical report by the end of fiscal year 1998. However, the report will propose a reduced scope of applicability that is expected to result in a reduction in staff review time. DOE indicated that the revised topical report will not address burnup credit issues generically, as was done in the past, but instead, will be developed such that each cask vendor can address pertinent issues relative to their specific cask designs.

Safety Evaluation Report for the HI-STAR 100 Storage Cask

The staff of the Spent Fuel Project Office has begun to prepare a draft Safety Evaluation Report/Certificate of Compliance (SER/CoC) for the Holtec International (Holtec) HI-STAR 100 storage cask. After the completion of draft SER/CoC, a rulemaking will begin to allow the HI-STAR 100 to be included in 10 CFR 72.214, "List of Approved Spent Fuel Storage Casks." A final SER would not be issued by the staff until after the rulemaking process is completed. The HI-STAR 100 cask system is planned for use by several utilities, including Commonwealth Edison, which plans to replace wet storage at its Dresden Unit I with dry cask storage. Other utilities are waiting for the staff's approval of Holtec's HI-STORM concrete overpack which is compatible with the all-metal HI-STAR 100. Southern Nuclear's Hatch Plant and the proposed Private Fuel Storage Facility on the reservation of the Skull Valley Band of Goshute Indians plan to use the HI-STORM system. The staff expects to complete the HI-STAR 100 storage draft SER/CoC by the end of September 1998.

Office for Analysis and Evaluation of Operational Data
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Advance Notice of Proposed Rulemaking

An advance notice of proposed rulemaking (ANPR) to modify reporting requirements for nuclear power reactors contained in 10 CFR 50.72 and 50.73 was published on July 23, 1998. The ANPR requests public comments on whether the NRC should proceed with rulemaking to modify the event reporting requirements and, if so, the general nature of the modifications that should be made. Several concrete proposals are provided for comment. These include proposals that would: (1) clarify the requirements for reporting of design issues and (2) limit such reporting to design issues that exceed a specified level of significance. They also include a proposal that would extend the required reporting times, consistent with the NRC's needs for prompt reporting. Finally, the notice requests public comments on other reactor reporting requirements, beyond 10 CFR 50.72 and 50.73, that could be simplified and/or made less burdensome and more risk-informed. These subjects will be discussed at two public meetings: (1) a public meeting on the ANPR at NRC Headquarters on August 21, 1998, and (2) a public workshop on the role of industry in nuclear regulation in Rosemont, Illinois on September 1, 1998. Written comments on the ANPR are due September 21, 1998.

Full-participation Exercise with Prairie Island Plant

On Wednesday, July 22, 1998, members of AEOD staff participated with other headquarters response personnel and Region III Base and Site Teams in a full-participation exercise with the Prairie Island Nuclear Generating Plant. The States of Wisconsin and Minnesota, the Prairie Island Indian Community, and other Federal agencies also participated in the exercise.

PRELIMINARY NOTIFICATIONS

1. PNO-III-98-036, Allied-Signal, Inc., RELEASE OF URANIUM HEXAFLUORIDE ON PLANT SITE
2. PNO-IV-98-032, Earth Consultants, Inc., DAMAGED TROXLER MOISTURE DENSITY GAUGE (SOURCE NOT DAMAGED)
3. PNO-I-98-031, St. Francis Medical Center, LOSS OF TWO IODINE 125 SEEDS

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Office of Administration
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FIXIT Program

On July 20, 1998, ADM initiated use of a new version of the agency-wide computer program - FIXIT - which is intended to be used by employees to identify building or facilities services related problems. The new FIXIT provides a better opportunity for employees to identify the nature of the problem or complaint and allows ADM staff and the facilities contractor to more efficiently respond to building and facility problems.

Respiratory Protection and Controls To Restrict Internal Exposures (Part 20)

A proposed rule that would amend the regulations governing the use of respiratory protection and other controls to restrict internal exposure to radioactive material was published in the Federal Register on July 17, 1998 (63 FR 38511). The proposed rule would make the regulations more consistent with the philosophy of controlling the sum of internal and external radiation exposure, reflect current on radiation protection, and make the requirements less prescriptive without reducing worker protection. The comment period for this proposed rule closes September 30, 1998.

Adjustment of the Maximum Retrospective Deferred Premium (Part 140)

A final rule amending the regulations that specify the required amount of financial protection for certain reactors was published in the Federal Register on July 21, 1998 (63 FR 39015). The final rule increases the maximum secondary retrospective deferred premium for liability insurance coverage in the event of nuclear accidents at licensed, operating, commercial power plants with a rated capacity of 100,000 kW or more. This amendment is an inflation adjustment required by the Price-Anderson Amendments Act of 1988. The final rule becomes effective August 20, 1998.

Minor Corrections, Clarifying Changes, and a Minor Policy Change (parts 20,32,35, 36 and 39)

A final rule that makes a number of minor changes to the Commission's standards for protection against radiation and conforming changes to other regulations was published in the Federal Register on July 23, 1998 (63 FR 39477). Among other

correcting or clarifying amendments, the final rule raises the monitoring criteria for minors and declared pregnant women from 0.05 rem to 0.1 rem. The final rule becomes effective August 24, 1998.

Expand Applicability of Regulations to Holders of, and Applicants for, Certificates of Compliance and heir Contractors and Subcontractors (Part 72)

A proposed rule that would amend the regulations concerning licensing requirements for the independent storage of spent nuclear fuel and high-level radioactive waste was published in the Federal Register on July 23, 1998 (63 FR 39526). The proposed rule would extend the applicability of these regulations those who hold or apply for a Certificate of Compliance as well as their contractors and subcontractors. The proposed rule would allow the Commission to take enforcement action against these persons when legally binding requirements are violated. The comment period for this action closes October 6, 1998.

Reporting Requirements for Nuclear Power Reactors (Part 50)

An advance notice of proposed rulemaking announcing that the Commission is considering amending the event reporting requirements applicable to nuclear power reactors was published in the Federal Register on July 23, 1998 (63 FR 39522). The advance notice invites public comment on issues relating to amendments to the reporting requirements that would be based on importance to risk, extend reporting times in a manner consistent with the need for prompt NRC action, and reduce or eliminate the reporting burden associated with events of little or no safety significance. The comment period for this action closes September 21, 1998.

Chief Information Officer
Items of Interest
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Freedom of Information and Privacy Act Requests Received during
the 5-Day Period of July 17, 1998 - July 23, 1998:

Named individual, radiation exposure records. (FOIA/PA-98-289)

Contract, Sterling Software, statement of work, modifications
affecting burdened labor rates. (FOIA/PA-98-290)

Contract, Dynamic Decisions, Order #DR980085, modifications and
statement of work. (FOIA/PA-98-291)

Diablo Canyon, invoices related to 1986 payments under 10 CFR
Part 170 fee schedule. (FOIA/PA-98-292)

Unresolved and generic safety issues status listing by plant.
(FOIA/PA-98-293)

Washington Hospital Center, Washington, DC, license #08-03604-03
application dated 8/26/94. (FOIA/PA-98-294)

Shelwell, 1998 Commission Paper, vote sheets, and staff
requirements memorandums. (FOIA/PA-98-296)

International Uranium (USA) Corp., mill tailings at White Mesa
Mill site in UT and Ashland 2 site in Tonawanda, NY.
(FOIA/PA-98-296)

MLTS database on diskette. (FOIA/PA-98-297)

B&W Co., Parks Township, PA; Directors Decision (DD95-12), all
data analyses, and calculations used as basis for decision.
(FOIA/PA-98-298)

Self, transcripts regarding procedural deviation incident at
Diablo Canyon, OI report 4-1998-007. (FOIA/PA-98-299)

Office of Human Resources
Items of Interest
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Arrivals		
KADE, Daryl M.	Branch Chief	OCFO
PASCARELLI, Robert J.	Reactor Operations Engineer	NRR
Departures		
DONOVAN-BECKFORD, Sherma K.	Program Assistant	ASLBP

Office of Public Affairs
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Media Interest

New Scientist magazine is printing a story on Peter Crane's petition on potassium iodide.

Press Releases	
Headquarters:	
98-122	NRC Considers Changes to Regulations for Spent Fuel Storage Cask Designers
98-123	NRC Increases Insurance Assessment for Nuclear Power Plants Based on Inflation
98-124	NRC to Hold Informal Public Hearing on Petition to Revoke, Modify or Suspend Operating License for D.C. Cook Plant
98-125	NRC to Hold Meetings on Proposed Revisions to Medical Policy Statement, Regulations
98-126	NRC Requests Comments on Draft Document Assessing Use of Potassium Iodide During a Severe Reactor Accident
Regions:	
I-98-86	NRC Assesses Performance of Oyster Creek Nuclear Power Plant, With 'Superior' Rating for Maintenance, 'Good' for Other Areas
I-98-87	NRC Schedules Meeting on Indian Point 3 Apparent Violations
I-98-88	Note to Editors: Enforcement Conference on Windham Memorial Hospital
III-98-46	NRC Staff Finds Sufficient Improvements for Restart of Unit 1 of the LaSalle Nuclear Power Station
IV-98-31	Neil O'Keefe Assigned Senior Resident Inspector for NRC at South Texas Project
IV-98-32	NRC Holds Public Meeting to Discuss Plans for Shipping Trojan Reactor Vessel to Hanford

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Office of International Programs
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Visit of Adolph Birkhofer

Adolph Birkhofer, Managing Director of the Gesellschaft für Anlagen und Reaktorsicherheit (GRS) of Germany, visited NRC on July 23 to sign an implementing Agreement in probabilistic risk assessment and safety research. The EDO signed the Agreement for NRC.

Additionally, Dr. Birkhofer met with Commissioner Diaz, Deputy EDO William Travers, and Director of Research Ashok Thadani. Much of the discussion was devoted to the need and manner of increasing our research cooperation during a time of declining budgets. Other topics that were addressed included increasing use of risk informed regulation and NRC efforts to revise the reactor regulatory process.

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Office of the Secretary
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Documents Released to Public	Date	Subject
Decision Documents		
1. SRM on SECY-98-128	7/21/98	Proposed Rule: Revision of 10 CFR Part 35, Medical Use of Byproduct Material
Commission Voting Record on 98-128	7/21/98	(same)
2. Manual	7/22/98	Internal Commission Procedures
3. SECY-98-165	7/2/98	Proposed Revision to 10 CFR 50.65(a)(3) to Require Licensees to Perform Safety Assessments
Negative Consent Documents		
1. SECY-98-131	6/8/98	Status of the Integration Plan for Closure of Severe Accident Issues and the Status of Severe Accident Research
SRM on 98-131	7/20/98	(same)
2. SECY-98-157	6/30/98	Compliance of Great Bay Power Corporation with the NRC's Decommissioning Funding Assurance Requirements
SRM on 98-157	7/20/98	(same)
3. SECY-98-158	6/30/98	Rulemaking Plan for Implementation of Revised Source Term at Operating Reactors
Information Papers		
1. SECY-98-159	7/2/98	Litigation Report - 1998 - 3
2. SECY-98-163	7/1/98	Annual Report on the Status of Prematurely Shut Down Plants

3. SECY-98-166	7/6/98	Summary of Activities Related to Generic Safety Issues
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Memoranda		
1. M980721	7/23/98	Staff Requirements: Meeting with Advisory Committee on Nuclear Waste, Tuesday, July 21, 1998

Commission Correspondence

1. Letter to Representative Edolphus "Ed" Towns dated July 14, 1998 concerns the NRC's handling of proprietary claims (Incoming of June 3, 1998 also released).
2. Letter to Representatives Edward Markey and John Dingell dated July 14, 1998 provides an interim response concerning the impact of budget reductions on the NRC's ability to carry out its mission.
3. Letter To Representative Gene Green concerns proposed revisions to Part 35 (Incoming of June 12, 1998 also released).
4. Letter to Representative Tom Allen dated July 14, 1998 concerns the proposed revisions to Part 35 (Incoming of June 21, 1998 also released).
5. Letter to John T. Conway, U.S. Defense Nuclear Facilities Safety Board, dated July 14, 1998 concerns the external regulation of DOE facilities (Incoming of April 9, 1998 also released).
6. Letter to Senator John Chafee dated July 16, 1998 provides views regarding the resolution of differences between EPA and NRC with respect to cleanup of radioactive contamination.
7. Letter to Pierre Barber dated July 16, 1998 concerns an invitation to participate on a task force (Incoming of June 23, 1998 also released).
8. Letter to David Lochbaum, Union of Concerned Scientists, dated July 16, 1998 concerns the NRC's regulatory oversight of the Maine Yankee Atomic Power Station (Incoming of April 10, 1998 also released).
9. Letter to Erle Nye, Texas Utilities Company, dated July 16, 1998 concerns the existing process for license renewal (Incoming of May 29, 1998 also released).

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10. Letter to Secretary of Labor Alexis Herman dated July 16, 1998 provides a revised MOU between NRC and DOE.

Federal Register Notices Issued

1. Designation of Presiding Officer for the Atlas Corporation (Request for Material License Amendment).

Region I
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ANS Utility/NRC Licensing Manager's Workshop

Richard V. Crlenjak, Deputy Director, Division of Reactor Projects, represented Region I at the ANS Utility/NRC Licensing Manager's Workshop conducted in Bethesda, Maryland on July 20-21, 1998

Indian Point 3

A predecisional enforcement conference was held July 24 at Region I with New York Power Authority, the operator of Indian Point 3. The conference focused on an inadequate design modification to electrical equipment which could have caused loss of the emergency diesel generators during certain postulated events. The adequacy of the licensee's corrective actions in response to a degraded valve that supplies cooling to the reactor coolant pump thermal barrier, and the lack of evaluation of the impact of the degraded valve on operators and equipment, also were discussed.

Peach Bottom 2 and 3

Regional Administrator Hubert J. Miller, accompanied by Charles W. Hehl, Director, DRP; and Clifford Anderson, Chief, DRP Branch 4; Robert Capra, PD I-2 Project Directorate and Mohan Thadani, Project Manager, NRR; visited the Peach Bottom site on July 22, 1998. The site visit included a plant tour and meetings with licensee managers and staff.

Region I Staff attends July meeting of Maine Yankee Community Advisory Panel

The Chief of the Region I Decommissioning and Laboratory Branch, DNMS, attended the July 16, 1998 meeting of the Community Advisory Panel (CAP) for decommissioning the Maine Yankee plant. The panel Chairman, State Senator Marge KilKelly (Wiscasset) requested the NRC provide comment on and answer questions

regarding NRC inspection oversight at Maine Yankee. One primary concern was that the NRC no longer has a full-time resident inspector at the site, which has been shut down since December 1996. Inspections at the facility will now be conducted by Regional and Headquarters specialists as needed, based on the scope of decommissioning activities and events at the facility. Beginning in October 1998, Region I, at the request of the committee, will provide quarterly updates to the CAP concerning regulatory matters regarding the facility.

Meeting with New Jersey Department of Environmental Protection

On July 17, 1998, Dr. Jill Lipoti and other members of New Jersey Department of Environmental Protection's (NJDEP) Radiation Protection Programs met with NRC Region I representatives, including the Deputy Regional Administrator, the Division Directors of DRP and DNMS, several Branch Chiefs and others to exchange information and concerns on topics of mutual interest. Topics discussed included Salem restart process, potential near-term permanent shutdown and decommissioning of Oyster Creek, NJDEP's work toward developing decommissioning criteria more restrictive than NRC's, and radiation incidents, particularly those involving NRC generally licensed materials.

Region II
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Virginia Power Company - North Anna

The North Anna Power Station conducted its biennial emergency preparedness exercise on July 21, 1998. The Commonwealth of Virginia and local counties near the site participated in the exercise. The onsite portion of the exercise was evaluated by the NRC. The Region II State Liaison Officer also participated in the exercise as a member of the FEMA Regional Assistance Committee.

On July 22, licensee personnel at Virginia Power's North Anna Power Station transported their first loaded spent fuel cask (Transnuclear TN-32) from the plants protected area to the recently licensed Independent Spent Fuel Storage Installation (ISFSI). The move was monitored by the Senior Resident Inspector and a region-based radiation specialist inspector. Approximately eight to ten media personnel were also present and observed portions of the move. The transportation of the cask was uneventful.

Region III
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State of Minnesota Plans To Become an Agreement State

On July 10, 1998, Arne H. Carlson, Governor of the State of Minnesota, sent NRC Chairman Shirley Ann Jackson a letter stating that the Minnesota Legislature has authorized the state to start preparations for becoming an Agreement State. Minnesota anticipates becoming an Agreement State in 2002.

Management Meeting with Commonwealth Edison Company - La Salle

On July 24, 1998, a management meeting was conducted at the Brookfield Township Hall, Seneca, Illinois, between management representatives from Commonwealth Edison Company and members of the NRC staff. At the meeting, the NRC staff and utility officials reviewed the readiness of Unit 1 at the LaSalle Nuclear Power Station to resume operations. The two LaSalle units have been shut down since September 1996 for equipment modification and maintenance, staff retraining, and other performance improvement activities. The utility is currently planning to begin Unit 1 startup activities at the end of this month. The Unit 2 startup schedule is about six months after Unit 1. The NRC conducted a special team inspection at the plant to assess the readiness of Unit 1 to resume operations.

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CONGRESSIONAL HEARING SCHEDULE, <u>NO. 24</u>					
OCA Contact	DATE & PLACE	TIME	WITNESS	SUBJECT	COMMITTEE
Combs	07/27/98 2123 RHOB	2:00	TBA	Uranium Mill Tailings Cleanup	Reps. Schaefer/Hall Energy and Power Commerce
Madden	07/30/98 406 DSOB	9:00	Commission	Oversight of Nuclear Regulatory Commission Activities	Senators Inhofe/Graham Clean Air, Wetlands, Private Property And Nuclear Safety Environment and Public Works

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