

February 19, 1997

SECY 97-043

For: The Commissioners

From: James L. Blaha, Assistant for Operations, Office of
the EDO

Subject: WEEKLY INFORMATION REPORT - WEEK ENDING FEBRUARY
14, 1997

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*No input this week.

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Office of Nuclear Reactor Regulation
Items of Interest
Week Ending February 14, 1997

Risk-Informed, Application-Specific Regulatory Guides Forwarded To ACRS

On February 7, 1997, the staff forwarded DRAFT versions of the risk-informed, application-specific regulatory guidance documents to the Advisory Committee on Reactor Safeguards. These regulatory guidance documents cover risk-informed approaches for graded quality assurance, technical specifications, and inservice testing of pumps and valves.

The staff is continuing to work with the Office of the General Counsel (OGC) to identify and resolve any legal concerns and is scheduled to brief the ACRS PRA Subcommittee on February 20 and 21, 1997. Once OGC indicates that they have no legal objection to the risk-informed, application-specific guidance documents, the documents will be forwarded to the Committee to Review Generic Requirements (CRGR).

CRGR is tentatively scheduled to review the risk-informed, application-specific guidance documents on March 11, 1997. The staff is working with CRGR staff to accelerate this review schedule to help ensure that the risk-informed guidance documents will be forwarded to the Commission by March 31, 1997.

Prairie Island Unit 2 -- NDE Indications in B&W Alloy 600 Rolled Tube Plugs

During the ongoing inservice inspections of the Prairie Island Unit 2 steam generators, the licensee identified 3 B&W alloy 600 rolled tube plugs with circumferentially-oriented eddy current indications in the lower hard roll transition. Due to the prior history of cracking in other types of alloy 600 tube plugs, the licensee is planning to remove the plugs for a root cause determination. To our knowledge this is the first plug inspection using a plus point mag-bias probe. Due to the preliminary nature of the information, it is unclear whether or not there are safety significant generic implications. The staff will continue discussions with the licensee as more information becomes available and consider actions as appropriate. On Thursday, February 13, the licensee provided to the staff an outline of their plans regarding this type of plug.

Kewaunee Nuclear Power Plant -- Status of Steam Generator Tube Repairs

About 1900 sleeved tubes in the two steam generators at the Kewaunee Nuclear Plant were repaired using laser weld technology. These repairs were performed on tubes whose sleeves had already

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been in service several years but had developed flaw indications in the lower hard roll transition area of the hybrid expansion joint. Although the NRC has approved laser weld technology for installation of new sleeves, the review of the laser weld repair was on-going at the time leakage was observed during hydrostatic testing.

The laser weld repair process involves (in order):

1. laser welding the sleeve to the tube,
2. performing ultrasonic testing to verify proper weld fusion between the tube and sleeve,
3. post-weld heat treatment, and
4. eddy current testing (ECT) to identify volumetric indications or weld cracks and to establish baseline ECT.

At the time of secondary side reflooding, some tubes in the A steam generator were still scheduled for repairs, such as plugging and tube end rerolling in previously plugged tubes. Therefore, some of the tubes in the A steam generator were expected to leak. About 92 of the 1900 repaired tubes (84 in SG A and 8 in SG B) showed indications of leakage when the generators were filled with water.

As noted above, ultrasonic testing is performed prior to post-weld heat treatment. The licensee is repeating ultrasonic testing of the tubes and has discovered changes in some of the UT signal characteristics. This could indicate that the post-weld heat treatment resulted in weld damage.

The licensee is conducting a root cause evaluation that will address structural, metallurgical, and non-destructive examination issues. The licensee is preparing to remove five tubes from steam generator A for destructive examination. Tube removal is tentatively scheduled for February 13. These tubes will consist of leaking as well as non-leaking repaired tubes. The plant remains in refueling shutdown during this evolution. No restart date is scheduled. Specialists from the Region III office, in coordination with NRR staff members, have been closely monitoring the licensee's activities onsite.

At this time the staff does not believe that the problems encountered in the laser weld repair are generic to the installation of new laser welded sleeves. It was known from previous experience, including experience at a foreign plant, that unsatisfactory welds may result when the surfaces cannot be cleaned. However, for new laser welded sleeves the surfaces are cleaned prior to welding. Also, there is extensive operating

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experience with laser welded sleeves, including approximately 17,000 such sleeves at Maine Yankee. None of these laser welds sleeves have been reported to have leaked. The staff will reevaluate this conclusion, as necessary, based on the Kewaunee root cause evaluation.

Salem Nuclear Generating Station Units 1 and 2 -- Major Milestones

Following are the major milestones, as listed in the NRC Restart Action Plan, and the current schedule of activities to the readiness of Salem, Unit 2 to restart:

NRC approve Salem restart plan - Completed February 13, 1996.

NRC perform inspections to evaluate the adequacy of Salem restart activities - underway, being completed to support each mode.

Management meeting with PSE&G to discuss design/licensing bases and restart issues scheduled March 6, 1997.

PSE&G present the results of its readiness determination in a management meeting with the NRC open to public observation - mid April 1997 (RATI entrance).

NRC conduct a meeting with the public to solicit their input - early April 1997.

NRC conduct a Restart Assessment Team Inspection (RATI) - mid April 1997.

NRC internally reviews results of RATI and makes appropriate recommendation regarding Salem unit restart to NRC Regional Administrator - late April 1997.

If results are acceptable, NRC will approve PSE&G release from the Confirmatory Action Letter. If further PSE&G action is necessary, communicate it to PSE&G and amend NRC plans, as necessary - mid May 1997.

Oyster Creek

On January 31, 1997, the Atomic Safety and Licensing Board (ASLB), in the proceeding regarding GPUN's amendment application to install the shield plug in the dry storage canister, issued a Memorandum and Order (LBP-97-1) granting Summary Disposition in favor of the licensee. Since the decision is a favorable one, no further action by the Staff is necessary.

Oconee Nuclear Station Units 1, 2, and 3

Unit 2 power has been increased to 100 percent and has been operating normally for 9 days.

After being shut down for 131 days, criticality on Unit 1 was established at 5:28 a.m. on February 12, 1997. The licensee has established a program to continuously monitor shaft displacement (vibration) instruments on the 1A1 Reactor Coolant Pump following a step increase from < 10 mils to > 40 mils. This increase occurred while the pump was operating alone and the reactor coolant system stable at approximately 225 psig and 180 °F. Balancing the pump shaft on February 8, 1997, reduced the vibration to approximately 26 mils, but a second balance adjustment attempt was not successful in reducing the level, which has remained in the 24-26 mil range. After performing an analysis, the limit has been increased from 26 to 28 mils. As long as the vibration is below 28 mils, startup of Unit 1 will continue to 70 percent power. However, if level increases to 28 mils, the pump will be shut down and unit operation will continue at reduced power until a maintenance outage can be scheduled. From past experience, it is expected that the vibration will decrease as reactor power is increased.

Unit 3 is in day 132 of its end-of-cycle 16 refueling outage. Mode 5 has been established. Critical path to startup continues with modifications that are being made to upgrade the reheater drain system, changes that were completed to Units 1 and 2 prior to startup. These plant modifications appear to have been successful in resolving the water hammer problem that resulted in the shutdown of the three units.

Commonwealth Edison Company -- ComEd Meeting on the Use of Mixed Oxide Fuels

On January 28, 1997, the staff met with Commonwealth Edison Company (ComEd), at ComEd's request to discuss its program to dispose of weapons grade plutonium through use as a mixed oxide (MOX) fuel.

The Department of Energy (DOE), as the lead agency, is developing a program for the disposition of weapons grade plutonium. In its Record of Decision issued in January 1997, DOE proposed a "dual track" approach comprised of immobilizing, through vitrification, some of the excess plutonium, and burning the remainder in commercial nuclear reactors as MOX. ComEd in concert with Duke Power, British Nuclear Fuels (BNFL) and Cogema has assembled a project team to develop a plan for the possible use of MOX. The project for Plutonium Excess Converted to Electricity (Project PEACE) will consider burning MOX at several of their sites, specifically Byron, Braidwood, LaSalle, Catawba, and McGuire.

If the Project PEACE team is selected as a responder to a DOE request for proposals, scheduled to be issued in 1997, the team

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proposes to evaluate the performance of MOX through the use of a lead test assembly program (LTA). Based on successful performance of the LTA, MOX fuel would be integrated as part of the normal core reloads. Using fuel fabricated in Europe, the earliest LTA placement in a core would occur in late 1999; and assuming a successful test, inclusion of MOX in a reload would occur in late 2003.

The licensees' team stressed that, while weapons grade plutonium has not been used in commercial reactors, there is a considerable amount of data on the use of MOX in Europe obtained from the utilization of reprocessed fuels. MOX was also used as fuel in several U.S. reactors in the 1960s and 1970s. However, the team acknowledged that the use of MOX as proposed in Project PEACE will require a license amendment supported by the appropriate safety and accident analyses.

During the meeting both the staff and the Project PEACE team acknowledged that there are a number of non-technical issues (e.g., political and contractual) that will have to be solved before the project can proceed. The staff provided questions and comments for those technical areas that it believes will have to be addressed in any request for a license amendment. At the conclusion of the meeting it was mutually agreed that as progress occurs on the use of MOX, further meetings between the staff and the Project PEACE team will be beneficial.

Stop Work Orders Issued

On February 7, 1997, Commonwealth Edison issued stop work orders at all six sites for onsite engineering work that uses corporate Nuclear Engineering Procedures. An NRC inspector had identified that some of these procedures at LaSalle had not been reviewed by the on-site review group. Based on this finding, ComEd issued Tech Alert No. 97-04, "Site Approval and Use of NEPs," which reinforced the requirements that apply to the approval and implementation of all safety-related procedures used on site. Although the NEPs had been reviewed and approved by corporate engineering, some NEPs had not received on-site reviews as required by Technical Specifications. The Tech Alert stated that all procedures used to perform work must be reviewed to assure that the required safety evaluations (50.59 or screening), Technical Reviews, and/or Onsite Reviews have been performed. The Tech Alert also directed that work should not be resumed until all Technical Specification required reviews have been completed. According to licensee personnel, no significant discrepancies have been identified during the procedure reviews to date, and the stop work has not caused a major impact on site schedules.

Zion Units 1 and 2 -- Unit 1 Refueling Outage Delay and Extended Low Power Operations

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The licensee has announced that the Unit 1 refueling outage, which was scheduled to start about April 5, 1997, will be delayed until sometime in the late summer. The unit is currently at about 40% power, where it will remain until May 3, 1997, at which time it will be shutdown for a 21 day maintenance outage. During this outage, some of the major work the licensee expects to accomplish includes the installation of the rod position indicator modification, performance of instrumentation surveillance tests, resolution of the containment coating issue, repair of certain steam leaks, and the elimination of some operator work arounds. The core has about 67 EFPD remaining, which translates into about 167.5 days of operation at 40% power. If the maintenance outage lasts 21 days, the 167.5 EFPD will last until about August 20, 1997.

The staff informed the licensee that it should consider the effect on plant systems and components of long term operation at reduced power. The resident staff will monitor the licensee's preparations for the maintenance outage.

Office of Nuclear Material Safety and Safeguards
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Department of Energy/Nuclear Regulatory Commission Meeting on
Level of Design Detail in a Potential Geologic Repository License
Application

On February 6, 1997, Division of Waste Management staff met with the Department of Energy (DOE) and its contractors to discuss DOE plans for developing design information on the geologic repository. The DOE design process for the geologic repository is based on a single evolving design to support the Viability Assessment, the Environmental Impact Statement, and a License Application. The design work would be prioritized to provide the necessary level of detail and the products required to support the major programmatic milestones over the next five years. The DOE also discussed the "binning" strategy it will employ in developing the necessary design information. The process being proposed by DOE relies on the extent to which there is existing regulatory guidance.

Overall, the process initially classifies geologic repository structures, systems, and components (SSCs) according to whether they are important to safety and waste isolation, and then provides further refinement to this classification based on existing regulatory or licensing precedent. DOE intends to focus on those SSCs important to safety and/or waste isolation for which there is little or no regulatory or licensing precedent. Design information on such SSCs will be developed to the highest level of detail in any potential License Application.

Savannah River Site - Residual Wastes

On January 27-28, 1997, Division of Waste Management staff met with Department of Energy (DOE) representatives and their contractors at the Savannah River Site in South Carolina. The meeting was in response to a December 20, 1996, DOE letter requesting Nuclear Regulatory Commission staff review of DOE's methodology for classifying the residual wastes remaining in 51 high-level waste (HLW) tanks, once the tanks have been emptied. The DOE general methodology uses criteria, specified in a March 1993 letter from NRC to DOE, to classify the residual wastes as "incidental." Since the March 1993 criteria were originally developed for the removal of wastes from HLW tanks, NRC staff will evaluate these criteria to determine their applicability to in-situ disposal, and will provide the results of the evaluation for Commission review.

State of Nevada Letters on Nuclear Regulatory Commission High-Level Waste Program

In late January 1997, the Nuclear Regulatory Commission received two letters from the State of Nevada concerning NRC high-level waste (HLW) disposal activities. NRC responses to both letters are being prepared. In the first letter, dated January 24, 1997, the State requested that NRC reconsider recent staffing realignments in the HLW program. The second letter, dated January 27, 1997, raised concerns that NRC staff is attempting to resolve technical issues with DOE during prelicensing consultation. The State is concerned that a "resolved issue" will be excluded from the licensing process or require a higher threshold level before the issue can be reopened for further consideration. The NRC staff definition of "resolved" in the HLW prelicensing consultation program simply means that the staff has no further comments or questions at this time, and in no manner precludes the issue from being reviewed again at the time of licensing. The label "resolved" during prelicensing also does not preclude DOE from using an alternative approach in the actual license application.

Director's Decision Issued Denying Petition to Envirocare

By letter dated February 7, 1997, the Director, Office of Nuclear Material Safety and Safeguards issued a Director's Decision denying a petition filed by Dr. Thomas B. Cochran, Director of Nuclear Programs, Natural Resources Defense Council (NRDC), under 10 CFR 2.206 of the Commission's regulations. In Dr. Cochran's petition, filed in a letter dated January 8, 1997, he requested that the Nuclear Regulatory Commission take immediate action to revoke all licenses held by Envirocare of Utah, Inc. NRDC asserts, as a basis for this request, that a December 28, 1996, article in *The Salt Lake Tribune* reported that between 1987 and 1995, Envirocare's President made secret cash payments to the Director of the Utah Division of Radiation Control. NRC review of the petition concluded that no substantial health and safety issues had been raised regarding Envirocare that would require the immediate action requested by the NRDC. The NRDC did not provide any information in support of its requests of which the NRC was not already aware. NRC inspections of the Envirocare facility have not revealed the existence of extraordinary circumstances that would warrant immediate suspension of the Envirocare license. Further, NRC staff review of the technical basis for its issuance of the license and subsequent amendments found no evidence of the existence of any substantial health or safety issue that would justify the actions requested by the NRDC. The NRC will monitor the investigations and actions being conducted by the State of Utah. If NRC receives any specific information indicating there is a public health or safety concern as a result of these actions or from any other source, including NRC ongoing Agreement State oversight activities (Utah is an Agreement State), NRC will evaluate the information and take such action as is warranted.

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Meeting with Nuclear Energy Institute on the Transport of Radioactive Materials

On February 7, 1997, Spent Fuel Project Office staff and Department of Transportation (DOT) staff attended a meeting at the Nuclear Energy Institute (NEI) in Washington, DC, to discuss the status of action items from an NEI-sponsored workshop held March 28, 1996, on regulations for transporting low-specific activity (LSA) material and surface contaminated objects (SCOs). As a result of the workshop, the Nuclear Regulatory Commission and DOT are jointly developing guidance for shipping LSA material and SCOs, as well as guidance for shipping large components resulting from decommissioning, such as pumps and steam generators. (The NRC issued Generic Letter [GL 96-07] on December 6, 1996, for shipping discarded steam generators.) NEI also informed DOT and NRC of their intent to petition DOT to amend its regulations to delay implementation of DOT's radiation protection program for carriers and mandatory use of International System of Measurements (SI) units on shipping documents.

Meeting with Holtec International

On February 10, 1997, Spent Fuel Project Office staff met with representatives of Holtec International. Also attending the meeting were representatives of Commonwealth Edison, Northern States Power Company, Private Fuel Storage, American Electric Power, Lawrence Livermore National Laboratories, and the Ixex Group. At the meeting, Holtec described to the staff the changes made in Revision 4 of the Safety Analysis Report (SAR) and Topical Safety Analysis Report (TSAR) for Holtec's HISTAR 100 dual purpose cask system. Extensive revisions to the SAR and TSAR resulted from the staff's request for additional information on Revision 3. The thermal, structural, and criticality areas were subjects of in-depth discussion.

Idaho National Engineering and Environmental Laboratory Site Visit and Nuclear Regulatory Commission Participation in Public Meeting

On February 6, 1997, Spent Fuel Project Office (SFPO) staff was given a site familiarization tour of the Idaho National Engineering and Environmental Laboratory (INEEL) (previously known as the Idaho National Engineering Laboratory) by the Department of Energy (DOE). The staff visited the proposed location for a dry cask independent spent fuel storage installation (ISFSI) that will house the core debris from Three Mile Island Unit 2 (TMI-2), as well as the wet storage facility where the fuel is presently stored.

On the evening of February 6, the SFPO staff participated in a public meeting in Idaho Falls, Idaho, to discuss the TMI-2 ISFSI.

DOE staff provided an overview of the project, and NRC staff discussed NRC policies and licensing procedures. Approximately 30 members of the public attended the meeting. Several attendees requested that NRC establish a local public document room in Idaho Falls for this project. On February 7, 1997, SFPO staff members met with representatives of the State of Idaho's INEEL Oversight Group. Since the State is interested in the licensing of the TMI-2 ISFSI, discussion centered on NRC policies for State participation in NRC licensing actions.

Canadian Observation of a Nuclear Regulatory Commission Safeguards Inspection

On February 3-5, 1997, Regulatory and International Safeguards Branch staff accompanied an official from Canada's "Atomic Energy Control Board" (AECB) on an NRC Regional inspection of the Turkey Point Nuclear Power Plant in Florida. The purpose was to observe how specialized NRC performance-based evaluations augment the NRC Safeguards Regulatory Program. Future plans are for the AECB official to return to the U.S. to observe an "Operational Safeguards Response Evaluation" later this year. These visits are part of a U.S.-Canadian Bilateral Agreement to strengthen the nuclear safeguards in our respective countries.

Meeting on the International Atomic Energy Agency's Safeguards Criteria

During the week of February 10, 1997, a Regulatory and International Safeguards Branch staff member participated in a workshop in Prague, Czechoslovakia, on the evolution of the International Atomic Energy Agency's (IAEA) safeguards criteria. The IAEA has undertaken an initiative to replace the currently-utilized 1991-95 criteria with a new set of criteria that will be more performance-based and will take into account current safeguards approaches and the phase-in of measures defined in Program 93+2. The Program 93+2 measures are intended to enhance the IAEA's capability to detect undeclared nuclear activities. The staff member's contributions included a presentation on the Nuclear Regulatory Commission's experience with prescriptive and performance-based safeguards regulations.

Extension of License Term for Materials Licensees

On February 6, 1997, a notice was published in the Federal Register (62 FR 5656) announcing the Commission's approval of an increase in license terms from the current 5-year period to a 10-year period for licenses issued pursuant to 10 CFR Parts 30, 40, and 70 (with the exception of licenses issued pursuant to Part 35). These new or renewed licenses will be issued for 10 years, unless the staff exercises the option to issue the license for a shorter term. Licenses can be issued for shorter terms, on a case-specific basis, in situations where the industry has not

had extensive experience in using, or the Nuclear Regulatory Commission has not had experience in regulating, the proposed use of the material, as well as in any other situation that warrants increased attention.

Public Meeting With Advanced Medical Systems, Inc.

On February 10, 1997, the Nuclear Regulatory Commission staff held a public meeting which included participants from the Office of Nuclear Material Safety and Safeguards, the Office of the General Counsel, Region III, and Advanced Medical Systems, Inc. (AMS), as well as observers from the Northeast Ohio Regional Sewer District (NEORS) and the City of Cleveland Law Department. In the meeting, AMS provided an update on the status of litigation with NEORS, the discharge of water from the London Road Facility, its license renewal application, its financial state, and their twelve-task Building Recovery Project (BRP). Having recently disposed of 40,000 curies of bulk and sealed sources of cobalt-60, AMS is now proceeding to: (1) dispose of its stored waste, (2) clean the basement areas and remediate most of the facility to unrestricted release status, and (3) initiate steps to reconnect with the NEORS. AMS needs to complete its Decommissioning Funding Plan and Financial Assurance Instrument for approval of its license renewal application. AMS also discussed other action plans and commitments under the BRP.

Office of Nuclear Regulatory Research
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RES Support for NRR Regional Inservice Testing Workshops

During the weeks of January 27-31 and February 3-7, Joel Page of RES/DET/GSIB provided technical support as a panel member to NRR/EMMEB for Regional Workshops addressing Inservice Testing of Pumps and Valves. These workshops were held in all four regions and consisted of presentations by NRC panel members, breakout sessions where individual panel members helped licensees identify technical areas of concern, and a general panel session where all the accumulated questions were addressed. The workshops were well received and attending licensees complimented both the quality and format of the workshops, and the quality of the NRC teams and panels.

Joint Technical Committee Meeting for International Project "Analogue Studies in the Alligator Rivers Region" and Research Information Workshop

RES and its USGS contractor hosted the third Joint Technical Committee Meeting (JTC) for the international project "Analogue Studies in the Alligator Rivers Region" (ASARR) on January 21-24, 1997. ASARR is an international cooperative project under OECD/NEA, sited at the Koongarra Ore Body in Northern Australia, to study the transport behavior of uranium.

Significant advances were reported on extending laboratory work on sorption of U(VI) to naturally found substrates. Experimental measurements of sorption were conducted under varying chemical conditions. Several highly sophisticated techniques were employed to characterize the sorption sites and surfaces on both artificial and natural mineral substrates.

The RES workshop was scheduled to follow and complement the ASARR JTC meeting. Additional staff from PNL and SNL attended the workshop along with the participants in the two days of ASARR technical meetings. The workshop objective was to assess the progress on understanding sorption mechanisms and the applicability of the mechanistic models to assess radionuclide transport in complex environments. SNL reported on complementary work on sorption mechanisms on mono and bivalent species on simple clays. PNL discussed the role of organic complexants and microparticulates in enhancing radionuclide migration, the effects of chelating agents on radionuclide mobility, determination of solubilities of radionuclides in SDMP slags and LLW, and the geochemistry of concrete buffered systems. RES staff reported on work being conducted using facilities and

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equipment at The Johns Hopkins University on the characterization
of decommissioning slags.

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Office for Analysis and Evaluation of Operational Data
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Future Electrical Grid Reliability and Stability

On Thursday, February 13, 1997, representatives of the Reactor Analysis Branch of AEOD and the Electrical Engineering Branch of NRR met with representatives of the Offices of Policy and General Council of FERC to discuss concerns about future electrical grid reliability and stability. There are several activities currently underway which address reliability including efforts by the North American Electrical Reliability Council, Edison Electric Institute, Interconnected Operations Service Group, Western Interstate Energy Board, and the Department of Energy. Future transmission companies will be able to recover ancillary costs including those associated with maintaining grid stability.

Annual Meeting with the Ukrainian Regulatory Agency

During the annual meeting with the Ukrainian regulatory agency, IRD staff met with a senior delegation to review the status and plans of the cooperative effort to develop an emergency response system in that country. The effort is one of several being performed and funded under the Lisbon Nuclear Safety Initiative.

PRELIMINARY NOTIFICATIONS (PNs)

- a. PNO-I-97-010, Department of the Army (Picatinny Arsenal), MISSING 7.62 MM DEPLETED URANIUM (DU) AMMUNITION.
- b. PNO-III-97-007A, Department of the Army, (Rock Island IL), MISSING OR LOST DEVICES AT ARMY FACILITIES.
- c. PNO-III-97-008, Consumers Power Co (Palisades 1), WORKER INJURED.
- d. PNO-IV-97-008, Kansas University Medical Center, THERMOLUMINESCENT DOSIMETER EXPOSURE.
- e. PNO-IV-97-009, U.S. Air Force (Davis Monthan Air Force Base), LOSS OF GENERALLY LICENSED DEVICE AND DISCOVERY OF LICENSED MATERIAL IN AN UNRESTRICTED AREA (EVENT NUMBERS: 31741/31742).
- f. PNO-IV-97-010, Sheffield Steel (Sand Springs, OK), NUCLEAR GAUGE FOUND AT SCRAP YARD.

Office of Administration
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Emergency Telephones

Installation of new emergency telephones have been completed for the OWFN and TWFN elevators. The new phones will automatically dial out to the NRC's central alarm station (which is manned 24 hours a day) simply by lifting the receiver when there is a problem. These new phones also will allow personnel to call back to speak with the occupants of the specific elevator experiencing a problem. The old telephones did not provide this call back feature.

Chief Information Officer
Items of Interest
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Computer Disaster Recovery Plan

Discussion between NRC and the National Institutes of Health (NIH) has ensued to ensure that NRC applications and data are protected and moved to a preplanned recovery site if a disaster occurs in the NIH computer center. A memo of understanding is being prepared by the Office of Information Resources Management.

Computer Security Awareness

The 1996-1997 annual Computer Security Awareness training for all headquarters personnel was held on January 8, 13, 15, and 21. During the first two days, four 90-minute presentations were provided by Ray Semko from the Department of Energy. The last four sessions were 45-minutes in length and presented by NRC staff. Based on attendance card submittals, 1,830 headquarters staff and contractors went through the training. The make-up session and a special session for Commissioner Dicus' office were scheduled for February 1997.

Office of Information Resources Management will provide Computer Security Awareness training for most of Region IV and half of Walnut Creek's staff in Arlington, TX, during the week of February 18, 1997. This visit will also include a stop at NRC's Las Vegas, Nevada, field office for the same purpose and to provide additional virus detection education.

FOIA Requests Received During the Week Ending February 13, 1997

OIG cases 96-25B and 97-10B. (FOIA/PA-97-0036)

OIG case 95-241 related to Ward Valley. (FOIA/PA-97-0037)

OIG case related to Millstone's full core fuel offload.
(FOIA/PA-97-0038)

Draft NUREG/CR-6412 comments, "Aging & Loss of Coolant Accident Testing of Electrical Connections." (FOIA/PA-97-0039)

OI reports 4-95-055 and 4-95-072. (FOIA/PA-97-0040)

Communications since 1985 with three AEC terminated licensees.
(FOIA/PA-97-0041)

OI report on 6/95 contamination incident at NIH.
(FOIA/PA-97-0042)

Ekotek Superfund Site in Utah. (FOIA/PA-97-0043)

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E.I. Dupont's by-product distribution listing under License # 20-00320-14E. (FOIA/PA-97-0044)

Review of licensee's practices for scheduled on-line maintenance of safety equipment. (FOIA/PA-97-0045)

Millstone refueling practices. (FOIA/PA-97-0046)

OIG report 94-092I related to improper dose calculations at San Onofre. (FOIA/PA-97-0047)

Tri-Med citizen's petition. (FOIA/PA-97-0048)

Six-month summaries of drug and alcohol testing programs for 1995 and 1996. (FOIA/PA-97-0050)

Office of Personnel
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Interactive Voice Response (IVR) System to Allow Employees to
Access Personnel Records Under Final Phase of Development

The Office of Personnel is in the final database design and test phase for an IVR system that will allow employees to check data from their personnel record over the telephone and have reports faxed to them at home or at the office. Initial implementation of the telephone portion of the IVR is planned for the end of March 1997. Point of contact for this project is Bob Loach. He can be reached via e-mail at REL1 or by phone at 415-7534.

NRC Vacancy Announcements to be Available Soon on the NRC Home
Page

The Office of Personnel is in negotiation with the Office of Information Resources Management on the publishing of vacancy announcements on the NRC home page. The vacancy will be available on the web as soon as the vacancy opens. It is anticipated that this will become operational by the end of February at the latest. Point of contact is Bob Loach.

Arrivals

CALLAN, Leonard	EXECUTIVE DIRECTOR FOR OPERATIONS*	OEDO
COLLINS, Samuel	DIRECTOR, NRR (PFT)**	NRR

* TRANSFERRING FROM RIV TO OEDO

** TRANSFERRING FROM RIV TO NRR

Departures

IHNEN, Kerry	RESIDENT INSPECTOR (PFT)	RIII
MOULTON, John	PROJECT MANAGER (PFT)	NRR
TEDROW, Jeffrey	SR RESIDENT INSPECTOR (PFT)	RIV

Retirements

None.

Office of Public Affairs
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Media Interest

Chairman Jackson was interviewed by Eric Pooley of Time Magazine for a follow-up story on the Millstone plants.

Press Releases

Headquarters:

- 97-021 Note to Editors: Denying a petition from Thomas B. Cochran, Natural Resources Defense Council, on Envirocare of Utah, Inc.
- 97-022 Note to Editors: ACRS Meeting
- 97-023 NRC Revises Regulations for Shipments of Uranium and Plutonium
- 97-024 NRC Revises Regulations for Uranium Enrichment Facilities Operated by USEC or its Successor
- 97-025 Note to Editors: ACRS Meeting
- 97-026 Note to Editors: ACNW Reports on low-level waste and comments on the strategic assessment

Regions:

- II-97-15 NRC Staff to Hold Predecisional Enforcement Conference With Georgia Power Company to Discuss Concerns at Vogtle
- III-97-11 NRC Predecisional Enforcement Conference With Detroit Edison Company on Apparent Violations of NRC Requirements at Fermi Plant
- III-97-12 NRC Predecisional Enforcement Conference on Apparent Violations of NRC Requirements at Quad Cities Station (**Rescheduled in III-97-16**)
- III-97-13 NRC Predecisional Enforcement Conference on Apparent Violations of NRC Requirements at Dresden Station
- III-97-15 NRC Staff Rates Dresden "Good" in Operations, Plant Support, "Acceptable" in Maintenance, Engineering in Latest Assessment

IV-97-10

NRC Fines United Nuclear \$100,000 for Failing to
Set Aside Funds for Decommissioning Church Rock
Uranium Mill

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ENCLOSURE M

Office of International Programs
Items of Interest
Week Ending February 14, 1997

Foreign Visitors -- OECD/NEA

On February 13, 1997, Mr. Luis E. Echavarri, Director-General of the Spanish Nuclear Industrial Forum met with Chairman Jackson, Commissioners Rogers and Diaz. The purpose of his visit was to discuss his candidacy for the OECD/NEA Director-General position and his view on the future of the NEA.

IAEA Vacancy Notices

The following notice from the International Atomic Energy Agency has been posted on NRC bulletin boards:

P-3 Unit Head, Documents Control Unit
97/005
Administration

P-3 Data Processing Network Analyst
97/006
Safeguards

P-4 Nuclear Engineer
97/007
Nuclear Energy

P-4 Thesaurus Specialist
97/008
Nuclear Energy

P-3 Biological Scientist
97/700
Research and Isotopes

Region II
Items of Interest
Week Ending February 14, 1997

Florida Power Corporation - Crystal River

The Director, Division of Reactor Safety, who is Chairman of the Restart Panel for Crystal River, represented Region II at a public meeting on February 12, 1997, with Florida Power Corporation concerning the status of restart activities. There was extensive media coverage at the meeting. The licensee provided substantive information on their process for determining the readiness of systems for operations.

On February 14, 1997, a closed predecisional enforcement conference on security matters was held with Florida Power Corporation in the Region II office concerning activities at the Crystal River Plant. The enforcement conference was closed because of the nature of the issues discussed.

Duke Power Company - Oconee

Following the September, 1996 Unit 2 reheater drainline rupture and subsequent shutdown of all three Oconee units, Unit 2 returned to service following feedwater heater modifications.

Unit 1 restarted on February 12 and is currently holding at 55 percent power due to a speed changer problem on the 1B Main Feedwater Pump. Due to elevated vibration on the 1A1 reactor coolant pump (RCP) (26 mils vs 11 mils nominal), restart of Unit 1 was delayed until an evaluation was performed to reflect a new 1A1 RCP alarm/trip set point (28 mils vs 20 mils). The 1A1 RCP vibration currently remains around 25 mils. If the pump must be tripped, the licensee plans to operate below 70 percent power with 3 RCPs until Unit 3 is on line. Unit 3 is currently scheduled to restart on February 25.

Florida Power and Light Company - St. Lucie

On February 13, 1997, the Regional Administrator as well as Region II and Headquarters staff met with the President, Nuclear Division, FP&L, and his staff at a public meeting on site at St. Lucie. The purpose of the meeting was to discuss performance improvement plans, self-assessment corrective actions, and the effectiveness of the licensee's employee concerns program. The discussion of the licensee's employee concerns program was in part, prompted by the NRC's observation of an increasing trend in the total number of allegations received by the NRC over the past several years. As a followup to the NRC's Annual Report on Allegations, the NRC staff committed in SECY 97-006 to conduct such a meeting with the licensee. The meeting was well attended by FP&L managers as well as local news media.

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ENCLOSURE N

Region III
Items of Interest
Week Ending February 14, 1997

Predecisional Enforcement Conference with Illinois Power Company
Individuals

On February 10, 1997, a predecisional enforcement conference was conducted in the Region III Office, Lisle, Illinois, between two managers from Illinois Power Company's Clinton Nuclear Power Station and members of the NRC staff. Discussion focused on the events associated with the September 5, 1996, reactor recirculation pump "B" seal leak.

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ENCLOSURE N

Region IV
Items of Interest
Week Ending February 14, 1997

Connell Limited Partnership and Tulsa Gamma Ray, Inc.

A predecisional enforcement conference was conducted in the Region IV office on February 10, 1997, with Connell Limited Partnership (CLP) and Tulsa Gamma Ray, Inc. (TGR). The conference is being conducted jointly with both licensees to discuss the findings of a reactive inspection conducted on December 9, 1996, through January 9, 1997. The inspection was initiated in response to notification provided by TGR that on November 20, 1996, one of TGR's radiographers who performed licensed activities at CLP's facility was involved in an incident which resulted in a radiation dose in excess of regulatory limits. Apparent violations relating to both licensees were identified.

With regard to CLP, the apparent violations identified include failures to: (1) perform a survey of a radiographic exposure device after each exposure; (2) secure a sealed source assembly in the shielded position each time the source was returned to that position; and (3) read and record the exposure reading on a pocket dosimeter at the end of the day. Collectively, the apparent violations have regulatory and safety significance because they represent failures to perform tasks intended to prevent unnecessary radiation exposures of radiography personnel and members of the public.

With regard to TGR, one apparent violation was identified involving the failure to limit the occupational dose received by an individual adult (TGR radiographer) to a total effective dose equivalent of 5 rems during calendar year 1996. Additionally, although the inspection revealed that the overexposure received by the TGR radiographer was caused, in part, by the failure of a CLP radiographer to perform several tasks, other contributing causes identified during the inspection indicate that the incident may have been prevented had the TGR radiographer been able to hear his alarming ratemeter while working in the conditions present at the CLP facility and he been more attentive as he prepared to begin work on November 20, 1996.

Commissioner Visits San Onofre Nuclear Generating Station

On February 12, 1997, Commissioner Dicus and her technical assistant toured the San Onofre Nuclear Generating Station facility, including Unit 1, the decommissioned unit. Following the tour, the Commissioner met with senior managers of the San Onofre facility.

Office of Congressional Affairs
 Items of Interest
 Week Ending February 14, 1997

CONGRESSIONAL HEARING SCHEDULE, No. 5					
OCA ASSIGN- MENT	DATE & PLACE	TIME	WITNESS	SUBJECT	COMMITTEE
Gerke	02/24/97 2154 RHOB	10:00	TBA	Year 200 Computer Problems	Reps. Horn/Maloney Government Mgmnt, Info & Technology Government Reform & Oversight
Combs	02/26/97 366 DSOB	9:30	Markup/Vote	S. 104, amend Nuclear Waste Policy Act; Vote on Nomination of Federico Pena as DOE Secretary	Senators Murkowski/Bumpers Energy & Natural Resources
Gerke	03/06/97 366 DSOB	TBA	TBA	Electricity Deregulation: Issues of Competition	Senators Murkowski/Bumpers Energy & Natural Resources
Gerke	03/13/97 366 DSOB	TBA	TBA	Role of Public Power in a Competitive Market	Senators Murkowski/Bumpers Energy & Natural Resources
Madden	03/12/97 2362 RHOB	10:00	DOE	Nuclear Waste Mgmnt & Disposal	Reps. McDade/Fazio Energy & Water Development Appropriations
Gerke	03/20/97 366 DSOB	TBA	TBA	Electricity Deregulation: Is Federal Legislation Necessary?	Senators Murkowski/Bumpers Energy & Natural Resources

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ENCLOSURE R

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