

July 9, 1999

SECY 99-180

For: The Commissioners  
From: James L. Blaha, Assistant for Operations, Office of the EDO  
Subject: WEEKLY INFORMATION REPORT - WEEK ENDING JULY 2, 1999

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\*No input this week.

James L. Blaha  
Assistant for Operations, OEDO

Contact:  
J. Shackelford, OEDO

Office of Nuclear Reactor Regulation  
Items of Interest  
Week Ending July 2, 1999

Deferment of Inspections for Weld Overlays as Required by Generic Letter 88-01 (EMEB)

The NRC staff has approved the Boiling Water Reactor Vessel and Internals Project (BWRVIP) request for a generic deferment of weld overlay inspections until March 2001 for several plants. The approval for the deferment is based on the successful inspection history for weld overlay repairs of IGSCC in BWR stainless steel susceptible piping, and other conditions specified in the approval.

BWR licensees that have a requirement for inspections of the subject weld overlays in their Technical Specifications (TS) will need to formally request a TS change allowing this deferral.

This generic deferral constitutes a significant reduction of burden to the licensee in reducing required inspections while maintaining adequate safety for the piping. The requesting licensees are expecting to save approximately \$6,000 and 1.5 rem dosage per weld not inspected. Since the industry will not be inspecting approximately 115 weld overlays, the total savings is approximately \$690,000 in direct inspection costs, \$1,725,000 in rem savings, and a dose reduction of 172.5 rem.

The staff is also pursuing a longer term change to the inspection requirements for weld overlays and BWR piping in general. Modifications to the current BWR piping inspections would be based on the extensive experience with intergranular stress corrosion cracking (IGSCC), and will be integrated with the ongoing development of risk-informed ISI programs.

Wisconsin Energy Corporation and WICOR Sign Merger Pact

Wisconsin Energy Corporation (WEC), the licensee for Point Beach, and WICOR, Inc. announced Monday, June 28, that they have entered into a definitive merger agreement which melds Wisconsin's largest electric and gas utilities into a single company. WICOR, Inc. is a Milwaukee-based, diversified holding company operating six subsidiaries in energy services and pump manufacturing. The subsidiaries are Wisconsin Gas, WICOR Energy, FieldTech, Sta-Rite Industries, SHURflo Pump Manufacturing Company, and Hypro Corporation. WEC is a holding company with subsidiaries in utility and non-utility businesses. Its principal subsidiaries are Wisconsin Electric, Edison Sault Electric, Wisvest, Minergy, and Wispark.

The merger is conditioned upon the approval of both companies' shareholders and upon customary regulatory approvals, including the Public Service Commission of Wisconsin. The companies anticipate that the transaction can be completed by the spring of 2000. The merger is not expected to impact WEC's nuclear interests. As reported previously, WEC is also engaged in a recently formed alliance of mid-western nuclear utility companies involving Wisconsin Electric, Northern States Power, Wisconsin Public Service, and Alliant-IES. The nuclear plants involved in this alliance are Monticello, Prairie Island, Point Beach, Kewaunee, and Duane Arnold.

Catawba Nuclear Station, Units 1 and 2, and McGuire Nuclear Station, Unit 2

By letter dated June 22, 1999, Duke Energy Corporation (Duke) requested an exemption from 10 CFR 54.17(c) for these 3 nuclear units. This regulation specifies that "an application for a renewed license may not be submitted to the Commission earlier than 20 years before the expiration of the operating license currently in effect."

Duke stated that it is determining "the feasibility of filing with the NRC in 2001 concurrent applications for renewal of the operating licenses" for the Catawba and McGuire units. By then, Catawba Units 1 and 2 will have 17 and 15 years, and McGuire Unit 2 will have 18 years operating experience. McGuire Unit 1 would have 20 years operating experience.

#### Decommissioning - Maine Yankee

During fuel inspections to determine the condition of spent fuel for acceptability in a dry cask storage system, Maine Yankee found a fuel element with a circumferential crack. The defect was located near the top of the element. The element was displaced by about the width of the cladding and no fuel pellets were lost. Ultrasonic testing performed on the assembly in 1991 did not detect any cracking. The licensee is discussing the defect with the fuel vendor. At this time the licensee does not believe the defect is reportable under Part 21, however, the applicability of Part 21 will be further evaluated as additional information is developed.

#### Decommissioning - Haddam Neck

On Friday, June 18, 1999, the licensee found low levels of contamination in the Spent Fuel Pool (SFP) spray cooling water. No contamination was anticipated, and the water had not been analyzed for contamination previously. The cooling spray flow was shut down for 4 days while the surge tanks were cleaned out. SFP cooling remained in service, but without the cooling spray the pool temperature increased from 100 deg F to 119 deg F. The cooling spray was returned to service on Tuesday, June 22. SFP temperature returned to its normal value of 100 deg F.

The SFP cooling system is a water to air radiator mounted on the roof on the spent fuel building. Heat rejection is increased by using a water spray on the radiator surface. Spray water that does not evaporate is collected and recycled. Evaporative losses are made up as needed from the primary water storage tank. The contamination level of the primary water is below detectable limits, however, the licensee believes that the evaporation and makeup of the cooling spray water concentrated the contamination enough to become detectable.

#### Transition Task Force Activities

The Inspection Program Branch conducted a training workshop on the Revised Oversight Process for the NRC staff in Irving, TX from June 21 through 25, 1999. The workshop provided detailed training on the Revised Oversight Process. The workshop was well attended.

As part of its efforts to communicate with external stakeholders regarding the Revised Oversight Process, the NRC conducted public meetings in the vicinity of eight of the nine pilot plants. The Harris Plant is scheduled for July 14, 1999, at 7:00 p.m., in Raleigh, NC. At these meetings, the NRC staff explained to members of the public the new risk informed oversight program being piloted at these plants and answered questions from the audience.

Office of Nuclear Material Safety and Safeguards  
Items of Interest  
Week Ending July 2, 1999

#### Summer Meeting of the Board on Radioactive Waste Management

On June 28, 1999, staff from the Nuclear Regulatory Commission (NRC) attended an open session of the National Academy of Sciences' (NAS) Board on Radioactive Waste Management. The session, entitled "Late-Breaking Developments in Radwaste Management," included a panel on Yucca Mountain Standards. NRC was represented on this panel by the Deputy Executive Director for Regulatory Programs and by the Director of the Division of Waste

Management. Representatives of both the Department of Energy (DOE) and the Environmental Protection Agency (EPA) noted that because a draft proposed EPA standards for Yucca Mountain was undergoing interagency review, neither agency would comment directly on the content of the draft proposal.

The Acting Director of DOE's Office of Civilian Radioactive Waste Management spoke briefly of DOE's general views about the importance of sound standards for Yucca Mountain (e.g., that they should not require a degree of proof beyond what science and engineering can provide and defend in a rigorous licensing proceeding). He went on to describe the evolution of DOE's repository design subsequent to the release of the Viability Assessment in December 1998. The Director of EPA's Radiation Protection Division summarized the process EPA has used to develop its draft proposal. He explained that, because EPA has the statutory authority to set the standards for Yucca Mountain, EPA exercised discretion in applying the guidance of the 1995 NAS report on the Technical Bases for Yucca Mountain Standards, and felt obligated to fully explain any differences in its proposal from the 40 CFR 191 standards applied to the Waste Isolation Pilot Plant (WIPP), as well as differences from the NAS recommendations. In effect, he stated, EPA viewed the NAS panel report as "super comments."

In its presentation to the Board, the NRC staff summarized NRC's proposed regulations in 10 CFR Part 63, explained why NRC had issued its proposal in advance of forthcoming EPA standards, and why NRC technical criteria differed from those applied by EPA in certifying WIPP. Questions from the Board members to NRC staff included perceived ambiguities in the Part 63 proposal with regard to defense-in-depth; the difference between NRC's concept of "reasonable assurance" and EPA's criterion of "reasonable expectation"; treatment of uncertainties in the Yucca Mountain performance assessment; and NRC's basis for selecting a 10,000-year compliance period. Following the panel on Yucca Mountain Standards, the Board was briefed on the status of civilian low-level waste disposal and on problems with in-tank precipitation of cesium at DOE's Savannah River Plant.

#### Meeting with Tennessee Valley Authority on Sequoyah Nuclear Plant Independent Spent Fuel Storage Installation

On June 29, 1999, staff from the Spent Fuel Project Office and the Office of Nuclear Reactor Regulation met with Tennessee Valley Authority (TVA) representatives to discuss their plans for an Independent Spent Fuel Storage Installation (ISFSI) at the Sequoyah Nuclear Plant. TVA plans to use the general license provisions of 10 CFR Part 72, Subpart K. TVA's objective is to have the ISFSI operational by Spring 2004.

#### Revision to the Certificates of Compliance for the AEA Technologies/QSA, Incorporated, (Amersham), Transport Packages

On June 22, 1999, the Spent Fuel Project Office issued an amendment to Certificates of Compliance Nos. 9027 and 9035, for the AEA (Amersham) Model Nos. 680 and 741 transport packages. The amendment was needed to provide additional time to allow the Certificate holder to retest and distribute a modified overpack for the cobalt radiography cameras. The revisions were necessary to meet operational needs. The radiography cameras must be shipped within an approved overpack by October 1, 1999.

#### Revision to Certificate of Compliance for the TRUPACT-II Transport Package

On June 25, 1999, the Spent Fuel Project Office issued an amendment to Certificate of Compliance No. 9218, for the TRUPACT-II package. TRUPACT-II is used by the Department of Energy to transport plutonium-contaminated wastes to the Waste Isolation Pilot Plant near Carlsbad, New Mexico. The amendment was requested to authorize shipment of additional

waste types, and to allow fabrication of additional TRUPACT-II packagings.

Meeting with BNFL Fuel Solutions

On June 24, 1999, Spent Fuel Project Office management and staff met with BNFL Fuel Solutions (BFS) to discuss the results of impact limiter testing for the TranStor Shipping Cask and to describe additional testing that will be performed in July 1999. In addition, BFS presented preliminary information on its technical review involving the application of coatings inside TranStor Storage Casks fabricated for Portland General Electric Company.

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Office of Nuclear Regulatory Research  
Items of Interest  
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Dry Cask Storage Characterization

The NRC, in cooperation with the Department of Energy (DOE) and the Electric Power Research Institute (EPRI), is participating in a cooperative research program on dry cask storage characterization. Under a demonstration program that has been under way since the mid-1980s, the DOE has managed quantities of commercial spent nuclear fuel in four dry storage casks at DOE's facilities at the Idaho National Engineering and Environmental Laboratory (INEEL) site. The intent of this cooperative program is to open casks, inspect the cask internals and stored fuel assemblies (or canisters), and withdraw several fuel rods for detailed evaluations. The results from this research will provide the technical basis to support license renewal of spent fuel dry cask storage systems.

On June 23-24, 1999, RES/DET staff conducted a technical review of this cooperatively funded research program at the INEEL. Representatives from NMSS/SFPO and the other funding participants: EPRI and DOE, also attended.

NMSS/SFPO is developing the technical basis for renewals of licenses for dry storage systems for spent nuclear fuel and high-level radioactive waste at independent spent-fuel storage installation (ISFSI) sites. This technical basis would also apply to the renewal of Certificates of Compliance for the casks to be used in the ISFSI sites. Both types of renewals could be for more than the present 20-year license terms. Verification of past performance of selected components of these systems is required as part of the technical basis for renewals.

Based on broad interest in this program, RES/DET will actively seek additional participation, both domestic and foreign.

Acceptability of Arrhenius Methodology

On June 29, 1999, RES and NRR staff met with Brookhaven National Laboratory's staff and its consultants to discuss the acceptability of Arrhenius methodology for analyzing loss-of-coolant accidents (LOCA) and Post-LOCA environments, in the context of environmental qualification of electric equipment. The nuclear industry recently submitted to the NRC staff a position paper for using Arrhenius to evaluate accident profiles. The position paper provided the technical bases for using this technique to evaluate differences between required accident profiles and actual test profiles, as well as post-LOCA duration. This technique is being used by the industry for (1) evaluations for power uprate amendments, (2) reevaluation of the qualified life of electric equipment after plant modifications and changes, and (3) evaluation of license renewal technical reports.

While the staff noted the limitations in the Arrhenius methodology, particularly in regard to activation energies, there was a consensus that the industry's approach is reasonable for using Arrhenius methodology to analyze LOCA and post-LOCA environments. Arrhenius methodology can be used to analyze varying temperature conditions based on the principle of cumulative damage (which implies that power uprate approaches are reasonable), provided the limitations inherent to Arrhenius are properly addressed. The consultants noted that there are many more uncertainties in the qualification process itself, such as the effects of oxidation and humidity, compared to the use of Arrhenius to analyze LOCA and post-LOCA environments.

DET staff will develop a technical position on the issues discussed at this meeting.

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Incident Response Operations  
Items of Interest  
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Preliminary Notifications

1. PNO-1-99-030, Maine Yankee Atomic Power Co., (Maine Yankee 1), DAMAGED RADIOACTIVE MATERIALS SHIPMENT
2. PNO-1-99-031, Pennsylvania Power & Light Co., (Susquehanna 1), SUSQUEHANNA UNIT 1 AUTOMATIC REACTOR SHUTDOWN - MEDIA INTEREST
3. PNO-III-99-033, Indiana Michigan Electric Co., (Cook 1 2), RESTART SCHEDULE ANNOUNCED
4. PNO-III-99-034, U.S. Enrichment Corporation (Portsmouth Gaseous Diffusion Plant), ALL CRITICALITY ACCIDENT ALARMS ADMINISTRATIVELY DECLARED INOPERABLE
5. PNO-IV-99-028, Columbia HCA Healthcare Corp., MEDICAL MISADMINISTRATIONS
6. PNO-IV-99-029, Wolf Creek Nuclear Oper. Corp., (Wolf Creek 1), WOLF CREEK GENERATING STATION PERSONNEL DOSE MISCALCULATION



Office of Administration  
Items of Interest  
Week Ending July 2, 1999

Release of Solid Materials at Licensed Facilities: Issues Paper, Scoping Process for Environmental Issues, and Notice of Public Meetings (Part 20)

A document announcing that the NRC is considering a rulemaking that would set specific requirements on releases of solid materials in order to establish a regulatory framework more consistent with NRC requirements on air and liquid releases was published in the Federal Register on June 30, 1999 (64 FR 35090). The NRC is seeking public input on the major issues associated with such a rulemaking including a scoping process related to the scope of environmental impacts. The NRC also announced that it is conducting four public meeting concerning these issues. The comment period closes November 15, 1999.

Chief Information Officer  
Items of Interest  
Week Ending July 2, 1999

Freedom of Information and Privacy Act Requests received during the 5-Day Period of June 25, 1999 through July 1, 1999:

Trojan Plant, warning sirens. (FOIA/PA 99-265)

NUREG-1092, reference on page II-33, 4/20/83 memo from L. Rouse to J. Norberg. (FOIA/PA 99-266)

Performance Appraisal Statistics, federal Workers' Compensation and Work-At-Home statistics at NRC since 1991. (FOIA/PA 99-267)

Memo, 5/5/94 E. Beckjord to W. Russell, "License Renewal Implications of Generic Safety Issues (GSIs) Prioritized and/or Resolved Between October 1990 and March 1994," as referenced in NUREG-0933, be placed in PDR. (FOIA/PA 99-268)

Availability Pay for OI Investigators, after 1994. (FOIA/PA 99-269)

Wellman Dynamics Corp., entire license file for No. STB-433 (transferred from Hills-McCanna Co. to Wellman in 1971 by AEC), Creston, IA. (FOIA/PA 99-270)

NRC licenses and release of radioactive materials into the sanitary sewer system, facilities located in four zip codes (18914, 18901, 18932, 18976). (FOIA/PA 99-271)

Contract, winning submittal for purchase of data disintegrator machine. (FOIA/PA 99-272)

Program Management Information: Draft Policy and Guidance Directive PG 83-2, Rev. 1, Suppl. 1, "Renewal of Materials Licenses" (SP-99-037). (FOIA/PA 99-273)

Materials licenses in specified Illinois zip codes and documents related to radioactive releases into the sanitary sewer system. (FOIA/PA 99-274)

Materials licenses in specific Massachusetts zip codes and documents related to radioactive releases into the sanitary sewer system. (FOIA/PA 99-275)

Materials licenses issued in specific Ohio zip codes since 3-23-98. (FOIA/PA 99-276)

Office of Human Resources  
 Items of Interest  
 Week Ending July 2, 1999

<b>Arrivals</b>		
AXELSON, Kimberly	SUMMER CLERICAL	RIII
ERICKSON, Randy	RADIATION SPECIALIST	RIV
POPE, Sabra	RADIATION SPECIALIST	RIV
<b>Retirements</b>		
BERNEY, Catherine	SECRETARY (OA)	NRR
BROWN, Bill	REGIONAL COUNSEL	RIV
GALANTE, Anthony	CHIEF INFORMATION OFFICER	OCIO
GINDES, Rosalie	SECRETARY (OA)	HR
JOHNSON, Virginia	SR COMMERCIAL VOUCHER EXAMINER	OCFO
MCPEEK, Eugene	REACTOR SECURITY SPECIALIST	NRR
STRYKER, William	TEAM LEADER	OIG
WARREN, Wanda	SECRETARY (OA)	RIV
<b>Departures</b>		
BOWMAN, Karen	PUBLICATIONS PRODUCTION ASST	OCIO
COMPTON, Eric	ENGINEERING AIDE	NMSS
JACKSON, Shirley Ann	CHAIRMAN	OCM

Office of Public Affairs  
Items of Interest  
Week Ending July 2, 1999

Media Interest:

There was press interest at headquarters in the results of licensee responses on Y2K preparedness.

<b>Press Releases</b>	
<b>Headquarters:</b>	
99-133	NRC Advisory Committee On Reactor Safeguards To Meet in Rockville, Maryland
99-134	NRC Staff To Meet With Public To Discuss Policies Related To The Release Of Information
99-135	Chairman Dicus Assumes Duties As Head Of NRC
S-99-20	"The Other Side Of Excellence," By Dr. Shirley Ann Jackson, 1999 Awards Ceremony
S-99-22	"NRC Regulation In The New Millennium," By Greta Joy Dicus Health Physics Society Plenary Session
<b>Regions:</b>	
I-99-58	Note To Editors: Meeting Location And Time Changed
III-99-38	NRC Staff Proposes \$55,000 Fine Against U.S. Enrichment Corp. For Failing To Declare "Alert" During Portsmouth Plant Fire
IV-99-27	NRC Proposes \$8,800 Fine For Anvil Corp.
IV-99-28	Russell L. Bywater, Jr. Assigned Senior Resident At Arkansas Nuclear

Office of International Programs  
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Visit of Taiwan Atomic Energy Council Chairman Dr. Ching-Piao Hu (July 2)

On Friday, Dr. Ching-Piao Hu, Chairman of the Taiwan Atomic Energy Council (AEC), visited Chairman Dicus, Commissioner McGaffigan and Commissioner Merrifield to update them on the current status of activities in the Taiwan nuclear power program. The Construction Permit for the Lungmen project has been issued and the two 1,350MWe ABWRs are scheduled for operation in 2003/2004. The storage of nuclear waste continues to pose challenge for the Taiwan Power Company. With the low level storage facilities on Orchid Island soon to be closed, a new location on an island in the China Sea is being carefully considered. While at NRC, Dr. Hu was accompanied from Taiwan by Mr. Ruey-Yan Wu, Deputy Director of the Fuel Cycle and Materials Administration.

Office of the Secretary  
Items of Interest  
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Document Released to Public	Date	Subject
<b>Decision Documents</b>		
1. SRM on SECY-99-100	6/28/99	Framework for Risk-Informed Regulation in the Office of Nuclear Material Safety and Safeguards
Commission Voting Record on 99-100	6/28/99	(same)
2. SRM on SECY-99-008	6/29/99	Rulemaking Plan, "Physical Security/Safeguards for Permanently Shutdown Power Reactors," for Amendments to 10 CFR Part 73
Commission Voting Record on 99-008	6/29/99	(same)
3. SRM on SECY-99-024	6/29/99	Recommendations of the Safeguards Performance Assessment Task Force
Commission Voting Record on 98-024	6/29/99	(same)
<b>Information Papers</b>		
1. SECY-99-154	6/16/99	Weekly Information Report - Week Ending June 11, 1999
<b>Memoranda</b>		
1. M990629	6/29/99	Staff Requirements - Affirmation Session, Tuesday, June 29, 1999: I.: COMSECY-99-019 - Proposed License to Export High Enriched Uranium (HEU) for Production of Medical Isotopes at the Canadian Maple Reactors (XSNM-03060)
2. M990615A/B	6/29/99	Staff Requirements - All Employees Meeting, 10:30 a.m. and 1:30 p.m., Tuesday, June 15, 1999

Commission Correspondence

1. Letter to Thomas Kress, dated June 18, 1999, offers a reappointment to a four-year term to the Advisory Committee on Reactor Safeguards
2. Letter to Senator Richard H. Bryan, dated June 18, 1999, concerns documents related to NRC efforts to streamline requirements for the license renewal of commercial nuclear

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power plants (incoming letter dated April 25, 1999, also released)

3. Letter to Representative Edward J. Markey, dated June 18, 1999, concerns emergency diesel generator reliability (incoming letter dated April 5, 1999, also released)
4. Letter to the President, dated June 18, 1999, concerns the 1999 Combined Federal Campaign (incoming letter dated May 12, 1999, also released)
5. Letter to B. John Garrick, Advisory Committee on Nuclear Waste (ACNW), dated June 18, 1999, concerns the ACNW 1999 Action Plan and Priority Issues (incoming letter dated January 22, 1999 also released)
6. Letter to Catherine W. LeBlanc, U.S. Department of Education, dated June 18, 1999, provides the Annual Federal Plan on Executive Agency Actions to Assist Historically Black Colleges and Universities
7. Letter to Congress, dated June 18, 1999, provides comments on the GAO report entitled "Nuclear Regulation - Strategy Needed to Regulate Safety Using Information on Risk" (incoming letter from GAO dated March 19, 1999, also released)

#### Federal Register Notices Issued

1. 10 CFR Parts 50 and 72; Reporting Requirements for Nuclear Power Reactors; Proposed Rule
2. Notice of establishment of the Pilot Program Evaluation Panel
3. 10 CFR Part 40; State of Colorado and Organization of Agreement States; Receipt of Petition for Rulemaking (PRM-40-27)
4. Applications for Licenses to Export Nuclear Material (DOE)

Region I  
Items of Interest  
Week Ending July 2, 1999

Millstone Unit 3

On June 29, 1999 at 2:49 p.m., Millstone Unit 3 was connected to the grid and commenced a normal power ascension to 100% power (it is currently at approximately 50% power). This event marked the first time in several years that two Millstone units were operational and producing power. Unit 2 was restarted on May 9, 1999, following an extended outage.

Susquehanna Steam Electric Station

Following a Susquehanna Unit 1 automatic shutdown at 2:08 on July 1, 1999, post shutdown indications showed that the outboard main steam isolation valve (MSIV) in the "C" steam line did not go fully closed. A preliminary visual inspection by PP&L indicated that the valve was approximately 1 inch off the fully closed seat. Susquehanna Unit 1 automatically shut down from 100 percent power due to a high neutron flux signal caused by the unexpected closing of the outboard main steam isolation valve (MSIV) in the "C" steam line. The reason for the valve closing is not yet known. The higher than normal steam flow in the other steam lines resulting from the closing of the "C" outboard MSIV caused the seven remaining MSIVs to close. All control rods fully inserted. The High Pressure Coolant Injection and the Reactor Core Isolation Cooling systems actuated on low reactor vessel water level as designed and successfully restored level. The safety relief valves and high pressure coolant injection system were used for pressure control until normal pressure control was restored. The unit is proceeding to a cold shutdown condition.

Nine Mile Point Unit 2

On June 30, 1999, Niagara Mohawk Power Corporation (NMPC) began a reactor startup of Nine Mile Point Unit 2 (NMP2). The unit had experienced a reactor scram on June 24 due to a malfunction in the feedwater master controller. Subsequent to the scram, the reactor core isolation cooling (RCIC) system was declared inoperable due to unexpected RCIC flow oscillations experienced while the operators were using the system to control vessel level with the RCIC flow controller in automatic. The operators were able to use RCIC to control vessel level with its flow controller in manual. Although manual operation resulted in the RCIC system being functional, the system was declared inoperable due to its inability to automatically control flow. Prior to the startup, NMPC personnel had taken corrective actions related to the feedwater and RCIC controllers, as well as offsite power line #5 which failed to transfer following the scram.

Predecisional Enforcement Conference - North Country Hospital

On July 1, 1999, Region I, DNMS conducted a Predecisional Enforcement Conference at North Country Hospital in Newport, VT. The Division Director and inspector were present at the facility during the conference while others from the Enforcement Staff, OGC and the DNMS Branch Chief participated by telephone conference. The primary issues discussed at the conference was the licensee's difficulties in controlling the use of I-131, the factors contributing to a misadministration of I-131, and their actions to prevent recurrence. This was a transcribed conference.

Exposure Event at Cooperheat-MQS

Cooperheat-MQS, Inc. is a large multi-site radiography licensee. On April 20, 1999, at their



Berlin, CT facility, the site Radiation Safety Officer mistakenly handled a radiography source assembly containing 52.6 curies of iridium-192 during a source exchange. The licensee estimated dose to his hand at 3.5 Rem. His dosimetry indicated a whole body dose of 110 - 170 millirem. On June 21 - 22, 1999, a reactive assist inspection was conducted by two Region I inspectors. Region I will forward the inspection findings to Region III for final disposition.

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Region II  
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South Carolina Electric and Gas Company - Refueling Outage Meeting - Summer

On July 1, representatives from South Carolina Electric and Gas presented an overview of the recently completed refueling outage at Summer. The licensee discussed overall facility performance, modification activities completed, emergent issues, and future initiatives.

Tennessee Valley Authority - Pilot Plant Public Meeting - Sequoyah

On June 28, Region II and NRR staff conducted a public meeting to discuss the new oversight process being piloted at Sequoyah. The meeting was attended by State and local officials, TVA staff, local media, and one member of the public.

On June 29, the Deputy Regional Administrator and Director, Division of Reactor Projects visited Sequoyah to solicit feedback on the pilot process from the resident inspectors and licensee management.

Tennessee Valley Authority - Management Changes

On June 28, John Scalice, TVA's Chief Nuclear Officer, announced that Browns Ferry Site Vice President, Karl Singer, will immediately begin serving a rotational assignment as Senior Vice President of Nuclear Operations. John Herron, Engineering & Support Services Manager at Sequoyah, will serve as Interim Site Vice President, Browns Ferry, during Singer's rotational assignment. Singer will have full responsibility for the overall management of TVA's nuclear plants and the Process Methods organization.

Region IV  
Items of Interest  
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San Onofre Nuclear Generating Station

Region IV management and staff personnel held a meeting on June 28, 1999, with licensee emergency preparedness representatives to discuss program activities. The meeting was open to attendance by members of the general public.

Meeting to Discuss the Results of the Culture Survey, Plant Status, License Renewal, and Various Other Topics

On June 30, 1999, the NRC conducted a public meeting with the Omaha Public Power District (OPPD) at the Fort Calhoun Station Administration Building, Fort Calhoun, Nebraska. Fort Calhoun managers shared information concerning the results of their recently completed culture survey, plant status, Fort Calhoun activities related to the new NRC oversight process, and license renewal. This meeting, conducted at the request of OPPD, was not related to any findings or insights associated with Fort Calhoun's participation in the pilot program for the new oversight process. It did, however, provide NRC and OPPD managers an opportunity to discuss recent performance issues and plans for future activities at the site.

RIV Public Workshop on NUREG-1021, Rev 8, Changes

On June 28 and 29, 1999, members of the Region IV and NRR staff met, in a public meeting, with Region IV licensees and other interested parties. The purpose of the workshop was to discuss the significant changes and clarifications to agency practice documented in Revision 8 of NUREG-1021, "Operator Licensing Examination Standards for power Reactors," to satisfy the recent revision to 10 CFR 55 reflecting the voluntary facility development of operator license examinations.

Diablo Canyon Presents Outage Self-Assessment

On July 1, 1999, Diablo Canyon management presented the results of their self-assessment of Refueling Outage 1R9. The licensee's discussion was led by the Senior Vice President and General Manager. The Region IV Regional Administrator was the senior NRC official present. The licensee presented their overall view of outage performance. They also described corrective actions for events that occurred during the outage, such as the sudden decrease in reactor coolant system level, the inadvertent loss of spent fuel pool cooling, and the higher than expected radiological exposure.

Office of Congressional Affairs  
 Items of Interest  
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CONGRESSIONAL HEARING SCHEDULE, NO. 26

OCA Contact	DATE & PLACE	TIME	WITNESS	SUBJECT	COMMITTEE
Portner	Week of 7/12	TBA	Markup	FY00 Energy & Water Appropriations	Reps. Packard/Visclosky Energy & Water Development Appropriations
Combs	TBA SD-406	TBA	Markup	S. 1090, Superfund Reauthorization	Senators Chafee/Baucus Environment & Public Works
Gerke	07/15/99 SH-216	9:30	TBA	Electricity Restructuring	Senators Murkowski/Bingaman Energy & Natural Resources
Portner	07/21/99 TBA	10:00	Commission	NRC Authorization & Legislative Proposals	Reps. Barton/Hall Energy & Power Commerce
Gerke	TBA	TBA	TBA	Y2K and Nuclear Issues	Senators Bennett/Dodd Special Committee on the Year 2000 Technology Problem

Note: Congress is on its Independence Day recess, returning on July 12th.