



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

September 25, 2026

Mr. Joseph Quinn
Vice President, Fleet Support
Tennessee Valley Authority
1101 Market Street, LP 4A-C
Chattanooga, TN 37402-2801

SUBJECT: BROWNS FERRY NUCLEAR PLANT, UNITS 1, 2, AND 3; SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2; WATTS BAR NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENT NOS. 346; 369, AND 329; 380 AND 375; AND 186 AND 91 REGARDING ADOPTION OF TSTF-596, REVISION 2; AND ADOPTION OF TSTF-374-A, REVISION 0, TSTF-118-A, REVISION 0, AND TSTF-600-A, REVISION 2, FOR WATTS BAR ONLY (EPID L-2026-LLA-0034)

Dear Mr. Quinn:

The U.S. Nuclear Regulatory Commission (NRC, the Commission) has issued the enclosed Amendment Nos. 346, 369, and 329 to Subsequent Renewed Facility Operating License Nos. DPR-33, DPR-52, and DPR-68 for the Browns Ferry Nuclear Plant, Units 1, 2, and 3, respectively; Amendment Nos. 380 and 375 to Renewed Facility Operating License Nos. DPR-77 and DPR-79, for the Sequoyah Nuclear Plant, Units 1 and 2, respectively; and Amendment Nos. 186 and 91 to Facility Operating License Nos. NPF-90 and NPF-96 for the Watts Bar Nuclear Plant, Units 1 and 2, respectively. These amendments are in response to your application dated February 17, 2026, as supplemented by letter dated July 21, 2026.

The amendments revise each unit's technical specifications (TSs) to adopt changes in accordance with Technical Specifications Task Force (TSTF) Traveler TSTF-596, Revision 2, "Expand the Applicability of the Surveillance Frequency Control Program (SFCP)." Additionally, the amendments revise the Watts Bar Nuclear Plant, Units 1 and 2, TSs, only, to adopt changes in accordance with TSTF-374, Revision 0, "Revision to TS 5.5.13 and associated TS Bases for Diesel Fuel Oil"; TSTF-118, Revision 0, "Administrative Controls Program Exceptions"; and TSTF-600, Revision 2, "Revise the Reactor Coolant System (RCS) Pressure Isolation Valve (PIV) Leakage Testing Frequency."

A copy of the NRC staff's Safety Evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Kimberly J. Green, Project Manager
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Docket Nos. 50-259, 50-260, 50-296,
50-327, 50-328, 50-390, and 50-391

Enclosures:

1. Amendment No. 346 to DPR-33
2. Amendment No. 369 to DPR-52
3. Amendment No. 329 to DPR-68
4. Amendment No. 380 to DPR-77
5. Amendment No. 375 to DPR-79
6. Amendment No. 186 to NPF-90
7. Amendment No. 91 to NPF-96
8. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-259

BROWNS FERRY NUCLEAR PLANT, UNIT 1

AMENDMENT TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE

Amendment No. 346
Subsequent Renewed License No. DPR-33

1. The Nuclear Regulatory Commission (NRC or the Commission) has found that:
 - A. The application for amendment by the Tennessee Valley Authority (TVA or the licensee) dated February 17, 2026, as supplemented by letter dated July 21, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR) Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Subsequent Renewed Facility Operating License No. DPR-33 is hereby amended, in part, to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 346, are hereby incorporated in this subsequent renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 6 months from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

MICHAEL
MAHONEY

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MICHAEL MAHONEY
Date: 2026.09.25
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Hipólito González, Chief
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to Subsequent Renewed
Facility Operating License No. DPR-33
and the Technical Specifications

Date of Issuance: September 25, 2026

ATTACHMENT TO LICENSE AMENDMENT NO. 346
TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-33
BROWNS FERRY NUCLEAR PLANT, UNIT 1
DOCKET NO. 50-259

Replace page 3 of Subsequent Renewed Facility Operating License No. DPR-33 with the attached page 3.

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain a marginal line indicating the area of change.

<u>REMOVE</u>	<u>INSERT</u>
1.1-3a	1.1-3a
1.1-7	1.1-7
3.4-8	3.4-8
3.5-5	3.5-5
3.5-7	3.5-7
3.6-16	3.6-16
3.6-23	3.6-23
3.6-33	3.6-33
5.0-9	5.0-9
5.0-12	5.0-12
5.0-14	5.0-14
5.0-15	5.0-15
5.0-16	5.0-16
5.0-17a	50-17a
5.0-21a	5.0-21a
5.0-21b	5.0-21b
5.0-21c	5.0-21c
5.0-21d	5.0-21d
---	5.0-21e

accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report as supplemented and amended;

- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or equipment and instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This subsequent renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 3952 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 346, are hereby incorporated in this subsequent renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

For Surveillance Requirements (SRs) that are new in Amendment 234 to Facility Operating License DPR-33, the first performance is due at the end of the first surveillance interval that begins at implementation of the Amendment 234. For SRs that existed prior to Amendment 234, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendment 234.

- (3) The licensee is authorized to relocate certain requirements included in Appendix A and the former Appendix B to licensee-controlled documents.

1.1 Definitions (continued)

DRAIN TIME (continued)

2. Penetration flow paths capable of being isolated by valves that will close automatically without offsite power prior to the RPV water level being equal to the TAF when actuated by RPV water level isolation instrumentation; or
 3. Penetration flow paths with isolation devices that can be closed prior to the RPV water level being equal to the TAF by a dedicated operator trained in the task, who is in continuous communication with the control room, is stationed at the controls, and is capable of closing the penetration flow path isolation devices without offsite power.
- c. The penetration flow paths required to be evaluated per paragraph b are assumed to open instantaneously and are not subsequently isolated, and no water is assumed to be subsequently added to the RPV water inventory;
 - d. No additional draining events occur; and
 - e. Realistic cross-sectional areas and drain rates are used.

A bounding DRAIN TIME may be used in lieu of a calculated value.

INSERVICE TESTING PROGRAM

The INSERVICE TESTING PROGRAM is the licensee program that fulfills the requirements of 10 CFR 50.55a(f). (see Section 5.5.15)

(continued)

1.1 Definitions (continued)

SHUTDOWN MARGIN
(SDM)

SDM shall be the amount of reactivity by which the reactor is subcritical or would be subcritical throughout the operating cycle assuming that:

- a. The reactor is xenon free;
- b. The moderator temperature is $\geq 68^{\circ}\text{F}$, corresponding to the most reactive state; and
- c. All control rods are fully inserted except for the single control rod of highest reactivity worth, which is assumed to be fully withdrawn. With control rods not capable of being fully inserted, the reactivity worth of these control rods must be accounted for in the determination of SDM.

THERMAL POWER

THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.3.1	Verify the as-left OPS lift pressures of the required safety/relief valves (S/RVs) are within $\pm 1\%$ of the nominal setpoint:	In accordance with the Surveillance Frequency Control Program
	<u>Number of OPS S/RVs</u>	
	<u>Nominal Setpoint (psig)</u>	
	4	1135
	4	1145
	5	1155
SR 3.4.3.2	Verify the as-found OPS lift pressures of the required S/RVs are within the limits specified in the COLR.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE				FREQUENCY																				
SR 3.5.1.5	-----NOTES----- 1. Only required to be performed when in MODE 4 > 48 hours. 2. Not required to be performed if performed within the previous 31 days. ----- Verify each recirculation pump discharge valve cycles through one complete cycle of full travel.			Once prior to entering MODE 2 from MODE 3 or 4																				
SR 3.5.1.6	Verify the following ECCS pumps develop the specified flow rate against a system head corresponding to the specified pressure. <table><thead><tr><th><u>SYSTEM</u></th><th><u>FLOW RATE</u></th><th><u>NO. OF PUMPS</u></th><th><u>SYSTEM HEAD CORRESPONDING TO A VESSEL TO TORUS DIFFERENTIAL PRESSURE OF</u></th></tr></thead><tbody><tr><td>Core Spray</td><td>≥ 6250 gpm</td><td>2</td><td>≥ 105 psid</td></tr></tbody></table> <table><thead><tr><th><u>SYSTEM</u></th><th><u>FLOW RATE</u></th><th><u>NO. OF PUMPS</u></th><th><u>INDICATED SYSTEM PRESSURE</u></th></tr></thead><tbody><tr><td>LPCI</td><td>≥ 12,000 gpm</td><td>2</td><td>≥ 250 psig</td></tr><tr><td>LPCI</td><td>≥ 9,000 gpm</td><td>1</td><td>≥ 125 psig</td></tr></tbody></table>			<u>SYSTEM</u>	<u>FLOW RATE</u>	<u>NO. OF PUMPS</u>	<u>SYSTEM HEAD CORRESPONDING TO A VESSEL TO TORUS DIFFERENTIAL PRESSURE OF</u>	Core Spray	≥ 6250 gpm	2	≥ 105 psid	<u>SYSTEM</u>	<u>FLOW RATE</u>	<u>NO. OF PUMPS</u>	<u>INDICATED SYSTEM PRESSURE</u>	LPCI	≥ 12,000 gpm	2	≥ 250 psig	LPCI	≥ 9,000 gpm	1	≥ 125 psig	In accordance with the Surveillance Frequency Control Program
<u>SYSTEM</u>	<u>FLOW RATE</u>	<u>NO. OF PUMPS</u>	<u>SYSTEM HEAD CORRESPONDING TO A VESSEL TO TORUS DIFFERENTIAL PRESSURE OF</u>																					
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LPCI	≥ 12,000 gpm	2	≥ 250 psig																					
LPCI	≥ 9,000 gpm	1	≥ 125 psig																					

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.5.1.10	-----NOTE----- Valve actuation may be excluded. ----- Verify the ADS actuates on an actual or simulated automatic initiation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.5.1.11	-----NOTE----- Not required to be performed until 12 hours after reactor steam pressure and flow are adequate to perform the test. ----- Verify each ADS valve is capable of being opened. <u>OR</u> Verify each ADS valve opens when manually actuated.	In accordance with the Surveillance Frequency Control Program
SR 3.5.1.12	(Deleted).	

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.1.3.5	Verify the isolation time of each power operated, automatic PCIV, except for MSIVs, is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.6	Verify the isolation time of each MSIV is ≥ 3 seconds and ≤ 5 seconds.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.7	Verify each automatic PCIV actuates to the isolation position on an actual or simulated isolation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.8	Verify a representative sample of reactor instrumentation line EFCVs actuate to the isolation position on a simulated instrument line break signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.9	Remove and test the explosive squib from each shear isolation valve of the TIP System.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.10	Verify leakage rate through each MSIV is ≤ 100 scfh and that the combined leakage rate for all four main steam lines is ≤ 150 scfh when tested at ≥ 25 psig.	In accordance with the Primary Containment Leakage Rate Testing Program
SR 3.6.1.3.11	Verify inboard MSIV air supply header pressure is ≥ 90 psig.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.6.1.6.1 -----NOTES----- 1. Not required to be met for vacuum breakers that are open during Surveillances. 2. One drywell suppression chamber vacuum breaker may be nonfully closed so long as it is determined to be not more than 3° open as indicated by the position lights. ----- Verify each vacuum breaker is closed.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.6.2 Perform a functional test of each required vacuum breaker.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.6.3 Verify the differential pressure required to open each vacuum breaker is ≤ 0.5 psid.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.2.3.1	Verify each RHR suppression pool cooling subsystem manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in the correct position or can be aligned to the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.6.2.3.2	Verify each RHR pump develops a flow rate ≥ 9000 gpm through the associated heat exchanger while operating in the suppression pool cooling mode.	In accordance with the Surveillance Frequency Control Program

5.5 Programs and Manuals

5.5.1 Offsite Dose Calculation Manual (ODCM) (continued)

- c. Shall be submitted to the NRC in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

5.5.2 Primary Coolant Sources Outside Containment

This program provides controls to minimize leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to levels as low as practicable. The systems include the Core Spray, High Pressure Coolant Injection, Residual Heat Removal, and Reactor Core Isolation Cooling. The program shall include the following preventive maintenance:

- a. Periodic visual inspection requirements; and
- b. System leak test requirements for each system, to the extent permitted by system design and radiological conditions, at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 are applicable.

(continued)

5.5 Programs and Manuals

5.5.4 Radioactive Effluent Controls Program (continued)

- h. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I;
- i. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half lives > 8 days in gaseous effluents released from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I; and
- j. Limitations on the annual dose or dose commitment to any member of the public beyond the site boundary due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190.
- k. The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Radioactive Effluent Controls Program surveillance frequency.

5.5.5 Component Cyclic or Transient Limit

This program provides controls to track the FSAR Section 4.2.5, cyclic and transient occurrences to ensure that components are maintained within the design limits.

5.5.6 Deleted

(continued)

5.5 Programs and Manuals (continued)

5.5.7 Ventilation Filter Testing Program (VFTP)

The VFTP shall establish the required testing of Engineered Safety Feature (ESF) filter ventilation systems. The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the VFTP test frequencies.

- a. Demonstrate for each of the ESF systems (Standby Gas Treatment (SGT) System and Control Room Emergency Ventilation (CREV) System) that an inplace test of the HEPA filters shows a penetration and system bypass $\leq 1.0\%$ when tested in accordance with ANSI N510-1975 at the system flowrate specified below, $\pm 10\%$. An alternative challenge aerosol may be used in accordance with Regulatory Guide 1.52, Revision 3, Section 6.3.

ESF Ventilation System	Flowrate (cfm)
SGT System	9000
CREV System	3000

This testing shall be performed 1) in accordance with the Surveillance Frequency Control Program, 2) after partial or complete replacement of HEPA filters, 3) after any structural maintenance on the system housing, or 4) following significant painting, fire, or chemical release in any ventilation zone communicating with the system.

(continued)

5.5 Programs and Manuals

5.5.7 Ventilation Filter Testing Program (VFTP) (continued)

- b. Demonstrate for each of the ESF systems that an inplace test of the charcoal adsorber shows a penetration and system bypass $\leq 1.0\%$ when tested in accordance with ANSI N510-1975 at the system flowrate specified below, $\pm 10\%$. An alternative challenge gas may be used in accordance with Regulatory Guide 1.52, Revision 3, Section 6.4.

ESF Ventilation System	Flowrate (cfm)
SGT System	9000
CREV System	3000

This testing shall be performed 1) in accordance with the Surveillance Frequency Control Program, 2) after partial or complete replacement of the charcoal adsorber bank, 3) after any structural maintenance on the system housing, or 4) following significant painting, fire, or chemical release in any ventilation zone communicating with the system.

- c. Demonstrate for each of the ESF systems that a laboratory test of a sample of the charcoal adsorber, shows a methyl iodide efficiency $\geq 90\%$ when tested in accordance with ASTM D3803-1989.

This testing shall be performed 1) in accordance with the Surveillance Frequency Control Program, 2) after every 720 hours of system operation, or 3) following significant painting, fire, or chemical release in any ventilation zone communicating with the system.

(continued)

5.5 Programs and Manuals

5.5.7 Ventilation Filter Testing Program (VFTP) (continued)

- d. In accordance with the Surveillance Frequency Control Program, demonstrate for each of the ESF systems that the pressure drop across the combined HEPA filters, the prefilters, and the charcoal adsorbers is less than the value specified below at the system flowrate specified below, $\pm 10\%$:

ESF Ventilation System	Delta P (inches water)	Flowrate (cfm)
SGT System	7	9000
CREV System	6	3000

- e. In accordance with the Surveillance Frequency Control Program, demonstrate that the heaters for the SGT System dissipate ≥ 40 kW when tested in accordance with ANSI N510-1975.

5.5.8 Explosive Gas and Storage Tank Radioactivity Monitoring Program

This program provides controls for potentially explosive gas mixtures contained downstream of the offgas recombiners, and the quantity of radioactivity contained in unprotected outdoor liquid storage tanks.

The program shall include:

- a. The limits for concentrations of hydrogen downstream of the offgas recombiners and a surveillance program to ensure the limits are maintained. Such limits shall be appropriate to the system's design criteria (i.e., whether or not the system is designed to withstand a hydrogen explosion); and

(continued)

5.5 Programs and Manuals

5.5.9 Diesel Fuel Oil Testing Program (continued)

- c. Total particulate concentration of the fuel oil in the 7-day storage tank is ≤ 10 mg/l when tested at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Diesel Fuel Oil Testing Program testing frequencies.

(continued)

5.5 Programs and Manuals

5.5.13 Control Room Envelope Habitability Program (continued)

- a. The definition of CRE and the CRE boundary.
- b. Requirements for maintaining the CRE boundary in its design condition including configuration control and preventive maintenance.
- c. Requirements for (i) determining the unfiltered air leakage past the CRE boundary into the CRE at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and in accordance with the testing methods and at the event-driven Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.
- d. Measurements, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one subsystem of the CREV System, operating at the flow rate required by the VFTP, at a frequency in accordance with the Surveillance Frequency Control Program. The results shall be trended and used as part of the periodic assessment of the CRE boundary.
- e. The quantitative limits on unfiltered air leakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air leakage measured by the testing described in paragraph c. The unfiltered air leakage limit for radiological challenges is the leakage for flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air leakage limits for hazardous chemicals must ensure that exposure of the CRE occupants to these hazards will be within the assumptions in the licensing basis.
- f. The provisions of SR 3.0.2 are applicable to the Frequencies for assessing CRE habitability, determining CRE unfiltered leakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

(continued)

5.5 Programs and Manuals

5.5.14 Residual Heat Removal (RHR) Heat Exchanger Performance Monitoring Program

This program is established to ensure that the RHR heat exchangers are maintained in a condition that meets or exceeds the minimum performance capability assumed in containment analyses, which support not taking credit for containment accident pressure in the NPSH analyses. The RHR heat exchanger testing and determination of overall uncertainty in the fouling resistance shall be in accordance with the guidelines in EPRI report, EPRI 3002005340, Service Water Heat Exchanger Test Guidelines, May 2015. This program establishes the following attributes.

- a. The program establishes provisions to periodically monitor RHR heat exchanger thermal performance. The program includes frequency of monitoring and the methodology considers uncertainty of the result.
- b. The program establishes and controls acceptance criteria for RHR heat exchanger worst fouling resistance and number of plugged tubes.
- c. The program establishes limitations and allows for compensatory actions if degraded performance is observed.
- d. Changes to the program shall be made under appropriate administrative review.
- e. Details of the program including program limitations, compensatory actions for degraded performance, testing method, data acquisition method, data reduction method, overall uncertainty determination method, thermal performance analysis, acceptance criteria, and computer programs used that meet the 10 CFR 50 Appendix B, and 10 CFR 21 requirements are described in the UFSAR.

5.5.15 Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies and the frequencies of other periodic testing required by these Technical Specifications (herein also referred to as Surveillance Frequencies). The program shall ensure that Surveillance Requirements and other required testing specified in these Technical Specifications are performed at

(continued)

5.5 Programs and Manuals

5.5.15 Surveillance Frequency Control Program (continued)

intervals sufficient to assure the associated Limiting Conditions for Operation and other requirements are met.

- a. The Surveillance Frequency Control Program shall contain a list of Frequencies controlled by the programs that govern the activities listed in items b.1, b.2, and b.3.
- b. Changes to the Frequencies listed in the Surveillance Frequency Control Program shall be made:
 - 1. In accordance with the requirements of 10 CFR 50.55a for the Inservice Testing Program required by 10 CFR 50.55a(f);
 - 2. In accordance with the requirements of 10 CFR 50.69(d)(2) for testing permitted under 10 CFR 50.69(b)(1)(v);
 - 3. Otherwise, in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1, or as specifically approved by the NRC.
- c. The provisions of Surveillance Requirements 3.0.2 and 3.0.3 are applicable to the Frequencies established in the Surveillance Frequency Control Program, except :
 - 1. The SR 3.0.2 test interval extensions for components in the Inservice Testing Program cannot exceed those allowed by the ASME OM Code as incorporated by reference in 10 CFR 50.55a; and
 - 2. Surveillance Requirement 3.0.3 is only applicable to other required testing when invoked by a Chapter 5 Specification.

(continued)

5.5 Programs and Manuals

5.5.16 Risk Informed Completion Time Program

This program provides controls to calculate a Risk Informed Completion Time (RICT) and must be implemented in accordance with NEI 06-09-A, Revision 0, "Risk-Managed Technical Specifications (RMTS) Guidelines." The program shall include the following:

- a. The RICT may not exceed 30 days;
- b. A RICT may only be utilized in MODES 1 and 2;
- c. When a RICT is being used, any change to the plant configuration, as defined in NEI-06-09-A, Appendix A, must be considered for the effect on the RICT.
 1. For planned changes, the revised RICT must be determined prior to implementation of the change in configuration.
 2. For emergent conditions, the revised RICT must be determined within the time limits of the Required Action Completion Time (i.e., not the RICT) or 12 hours after the plant configuration change, whichever is less.
 3. Revising the RICT is not required If the plant configuration change would lower plant risk and would result in a longer RICT.
- d. For emergent conditions, if the extent of condition evaluation for inoperable structures, systems, or components (SSCs) is not complete prior to exceeding the Completion Time, the RICT shall account for the increased possibility of common cause failure (CCF) by either:
 1. Numerically accounting for the increased possibility of CCF in the RICT calculation; or

(continued)

5.5 Programs and Manuals

5.5.16 Risk Informed Completion Time Program (continued)

2. Risk Management Actions (RMAs) not already credited in the RICT calculation shall be implemented that support redundant or diverse SSCs that perform the function(s) of the inoperable SSCs, and, if practicable, reduce the frequency of initiating events that challenge the function(s) performed by the inoperable SSCs.
 - e. A RICT calculation must include the following hazard groups; internal flood and internal events PRA model, internal fire PRA model, and seismic PRA model. Changes to these means of assessing the hazard groups require prior NRC approval.
 - f. The PRA models used to calculate a RICT shall be maintained and upgraded in accordance with the processes endorsed in the regulatory positions of Regulatory Guide 1.200, Revision 3, "Acceptability of Probabilistic Risk Assessment Results for Risk-Informed Activities."
 - g. A report shall be submitted in accordance with Specification 5.6.8 before a newly developed method is used to calculate a RICT.
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-260

BROWNS FERRY NUCLEAR PLANT, UNIT 2

AMENDMENT TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE

Amendment No. 369
Subsequent Renewed License No. DPR-52

1. The Nuclear Regulatory Commission (NRC or the Commission) has found that:
 - A. The application for amendment by the Tennessee Valley Authority (TVA or the licensee) dated February 17, 2026, as supplemented by letter dated July 21, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR) Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Subsequent Renewed Facility Operating License No. DPR-52 is hereby amended, in part, to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 369, are hereby incorporated in this subsequent renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 6 months from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

MICHAEL
MAHONEY

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MICHAEL MAHONEY
Date: 2026.09.25
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Hipólito González, Chief
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to Subsequent Renewed
Facility Operating License No. DPR-52
and the Technical Specifications

Date of Issuance: September 25, 2026

ATTACHMENT TO LICENSE AMENDMENT NO. 369
TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-52
BROWNS FERRY NUCLEAR PLANT, UNIT 2
DOCKET NO. 50-260

Replace page 3 of Subsequent Renewed Facility Operating License No. DPR-52 with the attached page 3.

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain a marginal line indicating the area of change.

<u>REMOVE</u>	<u>INSERT</u>
1.1-3a	1.1-3a
1.1-7	1.1-7
3.4-8	3.4-8
3.5-5	3.5-5
3.5-7	3.5-7
3.6-16	3.6-16
3.6-23	3.6-23
3.6-33	3.6-33
5.0-9	5.0-9
5.0-12	5.0-12
5.0-14	5.0-14
5.0-15	5.0-15
5.0-16	5.0-16
5.0-17a	50-17a
5.0-21a	5.0-21a
5.0-21b	5.0-21b
5.0-21c	5.0-21c
5.0-21d	5.0-21d
---	5.0-21e

accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report as supplemented and amended;

- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or equipment and instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This subsequent renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 3952 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 369, are hereby incorporated in this subsequent renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

For Surveillance Requirements (SRs) that are new in Amendment 253 to Facility Operating License DPR-52, the first performance is due at the end of the first surveillance interval that begins at implementation of the Amendment 253. For SRs that existed prior to Amendment 253, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendment 253.

- (3) The licensee is authorized to relocate certain requirements included in Appendix A and the former Appendix B to licensee-controlled documents.

1.1 Definitions (continued)

DRAIN TIME (continued)

automatic valves that are closed and administratively controlled in the closed position, blank flanges, or other devices that prevent flow of reactor coolant through the penetration flow paths;

2. Penetration flow paths capable of being isolated by valves that will close automatically without offsite power prior to the RPV water level being equal to the TAF when actuated by RPV water level isolation instrumentation; or

3. Penetration flow paths with isolation devices that can be closed prior to the RPV water level being equal to the TAF by a dedicated operator trained in the task, who is in continuous communication with the control room, is stationed at the controls, and is capable of closing the penetration flow path isolation devices without offsite power.

c. The penetration flow paths required to be evaluated per paragraph b are assumed to open instantaneously and are not subsequently isolated, and no water is assumed to be subsequently added to the RPV water inventory;

d. No additional draining events occur; and

e. Realistic cross-sectional areas and drain rates are used.

A bounding DRAIN TIME may be used in lieu of a calculated value.

INSERVICE TESTING PROGRAM

The INSERVICE TESTING PROGRAM is the licensee program that fulfills the requirements of 10 CFR 50.55a(f). (see Section 5.5.15)

(continued)

1.1 Definitions (continued)

THERMAL POWER	THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.
TURBINE BYPASS SYSTEM RESPONSE TIME	The TURBINE BYPASS SYSTEM RESPONSE TIME consists of two components: - The time from initial movement of the main turbine stop valve or control valve until 80% of the turbine bypass capacity is established; and - The time from initial movement of the main turbine stop valve or control valve until initial movement of the turbine bypass valve. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured.

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.3.1	Verify the as-left OPS lift pressures of the required safety/relief valves (S/RVs) are within $\pm 1\%$ of the nominal setpoint:	In accordance with the Surveillance Frequency Control Program
	<u>Number of OPS S/RVs</u>	
	<u>Nominal Setpoint (psig)</u>	
	4	
	4	
	5	
	1135	
	1145	
	1155	
SR 3.4.3.2	Verify the as-found OPS lift pressures of the required S/RVs are within the limits specified in the COLR.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY																												
SR 3.5.1.5	-----NOTES----- 1. Only required to be performed when in MODE 4 > 48 hours. 2. Not required to be performed if performed within the previous 31 days. ----- Verify each recirculation pump discharge valve cycles through one complete cycle of full travel.	Once prior to entering MODE 2 from MODE 3 or 4																												
SR 3.5.1.6	Verify the following ECCS pumps develop the specified flow rate against a system head corresponding to the specified pressure. <table><tr><td></td><td></td><td></td><td>SYSTEM HEAD CORRESPONDING TO A VESSEL TO TORUS DIFFERENTIAL PRESSURE OF</td></tr><tr><td><u>SYSTEM</u></td><td><u>FLOW RATE</u></td><td><u>NO. OF PUMPS</u></td><td><u>PRESSURE OF</u></td></tr><tr><td>Core Spray</td><td>≥ 6250 gpm</td><td>2</td><td>≥ 105 psid</td></tr></table> <table><tr><td></td><td></td><td></td><td>INDICATED SYSTEM PRESSURE</td></tr><tr><td><u>SYSTEM</u></td><td><u>FLOW RATE</u></td><td><u>NO. OF PUMPS</u></td><td><u>PRESSURE</u></td></tr><tr><td>LPCI</td><td>≥ 12,000 gpm</td><td>2</td><td>≥ 250 psig</td></tr><tr><td>LPCI</td><td>≥ 9,000 gpm</td><td>1</td><td>≥ 125 psig</td></tr></table>				SYSTEM HEAD CORRESPONDING TO A VESSEL TO TORUS DIFFERENTIAL PRESSURE OF	<u>SYSTEM</u>	<u>FLOW RATE</u>	<u>NO. OF PUMPS</u>	<u>PRESSURE OF</u>	Core Spray	≥ 6250 gpm	2	≥ 105 psid				INDICATED SYSTEM PRESSURE	<u>SYSTEM</u>	<u>FLOW RATE</u>	<u>NO. OF PUMPS</u>	<u>PRESSURE</u>	LPCI	≥ 12,000 gpm	2	≥ 250 psig	LPCI	≥ 9,000 gpm	1	≥ 125 psig	In accordance with the Surveillance Frequency Control Program
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LPCI	≥ 12,000 gpm	2	≥ 250 psig																											
LPCI	≥ 9,000 gpm	1	≥ 125 psig																											

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.5.1.10	-----NOTE----- Valve actuation may be excluded. ----- Verify the ADS actuates on an actual or simulated automatic initiation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.5.1.11	-----NOTE----- Not required to be performed until 12 hours after reactor steam pressure and flow are adequate to perform the test. ----- Verify each ADS valve is capable of being opened. <u>OR</u> Verify each ADS valve opens when manually actuated.	In accordance with the Surveillance Frequency Control Program
SR 3.5.1.12	(Deleted)	

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.1.3.5	Verify the isolation time of each power operated, automatic PCIV, except for MSIVs, is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.6	Verify the isolation time of each MSIV is ≥ 3 seconds and ≤ 5 seconds.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.7	Verify each automatic PCIV actuates to the isolation position on an actual or simulated isolation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.8	Verify a representative sample of reactor instrumentation line EFCVs actuate to the isolation position on a simulated instrument line break signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.9	Remove and test the explosive squib from each shear isolation valve of the TIP System.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.10	Verify leakage rate through each MSIV is ≤ 100 scfh and that the combined leakage rate for all four main steam lines is ≤ 150 scfh when tested at ≥ 25 psig.	In accordance with the Primary Containment Leakage Rate Testing Program
SR 3.6.1.3.11	Verify inboard MSIV air supply header pressure is ≥ 90 psig.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.1.6.1	-----NOTES----- 1. Not required to be met for vacuum breakers that are open during Surveillances. 2. One drywell suppression chamber vacuum breaker may be nonfully closed so long as it is determined to be not more than 3° open as indicated by the position lights. ----- Verify each vacuum breaker is closed.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.6.2	Perform a functional test of each required vacuum breaker.	In accordance with the Surveillance Frequency Program
SR 3.6.1.6.3	Verify the differential pressure required to open each vacuum breaker is ≤ 0.5 psid.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.2.3.1	Verify each RHR suppression pool cooling subsystem manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in the correct position or can be aligned to the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.6.2.3.2	Verify each RHR pump develops a flow rate ≥ 9000 gpm through the associated heat exchanger while operating in the suppression pool cooling mode.	In accordance with the Surveillance Frequency Control Program

5.5 Programs and Manuals

5.5.1 Offsite Dose Calculation Manual (ODCM) (continued)

- c. Shall be submitted to the NRC in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

5.5.2 Primary Coolant Sources Outside Containment

This program provides controls to minimize leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to levels as low as practicable. The systems include the Core Spray, High Pressure Coolant Injection, Residual Heat Removal, and Reactor Core Isolation Cooling. The program shall include the following preventive maintenance:

- a. Periodic visual inspection requirements; and
- b. System leak test requirements for each system, to the extent permitted by system design and radiological conditions, at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 are applicable.

(continued)

5.5 Programs and Manuals

5.5.4 Radioactive Effluent Controls Program (continued)

- h. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I;
- i. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half lives > 8 days in gaseous effluents released from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I; and
- j. Limitations on the annual dose or dose commitment to any member of the public beyond the site boundary due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190.
- k. The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Radioactive Effluent Controls Program surveillance frequency.

5.5.5 Component Cyclic or Transient Limit

This program provides controls to track the FSAR Section 4.2.5, cyclic and transient occurrences to ensure that components are maintained within the design limits.

5.5.6 Deleted

(continued)

5.5 Programs and Manuals (continued)

5.5.7 Ventilation Filter Testing Program (VFTP)

The VFTP shall establish the required testing of Engineered Safety Feature (ESF) filter ventilation systems. The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the VFTP test frequencies.

- a. Demonstrate for each of the ESF systems (Standby Gas Treatment (SGT) System and Control Room Emergency Ventilation (CREV) System) that an inplace test of the HEPA filters shows a penetration and system bypass $\leq 1.0\%$ when tested in accordance with ANSI N510-1975 at the system flowrate specified below, $\pm 10\%$. An alternative challenge aerosol may be used in accordance with Regulatory Guide 1.52, Revision 3, Section 6.3.

ESF Ventilation System	Flowrate (cfm)
SGT System	9000
CREV System	3000

This testing shall be performed 1) in accordance with the Surveillance Frequency Control Program, 2) after partial or complete replacement of HEPA filters, 3) after any structural maintenance on the system housing, or 4) following significant painting, fire, or chemical release in any ventilation zone communicating with the system.

(continued)

5.5 Programs and Manuals

5.5.7 Ventilation Filter Testing Program (VFTP) (continued)

- b. Demonstrate for each of the ESF systems that an inplace test of the charcoal adsorber shows a penetration and system bypass $\leq 1.0\%$ when tested in accordance with ANSI N510-1975 at the system flowrate specified below, $\pm 10\%$. An alternative challenge gas may be used in accordance with Regulatory Guide 1.52, Revision 3, Section 6.4.

ESF Ventilation System	Flowrate (cfm)
SGT System	9000
CREV System	3000

This testing shall be performed 1) in accordance with the Surveillance Frequency Control Program, 2) after partial or complete replacement of the charcoal adsorber bank, 3) after any structural maintenance on the system housing, or 4) following significant painting, fire, or chemical release in any ventilation zone communicating with the system.

- c. Demonstrate for each of the ESF systems that a laboratory test of a sample of the charcoal adsorber, shows a methyl iodide efficiency $\geq 90\%$ when tested in accordance with ASTM D3803-1989.

This testing shall be performed 1) in accordance with the Surveillance Frequency Control Program, 2) after every 720 hours of system operation, or 3) following significant painting, fire, or chemical release in any ventilation zone communicating with the system.

(continued)

5.5 Programs and Manuals

5.5.7 Ventilation Filter Testing Program (VFTP) (continued)

- d. In accordance with the Surveillance Frequency Control Program, demonstrate for each of the ESF systems that the pressure drop across the combined HEPA filters, the prefilters, and the charcoal adsorbers is less than the value specified below at the system flowrate specified below, $\pm 10\%$:

ESF Ventilation System	Delta P (inches water)	Flowrate (cfm)
SGT System	7	9000
CREV System	6	3000

- e. In accordance with the Surveillance Frequency Control Program, demonstrate that the heaters for the SGT System dissipate ≥ 40 kW when tested in accordance with ANSI N510-1975.

5.5.8 Explosive Gas and Storage Tank Radioactivity Monitoring Program

This program provides controls for potentially explosive gas mixtures contained downstream of the offgas recombiners, and the quantity of radioactivity contained in unprotected outdoor liquid storage tanks.

The program shall include:

- a. The limits for concentrations of hydrogen downstream of the offgas recombiners and a surveillance program to ensure the limits are maintained. Such limits shall be appropriate to the system's design criteria (i.e., whether or not the system is designed to withstand a hydrogen explosion); and

(continued)

5.5 Programs and Manuals

5.5.9 Diesel Fuel Oil Testing Program (continued)

- b. Within 31 days following addition of the new fuel oil to the 7-day storage tanks, verify that the properties of the new fuel oil, other than those addressed in a., above, are within limits for ASTM 2D fuel oil, and
- c. Total particulate concentration of the fuel oil in the 7-day storage tank is ≤ 10 mg/l when tested at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Diesel Fuel Oil Testing Program testing frequencies.

(continued)

5.5 Programs and Manuals

5.5.13 Control Room Envelope Habitability Program (continued)

- a. The definition of CRE and the CRE boundary.
- b. Requirements for maintaining the CRE boundary in its design condition including configuration control and preventive maintenance.
- c. Requirements for (i) determining the unfiltered air leakage past the CRE boundary into the CRE at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and in accordance with the testing methods and at the event-driven Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.
- d. Measurements, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one subsystem of the CREV System, operating at the flow rate required by the VFTP, at a frequency in accordance with the Surveillance Frequency Control Program. The results shall be trended and used as part of the periodic assessment of the CRE boundary.
- e. The quantitative limits on unfiltered air leakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air leakage measured by the testing described in paragraph c. The unfiltered air leakage limit for radiological challenges is the leakage for flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air leakage limits for hazardous chemicals must ensure that exposure of the CRE occupants to these hazards will be within the assumptions in the licensing basis.
- f. The provisions of SR 3.0.2 are applicable to the Frequencies for assessing CRE habitability, determining CRE unfiltered leakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

(continued)

5.5 Programs and Manuals

5.5.14 Residual Heat Removal (RHR) Heat Exchanger Performance Monitoring Program

This program is established to ensure that the RHR heat exchangers are maintained in a condition that meets or exceeds the minimum performance capability assumed in containment analyses, which support not taking credit for containment accident pressure in the NPSH analyses. The RHR heat exchanger testing and determination of overall uncertainty in the fouling resistance shall be in accordance with the guidelines in EPRI report, EPRI 3002005340, Service Water Heat Exchanger Test Guidelines, May 2015. This program establishes the following attributes.

- a. The program establishes provisions to periodically monitor RHR heat exchanger thermal performance. The program includes frequency of monitoring and the methodology considers uncertainty of the result.
- b. The program establishes and controls acceptance criteria for RHR heat exchanger worst fouling resistance and number of plugged tubes.
- c. The program establishes limitations and allows for compensatory actions if degraded performance is observed.
- d. Changes to the program shall be made under appropriate administrative review.
- e. Details of the program including program limitations, compensatory actions for degraded performance, testing method, data acquisition method, data reduction method, overall uncertainty determination method, thermal performance analysis, acceptance criteria, and computer programs used that meet the 10 CFR 50 Appendix B, and 10 CFR 21 requirements are described in the UFSAR.

5.5.15 Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies and the frequencies of other periodic testing required by these Technical Specifications (herein also referred to as Surveillance Frequencies). The program shall ensure that Surveillance Requirements and other required testing specified in these Technical Specifications are performed at

(continued)

5.5 Programs and Manuals

5.5.15 Surveillance Frequency Control Program (continued)

intervals sufficient to assure the associated Limiting Conditions for Operation and other requirements are met.

- a. The Surveillance Frequency Control Program shall contain a list of Frequencies controlled by the programs that govern the activities listed in items b.1, b.2, and b.3.
- b. Changes to the Frequencies listed in the Surveillance Frequency Control Program shall be made:
 - 1. In accordance with the requirements of 10 CFR 50.55a for the Inservice Testing Program required by 10 CFR 50.55a(f);
 - 2. In accordance with the requirements of 10 CFR 50.69(d)(2) for testing permitted under 10 CFR 50.69(b)(1)(v);
 - 3. Otherwise, in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1, or as specifically approved by the NRC.
- c. The provisions of Surveillance Requirements 3.0.2 and 3.0.3 are applicable to the Frequencies established in the Surveillance Frequency Control Program, except:
 - 1. The SR 3.0.2 test interval extensions for components in the Inservice Testing Program cannot exceed those allowed by the ASME OM Code as incorporated by reference in 10 CFR 50.55a; and
 - 2. Surveillance Requirement 3.0.3 is only applicable to other required testing when invoked by a Chapter 5 Specification.

(continued)

5.5 Programs and Manuals

5.5.16 Risk Informed Completion Time Program

This program provides controls to calculate a Risk Informed Completion Time (RICT) and must be implemented in accordance with NEI 06-09-A, Revision 0, "Risk-Managed Technical Specifications (RMTS) Guidelines." The program shall include the following:

- a. The RICT may not exceed 30 days;
- b. A RICT may only be utilized in MODES 1 and 2;
- c. When a RICT is being used, any change to the plant configuration, as defined in NEI-06-09-A, Appendix A, must be considered for the effect on the RICT.
 1. For planned changes, the revised RICT must be determined prior to implementation of the change in configuration.
 2. For emergent conditions, the revised RICT must be determined within the time limits of the Required Action Completion Time (i.e., not the RICT) or 12 hours after the plant configuration change, whichever is less.
 3. Revising the RICT is not required If the plant configuration change would lower plant risk and would result in a longer RICT.
- d. For emergent conditions, if the extent of condition evaluation for inoperable structures, systems, or components (SSCs) is not complete prior to exceeding the Completion Time, the RICT shall account for the increased possibility of common cause failure (CCF) by either:
 1. Numerically accounting for the increased possibility of CCF in the RICT calculation; or

(continued)

5.5 Programs and Manuals

5.5.16 Risk Informed Completion Time Program (continued)

2. Risk Management Actions (RMAs) not already credited in the RICT calculation shall be implemented that support redundant or diverse SSCs that perform the function(s) of the inoperable SSCs, and, if practicable, reduce the frequency of initiating events that challenge the function(s) performed by the inoperable SSCs.
 - e. A RICT calculation must include the following hazard groups; internal flood and internal events PRA model, internal fire PRA model, and seismic PRA model. Changes to these means of assessing the hazard groups require prior NRC approval.
 - f. The PRA models used to calculate a RICT shall be maintained and upgraded in accordance with the processes endorsed in the regulatory positions of Regulatory Guide 1.200, Revision 3, "Acceptability of Probabilistic Risk Assessment Results for Risk-Informed Activities."
 - g. A report shall be submitted in accordance with Specification 5.6.8 before a newly developed method is used to calculate a RICT.
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-296

BROWNS FERRY NUCLEAR PLANT, UNIT 3

AMENDMENT TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE

Amendment No. 329
Subsequent Renewed License No. DPR-68

1. The Nuclear Regulatory Commission (NRC or the Commission) has found that:
 - A. The application for amendment by the Tennessee Valley Authority (TVA or the licensee) dated February 17, 2026, as supplemented by letter dated July 21, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR) Chapter I;
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3. This license amendment is effective as of its date of issuance and shall be implemented within 6 months from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Date: 2026.09.25
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Hipólito González, Chief
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to Subsequent Renewed
Facility Operating License No. DPR-68
and the Technical Specifications

Date of Issuance: September 25, 2026

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TO SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-68
BROWNS FERRY NUCLEAR PLANT, UNIT 3
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3.6-23	3.6-23
3.6-33	3.6-33
5.0-9	5.0-9
5.0-12	5.0-12
5.0-14	5.0-14
5.0-15	5.0-15
5.0-16	5.0-16
5.0-17a	50-17a
5.0-21a	5.0-21a
5.0-21b	5.0-21b
5.0-21c	5.0-21c
5.0-21d	5.0-21d
---	5.0-21e

accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report as supplemented and amended;

- (3) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use at any time any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source, or special nuclear material without restriction to chemical or physical form for sample analysis or equipment and instrument calibration or associated with radioactive apparatus or components;
- (5) Pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This subsequent renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at steady state reactor core power levels not in excess of 3952 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 329, are hereby incorporated in this subsequent renewed operating license. The licensee shall operate the facility in accordance with the Technical Specifications.

For Surveillance Requirements (SRs) that are new in Amendment 212 to Facility Operating License DPR-68, the first performance is due at the end of the first surveillance interval that begins at implementation of the Amendment 212. For SRs that existed prior to Amendment 212, including SRs with modified acceptance criteria and SRs whose frequency of performance is being extended, the first performance is due at the end of the first surveillance interval that begins on the date the surveillance was last performed prior to implementation of Amendment 212.

- (3) The licensee is authorized to relocate certain requirements included in Appendix A and the former Appendix B to licensee-controlled documents.

1.1 Definitions (continued)

DRAIN TIME (continued)

2. Penetration flow paths capable of being isolated by valves that will close automatically without offsite power prior to the RPV water level being equal to the TAF when actuated by RPV water level isolation instrumentation; or
 3. Penetration flow paths with isolation devices that can be closed prior to the RPV water level being equal to the TAF by a dedicated operator trained in the task, who is in continuous communication with the control room, is stationed at the controls, and is capable of closing the penetration flow path isolation devices without offsite power.
- c. The penetration flow paths required to be evaluated per paragraph b are assumed to open instantaneously and are not subsequently isolated, and no water is assumed to be subsequently added to the RPV water inventory;
- d. No additional draining events occur; and
- e. Realistic cross-sectional areas and drain rates are used.

A bounding DRAIN TIME may be used in lieu of a calculated value.

INSERVICE TESTING PROGRAM

The INSERVICE TESTING PROGRAM is the licensee program that fulfills the requirements of 10 CFR 50.55a(f). (see Section 5.5.15)

(continued)

1.1 Definitions (continued)

THERMAL POWER	THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.
TURBINE BYPASS SYSTEM RESPONSE TIME	The TURBINE BYPASS SYSTEM RESPONSE TIME consists of two components: - The time from initial movement of the main turbine stop valve or control valve until 80% of the turbine bypass capacity is established; and - The time from initial movement of the main turbine stop valve or control valve until initial movement of the turbine bypass valve. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured.

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.3.1	Verify the as-left OPS lift pressures of the required safety/relief valves (S/RVs) are within $\pm 1\%$ of the nominal setpoint:	In accordance with the Surveillance Frequency Control Program
	<u>Number of OPS S/RVs</u>	
	<u>Nominal Setpoint (psig)</u>	
	4	
	4	
	5	In accordance with the Surveillance Frequency Control Program
	1135	
	1145	
	1155	
SR 3.4.3.2	Verify the as-found OPS lift pressures of the required S/RVs are within the limits specified in the COLR.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE				FREQUENCY																												
SR 3.5.1.5	-----NOTES----- 1. Only required to be performed when in MODE 4 > 48 hours. 2. Not required to be performed if performed within the previous 31 days. ----- Verify each recirculation pump discharge valve cycles through one complete cycle of full travel.			Once prior to entering MODE 2 from MODE 3 or 4																												
SR 3.5.1.6	Verify the following ECCS pumps develop the specified flow rate against a system head corresponding to the specified pressure. <table><tr><td></td><td></td><td></td><td>SYSTEM HEAD CORRESPONDING TO A VESSEL TO TORUS DIFFERENTIAL PRESSURE OF</td></tr><tr><td><u>SYSTEM</u></td><td><u>FLOW RATE</u></td><td><u>NO. OF PUMPS</u></td><td><u>PRESSURE OF</u></td></tr><tr><td>Core Spray</td><td>≥ 6250 gpm</td><td>2</td><td>≥ 105 psid</td></tr></table> <table><tr><td></td><td></td><td></td><td>INDICATED SYSTEM PRESSURE</td></tr><tr><td><u>SYSTEM</u></td><td><u>FLOW RATE</u></td><td><u>NO. OF PUMPS</u></td><td><u>PRESSURE</u></td></tr><tr><td>LPCI</td><td>≥ 12,000 gpm</td><td>2</td><td>≥ 250 psig</td></tr><tr><td>LPCI</td><td>≥ 9,000 gpm</td><td>1</td><td>≥ 125 psig</td></tr></table>						SYSTEM HEAD CORRESPONDING TO A VESSEL TO TORUS DIFFERENTIAL PRESSURE OF	<u>SYSTEM</u>	<u>FLOW RATE</u>	<u>NO. OF PUMPS</u>	<u>PRESSURE OF</u>	Core Spray	≥ 6250 gpm	2	≥ 105 psid				INDICATED SYSTEM PRESSURE	<u>SYSTEM</u>	<u>FLOW RATE</u>	<u>NO. OF PUMPS</u>	<u>PRESSURE</u>	LPCI	≥ 12,000 gpm	2	≥ 250 psig	LPCI	≥ 9,000 gpm	1	≥ 125 psig	In accordance with the Surveillance Frequency Control Program
			SYSTEM HEAD CORRESPONDING TO A VESSEL TO TORUS DIFFERENTIAL PRESSURE OF																													
<u>SYSTEM</u>	<u>FLOW RATE</u>	<u>NO. OF PUMPS</u>	<u>PRESSURE OF</u>																													
Core Spray	≥ 6250 gpm	2	≥ 105 psid																													
			INDICATED SYSTEM PRESSURE																													
<u>SYSTEM</u>	<u>FLOW RATE</u>	<u>NO. OF PUMPS</u>	<u>PRESSURE</u>																													
LPCI	≥ 12,000 gpm	2	≥ 250 psig																													
LPCI	≥ 9,000 gpm	1	≥ 125 psig																													

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.5.1.10	-----NOTE----- Valve actuation may be excluded. ----- Verify the ADS actuates on an actual or simulated automatic initiation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.5.1.11	-----NOTE----- Not required to be performed until 12 hours after reactor steam pressure and flow are adequate to perform the test. ----- Verify each ADS valve is capable of being opened. <u>OR</u> Verify each ADS valve opens when manually actuated.	In accordance with the Surveillance Frequency Control Program
SR 3.5.1.12	(Deleted)	

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.1.3.5	Verify the isolation time of each power operated, automatic PCIV, except for MSIVs, is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.6	Verify the isolation time of each MSIV is ≥ 3 seconds and ≤ 5 seconds.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.7	Verify each automatic PCIV actuates to the isolation position on an actual or simulated isolation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.8	Verify a representative sample of reactor instrumentation line EFCVs actuate to the isolation position on a simulated instrument line break signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.9	Remove and test the explosive squib from each shear isolation valve of the TIP System.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.3.10	Verify leakage rate through each MSIV is ≤ 100 scfh and that the combined leakage rate for all four main steam lines is ≤ 150 scfh when tested at ≥ 25 psig.	In accordance with the Primary Containment Leakage Rate Testing Program
SR 3.6.1.3.11	Verify inboard MSIV air supply header pressure is ≥ 90 psig.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.1.6.1	-----NOTES----- 1. Not required to be met for vacuum breakers that are open during Surveillances. 2. One drywell suppression chamber vacuum breaker may be nonfully closed so long as it is determined to be not more than 3° open as indicated by the position lights. ----- Verify each vacuum breaker is closed.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.6.2	Perform a functional test of each required vacuum breaker.	In accordance with the Surveillance Frequency Control Program
SR 3.6.1.6.3	Verify the differential pressure required to open each vacuum breaker is ≤ 0.5 psid.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.2.3.1	Verify each RHR suppression pool cooling subsystem manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in the correct position or can be aligned to the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.6.2.3.2	Verify each RHR pump develops a flow rate ≥ 9000 gpm through the associated heat exchanger while operating in the suppression pool cooling mode.	In accordance with the Surveillance Frequency Control Program

5.5 Programs and Manuals

5.5.1 Offsite Dose Calculation Manual (ODCM) (continued)

- c. Shall be submitted to the NRC in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

5.5.2 Primary Coolant Sources Outside Containment

This program provides controls to minimize leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to levels as low as practicable. The systems include the Core Spray, High Pressure Coolant Injection, Residual Heat Removal, and Reactor Core Isolation Cooling. The program shall include the following preventive maintenance:

- a. Periodic visual inspection requirements; and
- b. System leak test requirements for each system, to the extent permitted by system design and radiological conditions, at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 are applicable.

(continued)

5.5 Programs and Manuals

5.5.4 Radioactive Effluent Controls Program (continued)

- h. Limitations on the annual and quarterly air doses resulting from noble gases released in gaseous effluents from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I;
- i. Limitations on the annual and quarterly doses to a member of the public from iodine-131, iodine-133, tritium, and all radionuclides in particulate form with half lives > 8 days in gaseous effluents released from each unit to areas beyond the site boundary, conforming to 10 CFR 50, Appendix I; and
- j. Limitations on the annual dose or dose commitment to any member of the public beyond the site boundary due to releases of radioactivity and to radiation from uranium fuel cycle sources, conforming to 40 CFR 190.
- k. The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Radioactive Effluent Controls Program surveillance frequency.

5.5.5 Component Cyclic or Transient Limit

This program provides controls to track the FSAR Section 4.2.5, cyclic and transient occurrences to ensure that components are maintained within the design limits.

5.5.6 Deleted

(continued)

5.5 Programs and Manuals (continued)

5.5.7 Ventilation Filter Testing Program (VFTP)

The VFTP shall establish the required testing of Engineered Safety Feature (ESF) filter ventilation systems. The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the VFTP test frequencies.

- a. Demonstrate for each of the ESF systems (Standby Gas Treatment (SGT) System and Control Room Emergency Ventilation (CREV) System) that an inplace test of the HEPA filters shows a penetration and system bypass $\leq 1.0\%$ when tested in accordance with ANSI N510-1975 at the system flowrate specified below, $\pm 10\%$. An alternative challenge aerosol may used in accordance with Regulatory Guide 1.52, Revision 3, Section 6.3.

ESF Ventilation System	Flowrate (cfm)
SGT System	9000
CREV System	3000

This testing shall be performed 1) in accordance with the Surveillance Frequency Control Program, 2) after partial or complete replacement of HEPA filters, 3) after any structural maintenance on the system housing, or 4) following significant painting, fire, or chemical release in any ventilation zone communicating with the system.

(continued)

5.5 Programs and Manuals

5.5.7 Ventilation Filter Testing Program (VFTP) (continued)

- b. Demonstrate for each of the ESF systems that an inplace test of the charcoal adsorber shows a penetration and system bypass $\leq 1.0\%$ when tested in accordance with ANSI N510-1975 at the system flowrate specified below, $\pm 10\%$. An alternative challenge gas may be used in accordance with Regulatory Guide 1.52, Revision 3, Section 6.4.

ESF Ventilation System	Flowrate (cfm)
SGT System	9000
CREV System	3000

This testing shall be performed 1) in accordance with the Surveillance Frequency Control Program, 2) after partial or complete replacement of the charcoal adsorber bank, 3) after any structural maintenance on the system housing, or 4) following significant painting, fire, or chemical release in any ventilation zone communicating with the system.

- c. Demonstrate for each of the ESF systems that a laboratory test of a sample of the charcoal adsorber, shows a methyl iodide efficiency $\geq 90\%$ when tested in accordance with ASTM D3803-1989.

This testing shall be performed 1) in accordance with the Surveillance Frequency Control Program, 2) after every 720 hours of system operation, or 3) following significant painting, fire, or chemical release in any ventilation zone communicating with the system.

(continued)

5.5 Programs and Manuals

5.5.7 Ventilation Filter Testing Program (VFTP) (continued)

- d. In accordance with the Surveillance Frequency Control Program, demonstrate for each of the ESF systems that the pressure drop across the combined HEPA filters, the prefilters, and the charcoal adsorbers is less than the value specified below at the system flowrate specified below, $\pm 10\%$:

ESF Ventilation System	Delta P (inches water)	Flowrate (cfm)
SGT System	7	9000
CREV System	6	3000

- e. In accordance with the Surveillance Frequency Control Program, demonstrate that the heaters for the SGT System dissipate ≥ 40 kW when tested in accordance with ANSI N510-1975.

5.5.8 Explosive Gas and Storage Tank Radioactivity Monitoring Program

This program provides controls for potentially explosive gas mixtures contained downstream of the offgas recombiners, and the quantity of radioactivity contained in unprotected outdoor liquid storage tanks.

The program shall include:

- a. The limits for concentrations of hydrogen downstream of the offgas recombiners and a surveillance program to ensure the limits are maintained. Such limits shall be appropriate to the system's design criteria (i.e., whether or not the system is designed to withstand a hydrogen explosion); and

(continued)

5.5 Programs and Manuals

5.5.9 Diesel Fuel Oil Testing Program (continued)

- c. Total particulate concentration of the fuel oil in the 7-day storage tank is ≤ 10 mg/l when tested at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Diesel Fuel Oil Testing Program testing frequencies.

(continued)

5.5 Programs and Manuals

5.5.13 Control Room Envelope Habitability Program (continued)

- a. The definition of CRE and the CRE boundary.
- b. Requirements for maintaining the CRE boundary in its design condition including configuration control and preventive maintenance.
- c. Requirements for (i) determining the unfiltered air leakage past the CRE boundary into the CRE at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and in accordance with the testing methods and at the event-driven Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.
- d. Measurements, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one subsystem of the CREV System, operating at the flow rate required by the VFTP, at a frequency in accordance with the Surveillance Frequency Control Program. The results shall be trended and used as part of the periodic assessment of the CRE boundary.
- e. The quantitative limits on unfiltered air leakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air leakage measured by the testing described in paragraph c. The unfiltered air leakage limit for radiological challenges is the leakage for flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air leakage limits for hazardous chemicals must ensure that exposure of the CRE occupants to these hazards will be within the assumptions in the licensing basis.
- f. The provisions of SR 3.0.2 are applicable to the Frequencies for assessing CRE habitability, determining CRE unfiltered leakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

(continued)

5.5 Programs and Manuals

5.5.14 Residual Heat Removal (RHR) Heat Exchanger Performance Monitoring Program

This program is established to ensure that the RHR heat exchangers are maintained in a condition that meets or exceeds the minimum performance capability assumed in containment analyses, which support not taking credit for containment accident pressure in the NPSH analyses. The RHR heat exchanger testing and determination of overall uncertainty in the fouling resistance shall be in accordance with the guidelines in EPRI report, EPRI 3002005340, Service Water Heat Exchanger Test Guidelines, May 2015. This program establishes the following attributes.

- a. The program establishes provisions to periodically monitor RHR heat exchanger thermal performance. The program includes frequency of monitoring and the methodology considers uncertainty of the result.
- b. The program establishes and controls acceptance criteria for RHR heat exchanger worst fouling resistance and number of plugged tubes.
- c. The program establishes limitations and allows for compensatory actions if degraded performance is observed.
- d. Changes to the program shall be made under appropriate administrative review.
- e. Details of the program including program limitations, compensatory actions for degraded performance, testing method, data acquisition method, data reduction method, overall uncertainty determination method, thermal performance analysis, acceptance criteria, and computer programs used that meet the 10 CFR 50 Appendix B, and 10 CFR 21 requirements are described in the UFSAR.

5.5.15 Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies and the frequencies of other periodic testing required by these Technical Specifications (herein also referred to as Surveillance Frequencies). The program shall ensure that Surveillance Requirements and other required testing specified in these Technical Specifications are performed at

(continued)

5.5 Programs and Manuals

5.5.15 Surveillance Frequency Control Program (continued)

intervals sufficient to assure the associated Limiting Conditions for Operation and other requirements are met.

- a. The Surveillance Frequency Control Program shall contain a list of Frequencies controlled by the programs that govern the activities listed in items b.1, b.2, and b.3.
- b. Changes to the Frequencies listed in the Surveillance Frequency Control Program shall be made:
 - 1. In accordance with the requirements of 10 CFR 50.55a for the Inservice Testing Program required by 10 CFR 50.55a(f);
 - 2. In accordance with the requirements of 10 CFR 50.69(d)(2) for testing permitted under 10 CFR 50.69(b)(1)(v);
 - 3. Otherwise, in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1, or as specifically approved by the NRC.
- c. The provisions of Surveillance Requirements 3.0.2 and 3.0.3 are applicable to the Frequencies established in the Surveillance Frequency Control Program, except:
 - 1. The SR 3.0.2 test interval extensions for components in the Inservice Testing Program cannot exceed those allowed by the ASME OM Code as incorporated by reference in 10 CFR 50.55a; and
 - 2. Surveillance Requirement 3.0.3 is only applicable to other required testing when invoked by a Chapter 5 Specification.

(continued)

5.5 Programs and Manuals

5.5.16 Risk Informed Completion Time Program

This program provides controls to calculate a Risk Informed Completion Time (RICT) and must be implemented in accordance with NEI 06-09-A, Revision 0, "Risk-Managed Technical Specifications (RMTS) Guidelines." The program shall include the following:

- a. The RICT may not exceed 30 days;
- b. A RICT may only be utilized in MODES 1 and 2;
- c. When a RICT is being used, any change to the plant configuration, as defined in NEI-06-09-A, Appendix A, must be considered for the effect on the RICT.
 1. For planned changes, the revised RICT must be determined prior to implementation of the change in configuration.
 2. For emergent conditions, the revised RICT must be determined within the time limits of the Required Action Completion Time (i.e., not the RICT) or 12 hours after the plant configuration change, whichever is less.
 3. Revising the RICT is not required If the plant configuration change would lower plant risk and would result in a longer RICT.
- d. For emergent conditions, if the extent of condition evaluation for inoperable structures, systems, or components (SSCs) is not complete prior to exceeding the Completion Time, the RICT shall account for the increased possibility of common cause failure (CCF) by either:
 1. Numerically accounting for the increased possibility of CCF in the RICT calculation; or

(continued)

5.5 Programs and Manuals

5.5.16 Risk Informed Completion Time Program (continued)

2. Risk Management Actions (RMAs) not already credited in the RICT calculation shall be implemented that support redundant or diverse SSCs that perform the function(s) of the inoperable SSCs, and, if practicable, reduce the frequency of initiating events that challenge the function(s) performed by the inoperable SSCs.
 - e. A RICT calculation must include the following hazard groups; internal flood and internal events PRA model, internal fire PRA model, and seismic PRA model. Changes to these means of assessing the hazard groups require prior NRC approval.
 - f. The PRA models used to calculate a RICT shall be maintained and upgraded in accordance with the processes endorsed in the regulatory positions of Regulatory Guide 1.200, Revision 3, "Acceptability of Probabilistic Risk Assessment Results for Risk-Informed Activities."
 - g. A report shall be submitted in accordance with Specification 5.6.8 before a newly developed method is used to calculate a RICT.
-



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-327

SEQUOYAH NUCLEAR PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 380
Renewed License No. DPR-77

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the Tennessee Valley Authority (the licensee or TVA) dated February 17, 2026, as supplemented by letter dated July 21, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 380, are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 6 months from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

MICHAEL
MAHONEY

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MICHAEL MAHONEY
Date: 2026.09.25
13:00:45 -04'00'

Hipólito González, Chief
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to Renewed Facility Operating
License No. DPR-77 and the
Technical Specifications

Date of Issuance: September 25, 2026

ATTACHMENT TO LICENSE AMENDMENT NO. 380
TO RENEWED FACILITY OPERATING LICENSE NO. DPR-77
SEQUOYAH NUCLEAR PLANT, UNIT 1
DOCKET NO. 50-327

Replace page 3 of the Renewed Facility Operating License No. DPR-77 with the attached page 3.

Replace the following page of the Appendix A, Technical Specifications, with the attached page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

<u>Remove</u>	<u>Insert</u>
1.1-6	1.1-6
3.4.10-2	3.4.10-2
3.4.14-2	3.4.14-2
3.5.2-2	3.5.2-2
3.5.2-3	3.5.2-3
3.6.3-7	3.6.3-7
--	3.6.3-8
3.6.6-2	3.6.6-2
--	3.6.6-3
3.6.9-1	3.6.9-1
3.7.1-2	3.7.1-2
3.7.2-2	3.7.2-2
3.7.3-2	3.7.3-2
3.7.5-4	3.7.5-4
--	3.7.5-5
5.5-2	5.5-2
5.5-4	5.5-4
5.5-5	5.5-5
5.5-8	5.5-8
5.5-11	5.5-11
5.5-16	5.5-16
5.5-17	5.5-17
5.5-18	5.5-18
5.5-19	5.5-19
--	5.5-20

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the Sequoyah and Watts Bar Unit 1 Nuclear Plants.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The Tennessee Valley Authority is authorized to operate the facility at reactor core power levels not in excess of 3455 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 380 are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Initial Test Program

The Tennessee Valley Authority shall conduct the post-fuel-loading initial test program (set forth in Section 14 of Tennessee Valley Authority's Final Safety Analysis Report, as amended), without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- a. Elimination of any test identified in Section 14 of TVA's Final Safety Analysis Report as amended as being essential;
- b. Modification of test objectives, methods, or acceptance criteria for any test identified in Section 14 of TVA's Final Safety Analysis Report as amended as being essential;

1.1 Definitions

THERMAL POWER	THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.
TRIP ACTUATING DEVICE OPERATIONAL TEST (TADOT)	A TADOT shall consist of operating the trip actuating device and verifying the OPERABILITY of all devices in the channel required for trip actuating device OPERABILITY. The TADOT shall include adjustment, as necessary, of the trip actuating device so that it actuates at the required setpoint within the necessary accuracy. The TADOT may be performed by means of any series of sequential, overlapping, or total channel steps, and each step must be performed within the Frequency in the Surveillance Frequency Control Program for the devices included in the step.

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.10.1	Verify each pressurizer safety valve is OPERABLE with lift settings ≥ 2410 psig and ≤ 2559 psig. Following testing, lift settings shall be within $\pm 1\%$.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. Required Action and associated Completion Time for Condition A not met.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.4.14.1 -----NOTE----- Not required to be performed in MODES 3 and 4. ----- Verify leakage from each RCS PIV is equivalent to ≤ 0.5 gpm per nominal inch of valve size up to a maximum of 5 gpm at an RCS pressure ≥ 2215 psig and ≤ 2255 psig.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Less than 100% of the ECCS flow equivalent to a single OPERABLE ECCS train available.	C.1 Enter LCO 3.0.3.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE			FREQUENCY
SR 3.5.2.1	Verify the following valves are in the listed position with power to the valve operator removed.		In accordance with the Surveillance Frequency Control Program
<u>Number</u>	<u>Position</u>	<u>Function</u>	
FCV-63-1	Open	RHR Suction from RWST	
FCV-63-22	Open	SIS Discharge to Common Piping	
SR 3.5.2.2	Verify each ECCS manual, power operated, and automatic valve in the flow path, that is not locked, sealed, or otherwise secured in position, is in the correct position.		In accordance with the Surveillance Frequency Control Program
SR 3.5.2.3	Verify ECCS piping is full of water.		In accordance with the Surveillance Frequency Control Program
SR 3.5.2.4	Verify each ECCS pump's developed head at the test flow point is greater than or equal to the required developed head.		In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE			FREQUENCY
SR 3.5.2.5	Verify each ECCS automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.		In accordance with the Surveillance Frequency Control Program
SR 3.5.2.6	Verify each ECCS pump starts automatically on an actual or simulated actuation signal.		In accordance with the Surveillance Frequency Control Program
SR 3.5.2.7	Verify, for each ECCS throttle valve listed below, each mechanical stop is in the correct position.		In accordance with the Surveillance Frequency Control Program
	Charging Pump Injection Throttle Valves	Safety Injection Cold Leg Throttle Valves	
	63-582	63-550	
	63-583	63-552	
	63-584	63-554	
	63-585	63-556	
		Safety Injection Hot Leg Throttle Valves	
		63-542	
		63-544	
		63-546	
		63-548	

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.3.3	-----NOTE----- Valves and blind flanges in high radiation areas may be verified by use of administrative means. ----- Verify each containment isolation manual valve and blind flange that is located inside containment and not locked, sealed, or otherwise secured and required to be closed during accident conditions is closed, except for containment isolation valves that are open under administrative controls.	Prior to entering MODE 4 from MODE 5 if not performed within the previous 92 days
SR 3.6.3.4	Verify the isolation time of each automatic power operated containment isolation valve is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.6.3.5	Perform leakage rate testing for containment purge valves with resilient seals.	In accordance with the Surveillance Frequency Control Program
SR 3.6.3.6	Verify each automatic containment isolation valve that is not locked, sealed or otherwise secured in position, actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.3.7	Verify each containment purge valve is blocked to restrict the valve from opening ≥ 50 degrees.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.3.8	Verify the combined leakage rate for all shield building bypass leakage paths is $\leq 0.25 L_a$ when pressurized to $\geq P_a$.	In accordance with the Containment Leakage Rate Testing Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.6.2	Verify each containment spray train pump's developed head at the flow test point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.3	Verify each automatic containment spray train valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.4	Verify each containment spray train pump starts automatically on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.5	Verify each containment spray train spray nozzle is unobstructed.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.6	Verify each RHR spray train manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, is in the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.7	Verify each RHR spray train pump's developed head at the flow test point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.6.8	Verify each RHR spray train spray nozzle is unobstructed.	In accordance with the Surveillance Frequency Control Program

3.6 CONTAINMENT SYSTEMS

3.6.9 Vacuum Relief Valves

LCO 3.6.9 Three vacuum relief lines shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One vacuum relief line inoperable.	A.1 Restore vacuum relief line to OPERABLE status.	72 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3. <u>AND</u>	6 hours
	B.2 Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.9.1	Verify each vacuum relief line is OPERABLE by measuring the force required to open the vacuum relief valve.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. Required Action and associated Completion Time not met. <u>OR</u> One or more steam generators with ≥ 4 MSSVs inoperable.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.1.1 -----NOTE----- Only required to be performed in MODES 1 and 2. ----- Verify each required MSSV lift setpoint per Table 3.7.1-2. Following testing, lift setting shall be within $\pm 1\%$.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and associated Completion Time of Condition C not met.	D.1 Be in MODE 3. <u>AND</u>	6 hours
	D.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.2.1	-----NOTE----- Only required to be performed in MODES 1 and 2. ----- Verify the isolation time of each MSIV is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.7.2.2	-----NOTE----- Only required to be performed in MODES 1 and 2. ----- Verify each MSIV actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. One or more flow paths with two valves inoperable.	D.1 Isolate affected flow path.	8 hours
E. Required Action and associated Completion Time not met.	E.1 Be in MODE 3. <u>AND</u>	6 hours
	E.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.3.1	Verify the isolation time of each MFIV, MFRV, and MFRV bypass valve is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.7.3.2	Verify each MFIV, MFRV, and MFRV bypass valve actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
SR 3.7.5.2 -----NOTE----- Not required to be performed for the turbine driven AFW pump until 24 hours after ≥ 842 psig in the steam generator. ----- Verify the developed head of each AFW pump at the flow test point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program
SR 3.7.5.3 -----NOTES----- 1. AFW train(s) may be considered OPERABLE during alignment and operation for steam generator level control, if it is capable of being manually realigned to the AFW mode of operation. 2. Only required to be met in MODES 1, 2, and 3. ----- Verify each AFW automatic valve that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
SR 3.7.5.4 -----NOTES----- 1. Not required to be performed for the turbine driven AFW pump until 24 hours after ≥ 842 psig in the steam generator. 2. AFW train(s) may be considered OPERABLE during alignment and operation for steam generator level control, if it is capable of being manually realigned to the AFW mode of operation. 3. Only required to be met in MODES 1, 2, and 3. ----- Verify each AFW pump starts automatically on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

5.5 Programs and Manuals

5.5.2 Primary Coolant Sources Outside Containment

This program provides controls to minimize leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to levels as low as practicable. The systems include Safety Injection, Chemical and Volume Control, Residual Heat Removal, Containment Spray, and RCS Sampling. The program shall include the following:

- a. Preventive maintenance and periodic visual inspection requirements and
- b. Integrated leak test requirements for each system at a Frequency in accordance with the Surveillance Frequency Program.

The provisions of SR 3.0.2 are applicable.

5.5.3 Radioactive Effluent Controls Program

This program conforms to 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as reasonably achievable. The program shall be contained in the ODCM, shall be implemented by procedures, and shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- a. Limitations on the functional capability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM,
- b. Limitations on the concentrations of radioactive material released in liquid effluents to unrestricted areas, conforming to ten times the concentration values in Appendix B, Table 2, Column 2 to 10 CFR 20.1001-20.2402,
- c. Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the ODCM,
- d. Limitations on the annual and quarterly doses or dose commitment to a member of the public from radioactive materials in liquid effluents released from each unit to unrestricted areas, conforming to 10 CFR 50, Appendix I,
- e. Determination of cumulative dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days. Determination of projected dose contributions from radioactive effluents in accordance with the methodology in the ODCM at least every 31 days,

5.5 Programs and Manuals

5.5.4 Component Cyclic or Transient Limit

This program provides controls to track the UFSAR, Section 5.2.1, cyclic and transient occurrences to ensure that components are maintained within the design limits.

5.5.5 Reactor Coolant Pump Flywheel Inspection Program

This program shall provide for the inspection of each reactor coolant pump flywheel per the recommendations of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975.

In lieu of Position C.4.b(1) and C.4.b(2), a qualified in-place ultrasonic examination over the volume from the inner bore of the flywheel to the circle one-half of the outer radius or a surface examination (magnetic particle and/or liquid penetrant) of exposed surfaces of the removed flywheels may be conducted at 20 year intervals.

5.5.6 Inservice Testing Program (see Section 5.5.17)

The Inservice Testing Program is the licensee program that fulfills the requirements of 10 CFR 50.55a(f). The provisions of SR 3.0.3 are applicable to inservice testing activities.

5.5 Programs and Manuals

5.5.7 Steam Generator (SG) Program

An SG Program shall be established and implemented to ensure that SG tube integrity is maintained. In addition, the SG Program shall include the following:

- a. Provisions for condition monitoring assessments. Condition monitoring assessment means an evaluation of the "as found" condition of the tubing with respect to the performance criteria for structural integrity and accident induced leakage. The "as found" condition refers to the condition of the tubing during an SG inspection outage, as determined from the inservice inspection results or by other means, prior to the plugging of tubes. Condition monitoring assessments shall be conducted during each outage during which the SG tubes are inspected or plugged, to confirm that the performance criteria are being met.
- b. Performance criteria for SG tube integrity. SG tube integrity shall be maintained by meeting the performance criteria for tube structural integrity, accident induced leakage, and operational LEAKAGE.
 1. Structural integrity performance criterion: All in-service SG tubes shall retain structural integrity over the full range of normal operating conditions (including startup, operation in the power range, hot standby, and cool down), all anticipated transients included in the design specification, and design basis accidents. This includes retaining a safety factor of 3.0 against burst under normal steady state full power operation primary-to-secondary pressure differential and a safety factor of 1.4 against burst applied to the design basis accident primary-to-secondary pressure differentials. Apart from the above requirements, additional loading conditions associated with the design basis accidents, or combination of accidents in accordance with the design and licensing basis, shall also be evaluated to determine if the associated loads contribute significantly to burst or collapse. In the assessment of tube integrity, those loads that do significantly affect burst or collapse shall be determined and assessed in combination with the loads due to pressure with a safety factor of 1.2 on the combined primary loads and 1.0 on axial secondary loads.

5.5 Programs and Manuals

5.5.8 Secondary Water Chemistry Program

This program provides controls for monitoring secondary water chemistry to inhibit SG tube degradation. The program shall include:

- a. Identification of a sampling schedule for the critical variables and control points for these variables;
- b. Identification of the procedures used to measure the values of the critical variables;
- c. Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in leakage;
- d. Procedures for the recording and management of data;
- e. Procedures defining corrective actions for all off control point chemistry conditions; and
- f. A procedure identifying the authority responsible for the interpretation of the data and the sequence and timing of administrative events, which is required to initiate corrective action.

5.5.9 Ventilation Filter Testing Program (VFTP)

A program shall be established to implement the following required testing of Engineered Safety Feature (ESF) filter ventilation systems at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and the event-based Frequencies in accordance with Regulatory Positions C.5.a, C.5.c, C.5.d, and C.6.b of Regulatory Guide 1.52, Revision 2, ANSI N510-1975 and ASTM D3803-1989.

The test described in Specification 5.5.9.a and 5.5.9.b shall be performed in accordance with the Surveillance Frequency Control Program; after any structural maintenance on the high efficiency particulate air (HEPA) filter bank or charcoal adsorber bank housing; following painting, fire, or chemical release in any ventilation zone communicating with the system; and after each complete or partial replacement of a HEPA filter bank or charcoal adsorber bank.

The test described in Specification 5.5.9.c shall be performed in accordance with the Surveillance Frequency Control Program or after 720 hours of filter operation; after any structural maintenance on the HEPA filter bank or charcoal adsorber bank housing; and following painting, fire, or chemical release in any ventilation zone communicating with the system.

The test described in Specification 5.5.9.d and 5.5.9.e shall be performed in accordance with the Surveillance Frequency Control Program.

5.5 Programs and Manuals

5.5.10 Explosive Gas and Storage Tank Radioactivity Monitoring Program (continued)

- c. A surveillance program to ensure that the quantity of radioactivity contained in all outdoor temporary liquid radwaste storage tanks, Condensate Storage Tank, and Steam Generator Layup Tank is less than the amount that would result in concentrations exceeding the limits of 10 CFR 20, Appendix B, Table 2, Column 2, at the nearest potable water supply and the nearest surface water supply in an unrestricted area, in the event of an uncontrolled release of the tanks' contents.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Explosive Gas and Storage Tank Radioactivity Monitoring Program surveillance frequencies.

5.5.11 Diesel Fuel Oil Testing Program

A diesel fuel oil testing program to implement required testing of both new fuel oil and stored fuel oil shall be established. The program shall include sampling and testing requirements, and acceptance criteria, all in accordance with applicable ASTM Standards. The purpose of the program is to establish the following:

- a. Acceptability of new fuel oil for use prior to addition to storage tanks by determining that the fuel oil has:
 - 1. An API gravity or an absolute specific gravity within limits,
 - 2. A flash point and kinematic viscosity within limits for ASTM 2D fuel oil, and
 - 3. A clear and bright appearance with proper color or a water and sediment content within limits.
- b. Within 31 days following addition of the new fuel oil to storage tanks, verify that the properties of the new fuel oil, other than those addressed in a., above, are within limits for ASTM 2D fuel oil, and
- c. Total particulate concentration of the fuel oil is ≤ 10 mg/l when tested at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Diesel Fuel Oil Testing Program test frequencies.

5.5 Programs and Manuals

5.5.15 Battery Monitoring and Maintenance Program (continued)

5. A requirement to obtain specific gravity readings of all cells at each discharge test, consistent with manufacturer recommendations.

5.5.16 Control Room Envelope (CRE) Habitability Program

A Control Room Envelope (CRE) Habitability Program shall be established and implemented to ensure that CRE habitability is maintained such that, with an OPERABLE Control Room Emergency Ventilation System (CREVS), CRE occupants can control the reactor safely under normal conditions and maintain it in a safe condition following a radiological event, hazardous chemical release, or a smoke challenge. The program shall ensure that adequate radiation protection is provided to permit access and occupancy of the CRE under design basis accident (DBA) conditions without personnel receiving radiation exposures in excess of 5 rem total effective dose equivalent (TEDE) for the duration of the accident. The program shall include the following elements:

- a. The definition of the CRE and the CRE boundary.
- b. Requirements for maintaining the CRE boundary in its design condition including configuration control and preventive maintenance.
- c. Requirements for (i) determining the unfiltered air leakage past the CRE boundary into the CRE at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and in accordance with the testing methods and at the event-driven Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.
- d. Measurement, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one train of the CREVS, operating at the flow rate required by the VFTP, at a Frequency in accordance with the Surveillance Frequency Control Program. The results shall be trended and used as part of the periodic assessment of the CRE boundary.
- e. The quantitative limits on unfiltered air leakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air leakage measured by the testing described in paragraph c. The unfiltered air leakage limit for radiological challenges is the leakage flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air leakage limits for hazardous chemicals must ensure that exposure of CRE occupants to these hazards will be within the assumptions in the licensing basis.

5.5 Programs and Manuals

5.5.16 Control Room Envelope (CRE) Habitability Program (continued)

- f. The provisions of SR 3.0.2 are applicable to the Frequencies for assessing CRE habitability, determining CRE unfiltered inleakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

5.5.17 Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies and the frequencies of other periodic testing required by these Technical Specifications (herein also referred to as Surveillance Frequencies). The program shall ensure that Surveillance Requirements and other required testing specified in these Technical Specifications are performed at intervals sufficient to assure the associated Limiting Conditions for Operation and other requirements are met.

- a. The Surveillance Frequency Control Program shall contain a list of Frequencies controlled by the programs that govern the activities listed in items b.1, b.2, and b.3.
- b. Changes to the Frequencies listed in the Surveillance Frequency Control Program shall be made:
 - 1. In accordance with the requirements of 10 CFR 50.55a for the Inservice Testing Program required by 10 CFR 50.55a(f);
 - 2. In accordance with the requirements of 10 CFR 50.69(d)(2) for testing permitted under 10 CFR 50.69(b)(1)(v);
 - 3. Otherwise, in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1, or as specifically approved by the NRC.
- c. The provisions of Surveillance Requirements 3.0.2 and 3.0.3 are applicable to the Frequencies established in the Surveillance Frequency Control Program, except:
 - 1. The SR 3.0.2 test interval extensions for components in the Inservice Testing Program cannot exceed those allowed by the ASME OM Code as incorporated by reference in 10 CFR 50.55a; and
 - 2. Surveillance Requirement 3.0.3. is only applicable to other required testing when invoked by a Chapter 5 Specification.

5.5 Programs and Manuals

5.5.18 Risk Informed Completion Time Program

This program provides controls to calculate a Risk Informed Completion Time (RICT) and must be implemented in accordance with NEI 06-09-A, Revision 0, "Risk-Managed Technical Specifications (RMTS) Guidelines." The program shall include the following:

- a. The RICT may not exceed 30 days;
- b. A RICT may only be utilized in MODE 1 and 2;
- c. When a RICT is being used, any change to the plant configuration, as defined in NEI 06-09-A, Appendix A, must be considered for the effect on the RICT.
 1. For planned changes, the revised RICT must be determined prior to implementation of the change in configuration.
 2. For emergent conditions, the revised RICT must be determined within the time limits of the Required Action Completion Time (i.e., not the RICT) or 12 hours after the plant configuration change, whichever is less.
 3. Revising the RICT is not required if the plant configuration change would lower plant risk and would result in a longer RICT.
- d. For emergent conditions, if the extent of condition evaluation for inoperable structures, systems, or components (SSCs) is not complete prior to exceeding the Completion Time, the RICT shall account for the increased possibility of common cause failure (CCF) by either:
 1. Numerically accounting for the increased possibility of CCF in the RICT calculation; or
 2. Risk Management Actions (RMAs) not already credited in the RICT calculation shall be implemented that support redundant or diverse SSCs that perform the function(s) of the inoperable SSCs, and, if practicable, reduce the frequency of initiating events that challenge the function(s) performed by the inoperable SSCs.
- e. A RICT calculation must include the following hazard groups: internal flood and internal events PRA model, internal fire PRA model, and seismic PRA model. Changes to these means of assessing the hazard groups require prior NRC approval.

5.5 Programs and Manuals

5.5.18 Risk Informed Completion Time Program (continued)

- f. The PRA models used to calculate a RICT shall be maintained and upgraded in accordance with the processes endorsed in the regulatory positions of Regulatory Guide 1.200, Revision 3, "Acceptability of Probabilistic Risk Assessment Results for Risk Informed Activities."
- g. A report shall be submitted in accordance with Specification 5.6.7 before a newly developed method is used to calculate a RICT.

5.5.19 Online Monitoring Program

This program provides controls to determine the need for calibration of pressure, level, and flow transmitters using condition monitoring based on drift analysis. It also provides a means for in-situ dynamic response assessment using the noise analysis technique to detect failure modes that are not detectable by drift monitoring.

The Online Monitoring Program must be implemented in accordance with AMS-TR-0720R2-A, "Online Monitoring Technology to Extend Calibration Intervals of Nuclear Plant Pressure Transmitters" (proprietary version). The program shall include the following elements:

- a. Implementation of online monitoring for transmitters that have been evaluated in accordance with an NRC approved methodology during the plant operating cycle.
 - 1. Analysis of online monitoring data to identify those transmitters that require a calibration check and those that do not require a calibration check,
 - 2. Performance of online monitoring using noise analysis to assess in-situ dynamic response of transmitters that can affect response time performance,
 - 3. Calibration checks of identified transmitters no later than during the next scheduled refueling outage, and
 - 4. Documentation of the results of the online monitoring data analysis.
- b. Performance of a calibration check for any transmitter where the online monitoring was not implemented during the plant operating cycle no later than during the next scheduled refueling outage.
- c. Performance of calibration checks for transmitters at the specified backstop frequencies.

5.5 Programs and Manuals

5.5.19 Online Monitoring Program (continued)

- d. The provisions of Surveillance Requirement 3.0.3 are applicable to the required calibration checks specified in items a.3, b, and c above.
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-328

SEQUOYAH NUCLEAR PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 375
Renewed License No. DPR-79

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment by the Tennessee Valley Authority (the licensee or TVA) dated February 17, 2026, as supplemented by letter dated July 21, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-79 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 375, are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 6 months from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

MICHAEL
MAHONEY

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MICHAEL MAHONEY
Date: 2026.09.25
13:01:35 -04'00'

Hipólito González, Chief
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to Renewed Facility Operating
License No. DPR-79 and the
Technical Specifications

Date of Issuance: September 25, 2026

ATTACHMENT TO LICENSE AMENDMENT NO. 375
TO RENEWED FACILITY OPERATING LICENSE NO. DPR-79
SEQUOYAH NUCLEAR PLANT, UNIT 2
DOCKET NO. 50-328

Replace page 3 of the Renewed Facility Operating License No. DPR-79 with the attached page 3.

Replace the following page of the Appendix A, Technical Specifications, with the attached page. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

<u>Remove</u>	<u>Insert</u>
1.1-6	1.1-6
3.4.10-2	3.4.10-2
3.4.14-2	3.4.14-2
3.5.2-2	3.5.2-2
--	3.5.2-3
3.6.3-7	3.6.3-7
--	3.6.3-8
3.6.6-2	3.6.6-2
--	3.6.6-3
3.6.9-1	3.6.9-1
3.7.1-2	3.7.1-2
3.7.2-2	3.7.2-2
3.7.3-2	3.7.3-2
3.7.5-4	3.7.5-4
--	3.7.5-5
5.5-2	5.5-2
5.5-4	5.5-4
5.5-5	5.5-5
5.5-8	5.5-8
5.5-11	5.5-11
5.5-16	5.5-16
5.5-17	5.5-17
5.5-18	5.5-18
5.5-19	5.5-19
--	5.5-20

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the Sequoyah and Watts Bar Unit 1 Nuclear Plants.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The Tennessee Valley Authority is authorized to operate the facility at reactor core power levels not in excess of 3455 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 375 are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications.

(3) Initial Test Program

The Tennessee Valley Authority shall conduct the post-fuel-loading initial test program (set forth in Section 14 of Tennessee Valley Authority's Final Safety Analysis Report, as amended), without making any major modifications of this program unless modifications have been identified and have received prior NRC approval. Major modifications are defined as:

- a. Elimination of any test identified in Section 14 of TVA's Final Safety Analysis Report as amended as being essential;

1.1 Definitions

THERMAL POWER	THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.
TRIP ACTUATING DEVICE OPERATIONAL TEST (TADOT)	A TADOT shall consist of operating the trip actuating device and verifying the OPERABILITY of all devices in the channel required for trip actuating device OPERABILITY. The TADOT shall include adjustment, as necessary, of the trip actuating device so that it actuates at the required setpoint within the necessary accuracy. The TADOT may be performed by means of any series of sequential, overlapping, or total channel steps, and each step must be performed within the Frequency in the Surveillance Frequency Control Program for the devices included in the step.

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.10.1	Verify each pressurizer safety valve is OPERABLE with lift settings ≥ 2410 psig and ≤ 2559 psig. Following testing, lift settings shall be within $\pm 1\%$.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. Required Action and associated Completion Time for Condition A not met.	B.1 Be in MODE 3. <u>AND</u>	6 hours
	B.2 Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.4.14.1 -----NOTE----- Not required to be performed in MODES 3 and 4. ----- Verify leakage from each RCS PIV is equivalent to ≤ 0.5 gpm per nominal inch of valve size up to a maximum of 5 gpm at an RCS pressure ≥ 2215 psig and ≤ 2255 psig.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Less than 100% of the ECCS flow equivalent to a single OPERABLE ECCS train available.	C.1 Enter LCO 3.0.3.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE			FREQUENCY
SR 3.5.2.1	Verify the following valves are in the listed position with power to the valve operator removed.		In accordance with the Surveillance Frequency Control Program
<u>Number</u>	<u>Position</u>	<u>Function</u>	
FCV-63-1	Open	RHR Suction from RWST	
FCV-63-22	Open	SIS Discharge to Common Piping	
SR 3.5.2.2	Verify each ECCS manual, power operated, and automatic valve in the flow path, that is not locked, sealed, or otherwise secured in position, is in the correct position.		In accordance with the Surveillance Frequency Control Program
SR 3.5.2.3	Verify ECCS piping is full of water.		In accordance with the Surveillance Frequency Control Program
SR 3.5.2.4	Verify each ECCS pump's developed head at the test flow point is greater than or equal to the required developed head.		In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE			FREQUENCY
SR 3.5.2.5	Verify each ECCS automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.		In accordance with the Surveillance Frequency Control Program
SR 3.5.2.6	Verify each ECCS pump starts automatically on an actual or simulated actuation signal.		In accordance with the Surveillance Frequency Control Program
SR 3.5.2.7	Verify, for each ECCS throttle valve listed below, each mechanical stop is in the correct position.		In accordance with the Surveillance Frequency Control Program
	Charging Pump Injection Throttle Valves	Safety Injection Cold Leg Throttle Valves	
	63-582	63-550	
	63-583	63-552	
	63-584	63-554	
	63-585	63-556	
		Safety Injection Hot Leg Throttle Valves	
		63-542	
		63-544	
		63-546	
		63-548	

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.3.3	-----NOTE----- Valves and blind flanges in high radiation areas may be verified by use of administrative means. ----- Verify each containment isolation manual valve and blind flange that is located inside containment and not locked, sealed, or otherwise secured and required to be closed during accident conditions is closed, except for containment isolation valves that are open under administrative controls.	Prior to entering MODE 4 from MODE 5 if not performed within the previous 92 days
SR 3.6.3.4	Verify the isolation time of each automatic power operated containment isolation valve is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.6.3.5	Perform leakage rate testing for containment purge valves with resilient seals.	In accordance with the Surveillance Frequency Control Program
SR 3.6.3.6	Verify each automatic containment isolation valve that is not locked, sealed or otherwise secured in position, actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.3.7	Verify each containment purge valve is blocked to restrict the valve from opening ≥ 50 degrees.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.3.8	Verify the combined leakage rate for all shield building bypass leakage paths is $\leq 0.25 L_a$ when pressurized to $\geq P_a$.	In accordance with the Containment Leakage Rate Testing Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.6.2	Verify each containment spray train pump's developed head at the flow test point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.3	Verify each automatic containment spray train valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.4	Verify each containment spray train pump starts automatically on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.5	Verify each containment spray train spray nozzle is unobstructed.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.6	Verify each RHR spray train manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, is in the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.7	Verify each RHR spray train pump's developed head at the flow test point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.6.8	Verify each RHR spray train spray nozzle is unobstructed.	In accordance with the Surveillance Frequency Control Program

3.6 CONTAINMENT SYSTEMS

3.6.9 Vacuum Relief Valves

LCO 3.6.9 Three vacuum relief lines shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One vacuum relief line inoperable.	A.1 Restore vacuum relief line to OPERABLE status.	72 hours
B. Required Action and associated Completion Time not met.	B.1 Be in MODE 3. <u>AND</u>	6 hours
	B.2 Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.9.1	Verify each vacuum relief line is OPERABLE by measuring the force required to open the vacuum relief valve.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. Required Action and associated Completion Time not met. <u>OR</u> One or more steam generators with ≥ 4 MSSVs inoperable.	B.1 Be in MODE 3.	6 hours
	<u>AND</u> B.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.1.1 -----NOTE----- Only required to be performed in MODES 1 and 2. ----- Verify each required MSSV lift setpoint per Table 3.7.1-2. Following testing, lift setting shall be within $\pm 1\%$.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and associated Completion Time of Condition C not met.	D.1 Be in MODE 3. <u>AND</u>	6 hours
	D.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.2.1	-----NOTE----- Only required to be performed in MODES 1 and 2. ----- Verify the isolation time of each MSIV is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.7.2.2	-----NOTE----- Only required to be performed in MODES 1 and 2. ----- Verify each MSIV actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. One or more flow paths with two valves inoperable.	D.1 Isolate affected flow path.	8 hours
E. Required Action and associated Completion Time not met.	E.1 Be in MODE 3. <u>AND</u>	6 hours
	E.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.3.1	Verify the isolation time of each MFIV, MFRV, and MFRV bypass valve is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.7.3.2	Verify each MFIV, MFRV, and MFRV bypass valve actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
SR 3.7.5.2 -----NOTE----- Not required to be performed for the turbine driven AFW pump until 24 hours after ≥ 842 psig in the steam generator. ----- Verify the developed head of each AFW pump at the flow test point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program
SR 3.7.5.3 -----NOTES----- 1. AFW train(s) may be considered OPERABLE during alignment and operation for steam generator level control, if it is capable of being manually realigned to the AFW mode of operation. 2. Only required to be met in MODES 1, 2, and 3. ----- Verify each AFW automatic valve that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE	FREQUENCY
SR 3.7.5.4 -----NOTES----- 1. Not required to be performed for the turbine driven AFW pump until 24 hours after ≥ 842 psig in the steam generator. 2. AFW train(s) may be considered OPERABLE during alignment and operation for steam generator level control, if it is capable of being manually realigned to the AFW mode of operation. 3. Only required to be met in MODES 1, 2, and 3. ----- Verify each AFW pump starts automatically on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

5.5 Programs and Manuals

5.5.2 Primary Coolant Sources Outside Containment

This program provides controls to minimize leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to levels as low as practicable. The systems include Safety Injection, Chemical and Volume Control, Residual Heat Removal, Containment Spray, and RCS Sampling. The program shall include the following:

- a. Preventive maintenance and periodic visual inspection requirements and
- b. Integrated leak test requirements for each system at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 are applicable.

5.5.3 Radioactive Effluent Controls Program

This program conforms to 10 CFR 50.36a for the control of radioactive effluents and for maintaining the doses to members of the public from radioactive effluents as low as reasonably achievable. The program shall be contained in the ODCM, shall be implemented by procedures, and shall include remedial actions to be taken whenever the program limits are exceeded. The program shall include the following elements:

- a. Limitations on the functional capability of radioactive liquid and gaseous monitoring instrumentation including surveillance tests and setpoint determination in accordance with the methodology in the ODCM,
- b. Limitations on the concentrations of radioactive material released in liquid effluents to unrestricted areas, conforming to ten times the concentration values in Appendix B, Table 2, Column 2 to 10 CFR 20.1001-20.2402,
- c. Monitoring, sampling, and analysis of radioactive liquid and gaseous effluents in accordance with 10 CFR 20.1302 and with the methodology and parameters in the ODCM,
- d. Limitations on the annual and quarterly doses or dose commitment to a member of the public from radioactive materials in liquid effluents released from each unit to unrestricted areas, conforming to 10 CFR 50, Appendix I,
- e. Determination of cumulative dose contributions from radioactive effluents for the current calendar quarter and current calendar year in accordance with the methodology and parameters in the ODCM at least every 31 days. Determination of projected dose contributions from radioactive effluents in accordance with the methodology in the ODCM at least every 31 days,

5.5 Programs and Manuals

5.5.4 Component Cyclic or Transient Limit

This program provides controls to track the UFSAR, Section 5.2.1, cyclic and transient occurrences to ensure that components are maintained within the design limits.

5.5.5 Reactor Coolant Pump Flywheel Inspection Program

This program shall provide for the inspection of each reactor coolant pump flywheel per the recommendations of Regulatory Position C.4.b of Regulatory Guide 1.14, Revision 1, August 1975.

In lieu of Position C.4.b(1) and C.4.b(2), a qualified in-place ultrasonic examination over the volume from the inner bore of the flywheel to the circle one-half of the outer radius or a surface examination (magnetic particle and/or liquid penetrant) of exposed surfaces of the removed flywheels may be conducted at 20 year intervals.

5.5.6 Inservice Testing Program (see Section 5.5.17)

The Inservice Testing Program is the licensee program that fulfills the requirements of 10 CFR 50.55a(f). The provisions of SR 3.0.3 are applicable to inservice testing activities.

5.5 Programs and Manuals

5.5.7 Steam Generator (SG) Program

An SG Program shall be established and implemented to ensure that SG tube integrity is maintained. In addition, the SG Program shall include the following:

- a. Provisions for condition monitoring assessments. Condition monitoring assessment means an evaluation of the "as found" condition of the tubing with respect to the performance criteria for structural integrity and accident induced leakage. The "as found" condition refers to the condition of the tubing during an SG inspection outage, as determined from the inservice inspection results or by other means, prior to the plugging of tubes. Condition monitoring assessments shall be conducted during each outage during which the SG tubes are inspected or plugged, to confirm that the performance criteria are being met.
- b. Performance criteria for SG tube integrity. SG tube integrity shall be maintained by meeting the performance criteria for tube structural integrity, accident induced leakage, and operational LEAKAGE.
 1. Structural integrity performance criterion: All in-service SG tubes shall retain structural integrity over the full range of normal operating conditions (including startup, operation in the power range, hot standby, and cool down), all anticipated transients included in the design specification, and design basis accidents. This includes retaining a safety factor of 3.0 against burst under normal steady state full power operation primary-to-secondary pressure differential and a safety factor of 1.4 against burst applied to the design basis accident primary-to-secondary pressure differentials. Apart from the above requirements, additional loading conditions associated with the design basis accidents, or combination of accidents in accordance with the design and licensing basis, shall also be evaluated to determine if the associated loads contribute significantly to burst or collapse. In the assessment of tube integrity, those loads that do significantly affect burst or collapse shall be determined and assessed in combination with the loads due to pressure with a safety factor of 1.2 on the combined primary loads and 1.0 on axial secondary loads.

5.5 Programs and Manuals

5.5.8 Secondary Water Chemistry Program

This program provides controls for monitoring secondary water chemistry to inhibit SG tube degradation. The program shall include:

- a. Identification of a sampling schedule for the critical variables and control points for these variables;
- b. Identification of the procedures used to measure the values of the critical variables;
- c. Identification of process sampling points, which shall include monitoring the discharge of the condensate pumps for evidence of condenser in leakage;
- d. Procedures for the recording and management of data;
- e. Procedures defining corrective actions for all off control point chemistry conditions; and
- f. A procedure identifying the authority responsible for the interpretation of the data and the sequence and timing of administrative events, which is required to initiate corrective action.

5.5.9 Ventilation Filter Testing Program (VFTP)

A program shall be established to implement the following required testing of Engineered Safety Feature (ESF) filter ventilation systems at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and the event-based Frequencies in accordance with Regulatory Positions C.5.a, C.5.c, C.5.d, and C.6.b of Regulatory Guide 1.52, Revision 2, ANSI N510-1975 and ASTM D3803-1989.

The test described in Specification 5.5.9.a and 5.5.9.b shall be performed in accordance with the Surveillance Frequency Control Program; after any structural maintenance on the high efficiency particulate air (HEPA) filter bank or charcoal adsorber bank housing; following painting, fire, or chemical release in any ventilation zone communicating with the system; and after each complete or partial replacement of a HEPA filter bank or charcoal adsorber bank.

The test described in Specification 5.5.9.c shall be performed in accordance with the Surveillance Frequency Control Program or after 720 hours of filter operation; after any structural maintenance on the HEPA filter bank or charcoal adsorber bank housing; and following painting, fire, or chemical release in any ventilation zone communicating with the system.

The test described in Specification 5.5.9.d and 5.5.9.e shall be performed in accordance with the Surveillance Frequency Control Program.

5.5 Programs and Manuals

5.5.10 Explosive Gas and Storage Tank Radioactivity Monitoring Program (continued)

- c. A surveillance program to ensure that the quantity of radioactivity contained in all outdoor temporary liquid radwaste storage tanks, Condensate Storage Tank, and Steam Generator Layup Tank is less than the amount that would result in concentrations exceeding the limits of 10 CFR 20, Appendix B, Table 2, Column 2, at the nearest potable water supply and the nearest surface water supply in an unrestricted area, in the event of an uncontrolled release of the tanks' contents.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Explosive Gas and Storage Tank Radioactivity Monitoring Program surveillance frequencies.

5.5.11 Diesel Fuel Oil Testing Program

A diesel fuel oil testing program to implement required testing of both new fuel oil and stored fuel oil shall be established. The program shall include sampling and testing requirements, and acceptance criteria, all in accordance with applicable ASTM Standards. The purpose of the program is to establish the following:

- a. Acceptability of new fuel oil for use prior to addition to storage tanks by determining that the fuel oil has:
 - 1. An API gravity or an absolute specific gravity within limits,
 - 2. A flash point and kinematic viscosity within limits for ASTM 2D fuel oil, and
 - 3. A clear and bright appearance with proper color or a water and sediment content within limits.
- b. Within 31 days following addition of the new fuel oil to storage tanks, verify that the properties of the new fuel oil, other than those addressed in a., above, are within limits for ASTM 2D fuel oil, and
- c. Total particulate concentration of the fuel oil is ≤ 10 mg/l when tested at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Diesel Fuel Oil Testing Program test frequencies.

5.5 Programs and Manuals

5.5.15 Battery Monitoring and Maintenance Program (continued)

5. A requirement to obtain specific gravity readings of all cells at each discharge test, consistent with manufacturer recommendations.

5.5.16 Control Room Envelope (CRE) Habitability Program

A Control Room Envelope (CRE) Habitability Program shall be established and implemented to ensure that CRE habitability is maintained such that, with an OPERABLE Control Room Emergency Ventilation System (CREVS), CRE occupants can control the reactor safely under normal conditions and maintain it in a safe condition following a radiological event, hazardous chemical release, or a smoke challenge. The program shall ensure that adequate radiation protection is provided to permit access and occupancy of the CRE under design basis accident (DBA) conditions without personnel receiving radiation exposures in excess of 5 rem total effective dose equivalent (TEDE) for the duration of the accident. The program shall include the following elements:

- a. The definition of the CRE and the CRE boundary.
- b. Requirements for maintaining the CRE boundary in its design condition including configuration control and preventive maintenance.
- c. Requirements for (i) determining the unfiltered air leakage past the CRE boundary into the CRE at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and in accordance with the testing methods and at the event-driven Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.
- d. Measurement, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one train of the CREVS, operating at the flow rate required by the VFTP, at a Frequency in accordance with the Surveillance Frequency Control Program. The results shall be trended and used as part of the periodic assessment of the CRE boundary.
- e. The quantitative limits on unfiltered air leakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air leakage measured by the testing described in paragraph c. The unfiltered air leakage limit for radiological challenges is the leakage flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air leakage limits for hazardous chemicals must ensure that exposure of CRE occupants to these hazards will be within the assumptions in the licensing basis.

5.5 Programs and Manuals

5.5.16 Control Room Envelope (CRE) Habitability Program (continued)

- f. The provisions of SR 3.0.2 are applicable to the Frequencies for assessing CRE habitability, determining CRE unfiltered inleakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

5.5.17 Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies and the frequencies of other periodic testing required by these Technical Specifications (herein also referred to as Surveillance Frequencies). The program shall ensure that Surveillance Requirements and other required testing specified in these Technical Specifications are performed at intervals sufficient to assure the associated Limiting Conditions for Operation and other requirements are met.

- a. The Surveillance Frequency Control Program shall contain a list of Frequencies controlled by the programs that govern the activities listed in items b.1, b.2, and b.3.
- b. Changes to the Frequencies listed in the Surveillance Frequency Control Program shall be made:
 - 1. In accordance with the requirements of 10 CFR 50.55a for the Inservice Testing Program required by 10 CFR 50.55a(f);
 - 2. In accordance with the requirements of 10 CFR 50.69(d)(2) for testing permitted under 10 CFR 50.69(b)(1)(v);
 - 3. Otherwise, in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1, or as specifically approved by the NRC.
- c. The provisions of Surveillance Requirements 3.0.2 and 3.0.3 are applicable to the Frequencies established in the Surveillance Frequency Control Program, except:
 - 1. The SR 3.0.2 test interval extensions for components in the Inservice Testing Program cannot exceed those allowed by the ASME OM Code as incorporated by reference in 10 CFR 50.55a; and
 - 2. Surveillance Requirement 3.0.3. is only applicable to other required testing when invoked by a Chapter 5 Specification.

5.5 Programs and Manuals

5.5.18 Risk Informed Completion Time Program

This program provides controls to calculate a Risk Informed Completion Time (RICT) and must be implemented in accordance with NEI 06-09-A, Revision 0, "Risk-Managed Technical Specifications (RMTS) Guidelines." The program shall include the following:

- a. The RICT may not exceed 30 days;
- b. A RICT may only be utilized in MODE 1 and 2;
- c. When a RICT is being used, any change to the plant configuration, as defined in NEI 06-09-A, Appendix A, must be considered for the effect on the RICT.
 1. For planned changes, the revised RICT must be determined prior to implementation of the change in configuration.
 2. For emergent conditions, the revised RICT must be determined within the time limits of the Required Action Completion Time (i.e., not the RICT) or 12 hours after the plant configuration change, whichever is less.
 3. Revising the RICT is not required if the plant configuration change would lower plant risk and would result in a longer RICT.
- d. For emergent conditions, if the extent of condition evaluation for inoperable structures, systems, or components (SSCs) is not complete prior to exceeding the Completion Time, the RICT shall account for the increased possibility of common cause failure (CCF) by either:
 1. Numerically accounting for the increased possibility of CCF in the RICT calculation; or
 2. Risk Management Actions (RMAs) not already credited in the RICT calculation shall be implemented that support redundant or diverse SSCs that perform the function(s) of the inoperable SSCs, and, if practicable, reduce the frequency of initiating events that challenge the function(s) performed by the inoperable SSCs.
- e. A RICT calculation must include the following hazard groups: internal flood and internal events PRA model, internal fire PRA model, and seismic PRA model. Changes to these means of assessing the hazard groups require prior NRC approval.

5.5 Programs and Manuals

5.5.18 Risk Informed Completion Time Program (continued)

- f. The PRA models used to calculate a RICT shall be maintained and upgraded in accordance with the processes endorsed in the regulatory positions of Regulatory Guide 1.200, Revision 3, "Acceptability of Probabilistic Risk Assessment Results for Risk Informed Activities."
- g. A report shall be submitted in accordance with Specification 5.6.7 before a newly developed method is used to calculate a RICT.

5.5.19 Online Monitoring Program

This program provides controls to determine the need for calibration of pressure, level, and flow transmitters using condition monitoring based on drift analysis. It also provides a means for in-situ dynamic response assessment using the noise analysis technique to detect failure modes that are not detectable by drift monitoring.

The Online Monitoring Program must be implemented in accordance with AMS-TR-0720R2-A, "Online Monitoring Technology to Extend Calibration Intervals of Nuclear Plant Pressure Transmitters" (proprietary version). The program shall include the following elements:

- a. Implementation of online monitoring for transmitters that have been evaluated in accordance with an NRC approved methodology during the plant operating cycle.
 - 1. Analysis of online monitoring data to identify those transmitters that require a calibration check and those that do not require a calibration check,
 - 2. Performance of online monitoring using noise analysis to assess in-situ dynamic response of transmitters that can affect response time performance,
 - 3. Calibration checks of identified transmitters no later than during the next scheduled refueling outage, and
 - 4. Documentation of the results of the online monitoring data analysis.
- b. Performance of a calibration check for any transmitter where the online monitoring was not implemented during the plant operating cycle no later than during the next scheduled refueling outage.
- c. Performance of calibration checks for transmitters at the specified backstop frequencies.

5.5 Programs and Manuals

5.5.19 Online Monitoring Program (continued)

- d. The provisions of Surveillance Requirement 3.0.3 are applicable to the required calibration checks specified in items a.3, b, and c above.
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-390

WATTS BAR NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 186
License No. NPF-90

1. The Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment by the Tennessee Valley Authority (TVA, the licensee) dated February 17, 2026, as supplemented by letter dated July 21, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-90 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 186 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 6 months from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

MICHAEL
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Hipólito González, Chief
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to Facility Operating License
No. NPF-90 and the Technical
Specifications

Date of Issuance: September 25, 2026

ATTACHMENT TO AMENDMENT NO. 186
TO FACILITY OPERATING LICENSE NO. NPF-90
WATTS BAR NUCLEAR PLANT, UNIT 1
DOCKET NO. 50-390

Replace page 3 of Facility Operating License No. NPF-90 with the attached revised page 3.

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain a marginal line indicating the area of change.

<u>Remove</u>	<u>Insert</u>
1.1-6	1.1-6
3.4-21	3.4-21
3.4-34	3.4-34
3.4-35	3.4-35
3.5-5	3.5-5
3.6-13	3.6-13
3.6-19	3.6-19
3.7-2	3.7-2
3.7-6	3.7-6
3.7-8	3.7-8
3.7-13	3.7-13
5.0-11	5.0-11
5.0-14	5.0-14
5.0-18	5.0-18
5.0-22	5.0-22
5.0-23	5.0-23
5.0-25	5.0-25
5.0-25a	5.0-25a
5.0-25c	5.0-25c
5.0-25d	5.0-25d
5.0-25e	5.0-25e

- (4) TVA, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required, any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis, instrument calibration, or other activity associated with radioactive apparatus or components; and
 - (5) TVA, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified or incorporated below.
- (1) Maximum Power Level

TVA is authorized to operate the facility at reactor core power levels not in excess of 3459 megawatts thermal.
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 186 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.
 - (3) Safety Parameter Display System (SPDS) (Section 18.2 of SER Supplements 5 and 15)

Prior to startup following the first refueling outage, TVA shall accomplish the necessary activities, provide acceptable responses, and implement all proposed corrective actions related to having the Watts Bar Unit 1 SPDS operational.
 - (4) Vehicle Bomb Control Program (Section 13.6.9 of SSER 20)

During the period of the exemption granted in paragraph 2.D.(3) of this license, in implementing the power ascension phase of the approved initial test program, TVA shall not exceed 50% power until the requirements of 10 CFR 73.55(c)(7) and (8) are fully implemented. TVA shall submit a letter under oath or affirmation when the requirements of 73.55(c)(7) and (8) have been fully implemented.

1.1 Definitions (continued)

SHUTDOWN MARGIN (SDM)
(continued)

- a. All rod cluster control assemblies (RCCAs) are fully inserted except for the single RCCA of highest reactivity worth, which is assumed to be fully withdrawn. With any RCCA not capable of being fully inserted, the reactivity worth of the RCCA must be accounted for in the determination of SDM; and
- b. In MODES 1 and 2, the fuel and moderator temperatures are changed to the nominal zero power design level.

SLAVE RELAY TEST

A SLAVE RELAY TEST shall consist of energizing each slave relays in the channel required for channel OPERABILITY and verifying the OPERABILITY of each required slave relay. The SLAVE RELAY TEST shall include a continuity check of associated required testable actuation devices. The SLAVE RELAY TEST may be performed by means of any series of sequential, overlapping, or total steps.

THERMAL POWER

THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.

TRIP ACTUATING DEVICE
OPERATIONAL TEST
(TADOT)

A TADOT shall consist of operating the trip actuating device and verifying the OPERABILITY of all devices in the channel required for trip actuating device OPERABILITY. The TADOT shall include adjustment, as necessary, of the trip actuating device so that it actuates at the required setpoint within the necessary accuracy. The TADOT may be performed by means of any series of sequential, overlapping, or total channel steps, and each step must be performed within the Frequency in the Surveillance Frequency Control Program for the devices included in the step.

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.10.1	Verify each pressurizer safety valve is OPERABLE with lift settings ≥ 2410 psig and ≤ 2560 psig. Following testing, lift settings shall be within $\pm 1\%$ of the nominal lift setting of 2485 psig.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.14.1	-----NOTE----- Not required to be performed in MODES 3 and 4. ----- Verify leakage from each RCS PIV is equivalent to ≤ 0.5 gpm per nominal inch of valve size up to a maximum of 5 gpm at an RCS pressure ≥ 2215 psig and ≤ 2255 psig.	In accordance with the Surveillance Frequency Control Program

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SURVEILLANCE REQUIREMENTS

SURVEILLANCE			FREQUENCY								
SR 3.5.2.1	Verify the following valves are in the listed position with power to the valve operator removed. <table><tr><td>Number</td><td>Position</td><td>Function</td></tr><tr><td>FCV-63-1</td><td>Open</td><td>RHR Supply</td></tr><tr><td>FCV-63-22</td><td>Open</td><td>SIS Discharge</td></tr></table>	Number	Position	Function	FCV-63-1	Open	RHR Supply	FCV-63-22	Open	SIS Discharge	In accordance with the Surveillance Frequency Control Program
Number	Position	Function									
FCV-63-1	Open	RHR Supply									
FCV-63-22	Open	SIS Discharge									
SR 3.5.2.2	Verify each ECCS manual, power operated, and automatic valve in the flow path, that is not locked, sealed, or otherwise secured in position, is in the correct position.	In accordance with the Surveillance Frequency Control Program									
SR 3.5.2.3	Verify ECCS piping is full of water.	In accordance with the Surveillance Frequency Control Program									
SR 3.5.2.4	Verify each ECCS pump's developed head at the test flow point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program									
SR 3.5.2.5	Verify each ECCS automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program									

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.6.3.4	Verify the isolation time of each power operated and each automatic containment isolation valve is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.6.3.5	Perform leakage rate testing for containment purge valves with resilient seals.	In accordance with the Containment Leakage Rate Testing Program
SR 3.6.3.6	Verify each automatic containment isolation valve that is not locked, sealed, or otherwise secured in position, actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.3.7	Verify each 24 inch containment lower compartment purge supply and exhaust isolation valve is blocked to restrict the valve from opening > 50°.	In accordance with the Surveillance Frequency Control Program

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.6.1	Verify each containment spray manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.2	Verify each containment spray pump's developed head at the flow test point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.3	Verify each automatic containment spray valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.4	Verify each containment spray pump starts automatically on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.5	Verify each spray nozzle is unobstructed.	At first refueling <u>AND</u> In accordance with the Surveillance Frequency Control Program
SR 3.6.6.6	Perform SR 3.5.2.2 and SR 3.5.2.4 for the RHR spray system.	In accordance with Applicable SRs

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.1.1	-----NOTE----- Only required to be performed in MODES 1 and 2. ----- Verify each required MSSV lift setpoint per Table 3.7.1-2. Following testing, lift settings shall be within $\pm 1\%$.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and associated Completion Time of Condition C not met.	D.1 Be in MODE 3.	6 hours
	<u>AND</u> D.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.2.1 -----NOTE----- Required to be performed in MODE 3. ----- Verify closure time of each MSIV is ≤ 6.0 seconds on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. One or more MFIV or MFRV bypass valves inoperable.	C.1 Close or isolate bypass valve.	72 hours
	<u>AND</u> C.2 Verify bypass valve is closed or isolated.	Once per 7 days
D. Two valves in the same flow path inoperable.	D.1 Isolate affected flow path.	8 hours
E. Required Action and associated Completion Time not met.	E.1 Be in MODE 3.	6 hours
	<u>AND</u> E.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.3.1 Verify the isolation time of each MFIV, MFRV, and associated bypass valve is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.7.3.2 Verify each MFIV, MFRV, and associated bypass valves actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.5.1	Verify each AFW manual, power operated, and automatic valve in each water flow path, and in both steam supply flow paths to the steam turbine driven pump, that is not locked, sealed, or otherwise secured in position, is in the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.7.5.2	-----NOTE----- Not required to be performed for the turbine driven AFW pump until 24 hours after ≥ 1092 psig in the steam generator. ----- Verify the developed head of each AFW pump at the flow test point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program
SR 3.7.5.3	-----NOTE----- Not applicable in MODE 4 when steam generator is relied upon for heat removal. ----- Verify each AFW automatic valve that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.3 Offsite Dose Calculation Manual (ODCM) (continued)

- c. Shall be submitted to the NRC in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

5.7.2.4 Primary Coolant Sources Outside Containment

This program provides controls to minimize leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to levels as low as practicable. The systems include Containment Spray, Safety Injection, Residual Heat Removal, Chemical and Volume Control, Reactor Coolant System Sampling, and Waste Gas. The program shall include the following:

- a. Preventive maintenance and periodic visual inspection requirements; and
- b. Integrated leak test requirements for each system at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 are applicable.

5.7.2.5 (removed from Technical Specifications)

5.7.2.6 removed from Technical Specifications)

(continued)

5.7 Procedures, Programs, and Manuals (continued)

5.7.2.10 Reactor Coolant Pump Flywheel Inspection Program

This program shall provide for the inspection of each reactor coolant pump flywheel per the recommendations of Regulation Position c.4.b of Regulatory Guide 1.14, Revision 1, August 1975.

In lieu of Position C.4.b(1) and C.4.b(2), a qualified in-place UT examination over the volume from the inner bore of the flywheel to the circle one-half of the outer radius or a surface examination (MT and/or PT) of exposed surfaces of the removed flywheels may be conducted at 20 year intervals.

5.7.2.11 Inservice Testing Program (see Section 5.7.2.23)

This Inservice Testing Program is the licensee program that fulfills the requirements of 10 CFR 50.55a(f). The provisions of SR 3.0.3 are applicable to inservice testing activities.

(continued)

5.7 Procedures, Programs, and Manuals (continued)

5.7.2.14 Ventilation Filter Testing Program (VFTP)

A program shall be established to implement the following required testing of Engineered Safety Feature (ESF) filter ventilation systems at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and the event-based Frequencies in accordance with Regulatory Guide 1.52, Revision 2; ASME N510-1989, and the exceptions noted for each ESF system in Table 6.5 of the FSAR.

- a. Demonstrate for each of the ESF systems that an inplace test of the high efficiency particulate air (HEPA) filters shows a penetration and system bypass within acceptance criterion when tested in accordance with Regulatory Guide 1.52, Revision 2, the exceptions noted for each ESF system in Table 6.5 of the FSAR, and ASME N510-1989 at the system flowrate specified below. An alternative challenge aerosol may be used in accordance with Regulatory Guide 1.52, Revision 3, Section 6.3.

ESF VENTILATION SYSTEM	ACCEPTANCE CRITERIA	FLOW RATE
Emergency Gas Treatment	< 0.05%	4,000 cfm $\pm$ 10%
Auxiliary Building Gas Treatment	< 0.05%	9,000 cfm $\pm$ 10%
Control Room Emergency	< 1.00%	4,000 cfm $\pm$ 10%

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.15 Explosive Gas and Storage Tank Radioactivity Monitoring Program (continued)

- b. A surveillance program to ensure that the quantity of radioactivity contained in each gas storage tank is less than the amount that would result in a whole body exposure of > 0.5 rem to any individual in an unrestricted area, in the event of an uncontrolled release of the tanks' contents; and
- c. A surveillance program to ensure that the quantity of radioactivity contained in all outdoor liquid radwaste tanks that are not surrounded by liners, dikes, or walls, capable of holding the tanks' contents and that do not have tank overflows and surrounding area drains connected to the Liquid Radwaste Treatment System is less than the amount that would result in concentrations less than the limits of 10 CFR 20.1302(b)(2)(i), at the nearest potable water supply and the nearest surface water supply in an unrestricted area, in the event of an uncontrolled release of the tanks' contents.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Explosive Gas and Storage Tank Radioactivity Monitoring Program surveillance frequencies.

5.7.2.16 Diesel Fuel Oil Testing Program

A diesel fuel oil testing program to implement required testing of both new fuel oil and stored fuel oil shall be established. The program shall include sampling and testing requirements, and acceptance criteria, all in accordance with applicable ASTM Standards. The purpose of the program is to establish the following:

- a. Acceptability of new fuel oil for use prior to addition to the 7 day storage tanks by determining that the fuel oil has:
 - 1. an API gravity or an absolute specific gravity within limits,
 - 2. a flash point and kinematics viscosity within limits for ASTM 2D fuel oil, and
 - 3. a clear and bright appearance with proper color or a water and sediment content within limits;

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.16 Diesel Fuel Oil Testing Program (continued)

- b. Other properties for ASTM 2D fuel oil are within limits within 31 days following sampling and addition to the 7 day storage tanks; and
- c. Total particulate concentration of the fuel oil in each of the four interconnected tanks which constitute a 7 day storage tank is ≤ 10 mg/l when tested at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Diesel Fuel Oil Testing Program test frequencies.

5.7.2.17 (removed from Technical Specifications)

5.7.2.18 Safety Function Determination Program (SFDP)

This program ensures loss of safety function is detected and appropriate actions taken. Upon entry into LCO 3.0.6, an evaluation shall be made to determine if loss of safety function exists. Additionally, other appropriate actions may be taken as a result of the support system inoperability and corresponding exception to entering supported system Condition and Required Actions. This program implements the requirements of LCO 3.0.6. The SFDP shall contain the following:

- a. Provisions for cross train checks to ensure a loss of the capability to perform the safety function assumed in the accident analysis does not go undetected;
- b. Provisions for ensuring the plant is maintained in a safe condition if a loss of function condition exists;
- c. Provisions to ensure that an inoperable supported system's Completion Time is not inappropriately extended as a result of multiple support system inoperabilities; and
- d. Other appropriate limitations and remedial or compensatory actions.

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.19 Containment Leakage Rate Testing Program (continued)

Leakage rate acceptance criteria are:

- a. Containment overall leakage rate acceptance criterion is $\leq 1.0 L_a$. During the first unit startup following testing in accordance with this program, the leakage rate acceptance criteria are $< 0.60 L_a$ for the combined Type B and Type C tests, and $\leq 0.75 L_a$ for Type A tests.
- b. Air lock testing acceptance criteria are:
 - 1) Overall air lock leakage rate is $\leq 0.05 L_a$ when tested at $\geq P_a$.
 - 2) For each door, leakage rate is $\leq 0.01 L_a$ when pressurized to ≥ 6 psig.

The provisions of SR 3.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.

The provisions of SR 3.0.3 are applicable to the Containment Leakage Rate Testing Program.

5.7.2.20 Control Room Envelope Habitability Program

A Control Room Envelope (CRE) Habitability Program shall be established and implemented to ensure that CRE habitability is maintained such that, with an OPERABLE Control Room Emergency Ventilation System (CREVS), CRE occupants can control the reactor safely under normal conditions and maintain it in a safe condition following a radiological event, hazardous chemical release, or a smoke challenge. The program shall ensure that adequate radiation protection is provided to permit access and occupancy of the CRE under design basis accident (DBA) conditions without personnel receiving radiation exposures in excess of the applicable regulatory requirement (i.e., 5 rem Total Effective Dose Equivalent (TEDE) for a fuel handling accident or 5 rem whole body or its equivalent to any part of the body for other accidents) for the duration of the accident. The program shall include the following elements:

- a. The definition of the CRE and the CRE boundary.
- b. Requirements for maintaining the CRE boundary in its design condition including configuration control and preventive maintenance.
- c. Requirements for (i) determining the unfiltered air inleakage past the CRE boundary into the CRE at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and in accordance with the testing methods and at the event-driven Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.20 Control Room Envelope Habitability Program (continued)

- d. Measurement, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one train of the CREVS, operating at the flow rate defined in the Ventilation Filter Testing Program (VFTP), at a Frequency in accordance with the Surveillance Frequency Control Program. The results shall be trended and used as part of the periodic assessment of the CRE boundary.
- e. The quantitative limits on unfiltered air leakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air leakage measured by the testing described in paragraph c. The unfiltered air leakage limit for radiological challenges is the leakage flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air leakage limits for hazardous chemicals must ensure that exposure of CRE occupants to these hazards will be within the assumptions in the licensing basis.
- f. The provisions of SR 3.0.2 are applicable to the frequencies for assessing CRE habitability, determining CRE unfiltered leakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

5.7.2.21 Spent Fuel Storage Rack Neutron Absorber Monitoring Program

This Program provides controls for monitoring the condition of the neutron absorber used in the spent fuel pool storage racks to verify the Boron-10 areal density is consistent with the assumptions in the spent fuel pool criticality analysis. The Program shall be in accordance with NEI 16-03-A, "Guidance for Monitoring of Fixed Neutron Absorbers in Spent Fuel Pools," Revision 0, May 2017.

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.24 Online Monitoring Program

This program provides controls to determine the need for calibration of pressure, level, and flow transmitters using condition monitoring based on drift analysis. It also provides a means for in-situ dynamic response assessment using the noise analysis technique to detect failure modes that are not detectable by drift monitoring.

The Online Monitoring Program must be implemented in accordance with AMS-TR-0720R2-A, "Online Monitoring Technology to Extend Calibration Intervals of Nuclear Plant Pressure Transmitters" (proprietary version). The program shall include the following elements:

- a. Implementation of online monitoring for transmitters that have been evaluated in accordance with a NRC approved methodology during the plant operating cycle.
 - 1. Analysis of online monitoring data to identify those transmitters that require a calibration check and those that do not require a calibration check,
 - 2. Performance of online monitoring using noise analysis to assess in-situ dynamic response of transmitters that can affect response time performance,
 - 3. Calibration checks of identified transmitters no later than during the next scheduled refueling outage, and
 - 4. Documentation of the results of the online monitoring data analysis.
- b. Performance of a calibration check for any transmitter where the online monitoring was not implemented during the plant operating cycle no later than during the next scheduled refueling outage.
- c. Performance of calibration checks for transmitters at the specified backstop frequencies.
- d. The provisions of Surveillance Requirement 3.0.3 are applicable to the required calibration checks specified in items a.3, b, and c above.

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.25 Risk Informed Completion Time Program

This program provides controls to calculate a Risk Informed Completion Time (RICT) and must be implemented in accordance with NEI 06-09-A, Revision 0, "Risk-Managed Technical Specifications (RMTS) Guidelines." The program shall include the following:

- a. The RICT may not exceed 30 days;
- b. A RICT may only be utilized in MODES 1 and 2;
- c. When a RICT is being used, any change to the plant configuration, as defined in NEI 06-09-A, Appendix A, must be considered for the effect on the RICT.
 1. For planned changes, the revised RICT must be determined prior to implementation of the change in configuration
 2. For emergent conditions, the revised RICT must be determined within the time limits of the Required Action Completion Time (i.e., not the RICT) or 12 hours after the plant configuration change, whichever is less.
 3. Revising the RICT is not required If the plant configuration change would lower plant risk and would result in a longer RICT.
- d. For emergent conditions, if the extent of condition evaluation for inoperable structures, systems, or components (SSCs) is not complete prior to exceeding the Completion Time, the RICT shall account for the increased possibility of common cause failure (CCF) by either:
 1. Numerically accounting for the increased possibility of CCF in the RICT calculation; or
 2. Risk Management Actions (RMAs) not already credited in the RICT calculation shall be implemented that support redundant or diverse SSCs that perform the function(s) of the inoperable SSCs, and, if practicable, reduce the frequency of initiating events that challenge the function(s) performed by the inoperable SSCs.
- e. A RICT calculation must include the following hazard groups: internal flood and internal events PRA model, internal fire PRA model, and seismic PRA model. Changes to these means of assessing the hazard groups require prior NRC approval.
- f. The PRA models used to calculate a RICT shall be maintained and upgraded in accordance with the processes endorsed in the regulatory positions of Regulatory Guide 1.200, Revision 3, "Acceptability of Probabilistic Risk Assessment Results for Risk Informed Activities."
- g. A report shall be submitted in accordance with Specification 5.9.10 before a newly developed method is used to calculate a RICT.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-391

WATTS BAR NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 91
License No. NPF-96

1. The U.S. Nuclear Regulatory Commission (the Commission or the NRC) has found that:
 - A. The application for amendment by the Tennessee Valley Authority (TVA, the licensee) dated February 17, 2026, as supplemented by letter dated July 21, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-96 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 91 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 6 months from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

MICHAEL
MAHONEY

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MICHAEL MAHONEY
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Hipólito González, Chief
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to Facility Operating License
No. NPF-96 and the Technical
Specifications

Date of Issuance: September 25, 2026

ATTACHMENT TO AMENDMENT NO. 91
TO FACILITY OPERATING LICENSE NO. NPF-96
WATTS BAR NUCLEAR PLANT, UNIT 2
DOCKET NO. 50-391

Replace page 3 of Facility Operating License No. NPF-96 with the attached revised page 3.

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain a marginal line indicating the area of change.

<u>Remove</u>	<u>Insert</u>
1.1-7	1.1-7
3.4-19	3.4-19
3.4-32	3.4-32
3.5-4	3.5-4
3.6-12	3.6-12
3.6-18	3.6-18
3.7-2	3.7-2
3.7-5	3.7-5
3.7-7	3.7-7
3.7-12	3.7-12
5.0-10	5.0-10
5.0-14	5.0-14
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5.0-23	5.0-23
5.0-24	5.0-24
5.0-27	5.0-27
5.0-27b	5.0-27b
5.0-27c	5.0-27c
5.0-27d	5.0-27d
5.0-27e	5.0-27e

- C. The license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act, and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified or incorporated below.

(1) Maximum Power Level

TVA is authorized to operate the facility at reactor core power levels not in excess of 3459 megawatts thermal.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 91 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- (3) TVA shall implement permanent modifications to prevent overtopping of the embankments of the Fort Loudon Dam due to the Probable Maximum Flood by June 30, 2018.
- (4) FULL SPECTRUM LOCA Methodology shall be implemented when the WBN Unit 2 steam generators are replaced with steam generators equivalent to the existing steam generators at WBN Unit 1.
- (5) By December 31, 2019, the licensee shall report to the NRC that the actions to resolve the issues identified in Bulletin 2012-01, "Design Vulnerability in Electrical Power System," have been implemented.
- (6) The licensee shall maintain in effect the provisions of the physical security plan, security personnel training and qualification plan, and safeguards contingency plan, and all amendments made pursuant to the authority of 10 CFR 50.90 and 50.54(p).
- (7) TVA shall fully implement and maintain in effect all provisions of the Commission approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The TVA approved CSP was discussed in NUREG-0847, Supplement 28, as amended by changes approved in License Amendment No. 7.
- (8) TVA shall implement and maintain in effect all provisions of the approved fire protection program as described in the Fire Protection Report for the facility, as described in NUREG-0847, Supplement 29, subject to the following provision:

1.1 Definitions (continued)

THERMAL POWER	THERMAL POWER shall be the total reactor core heat transfer rate to the reactor coolant.
TRIP ACTUATING DEVICE OPERATIONAL TEST (TADOT)	A TADOT shall consist of operating the trip actuating device and verifying the OPERABILITY of all devices in the channel required for trip actuating device OPERABILITY. The TADOT shall include adjustment, as necessary, of the trip actuating device so that it actuates at the required setpoint within the necessary accuracy. The TADOT may be performed by means of any series of sequential, overlapping, or total channel steps, and each step must be performed within the Frequency in the Surveillance Frequency Control Program for the devices included in the step.

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.10.1	Verify each pressurizer safety valve is OPERABLE with lift settings ≥ 2410 psig and ≤ 2560 psig. Following testing, lift settings shall be within $\pm 1\%$ of the nominal lift setting of 2485 psig.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.14.1	-----NOTE----- Not required to be performed in MODES 3 and 4. ----- Verify leakage from each RCS PIV is equivalent to ≤ 0.5 gpm per nominal inch of valve size up to a maximum of 5 gpm at an RCS pressure ≥ 2215 psig and ≤ 2255 psig.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY									
SR 3.5.2.1	Verify the following valves are in the listed position with power to the valve operator removed. <table> <tr> <th><u>Number</u></th><th><u>Position</u></th><th><u>Function</u></th></tr> <tr> <td>2-FCV-63-1</td><td>Open</td><td>RHR Supply</td></tr> <tr> <td>2-FCV-63-22</td><td>Open</td><td>SIS Discharge</td></tr> </table>	<u>Number</u>	<u>Position</u>	<u>Function</u>	2-FCV-63-1	Open	RHR Supply	2-FCV-63-22	Open	SIS Discharge	In accordance with the Surveillance Frequency Control Program
<u>Number</u>	<u>Position</u>	<u>Function</u>									
2-FCV-63-1	Open	RHR Supply									
2-FCV-63-22	Open	SIS Discharge									
SR 3.5.2.2	Verify each ECCS manual, power operated, and automatic valve in the flow path, that is not locked, sealed, or otherwise secured in position, is in the correct position.	In accordance with the Surveillance Frequency Control Program									
SR 3.5.2.3	Verify ECCS piping is full of water.	In accordance with the Surveillance Frequency Control Program									
SR 3.5.2.4	Verify each ECCS pump's developed head at the test flow point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program									
SR 3.5.2.5	Verify each ECCS automatic valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program									
SR 3.5.2.6	Verify each ECCS pump starts automatically on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program									

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.3.1	Verify each containment purge valve is closed, except when the containment purge valves are open for pressure control, ALARA or air quality considerations for personnel entry, or for Surveillances that require the valves to be open.	In accordance with the Surveillance Frequency Control Program
SR 3.6.3.2	-----NOTE----- Valves and blind flanges in high radiation areas may be verified by use of administrative controls. ----- Verify each containment isolation manual valve and blind flange that is located outside containment, the containment annulus, and the Main Steam Valve Vault Rooms, and not locked, sealed, or otherwise secured and required to be closed during accident conditions is closed, except for containment isolation valves that are open under administrative controls.	In accordance with the Surveillance Frequency Control Program
SR 3.6.3.3	-----NOTE----- Valves and blind flanges in high radiation areas may be verified by use of administrative controls. ----- Verify each containment isolation manual valve and blind flange that is located inside containment, the containment annulus, and the Main Steam Valve Vault Rooms, and not locked, sealed, or otherwise secured and required to be closed during accident conditions is closed, except for containment isolation valves that are open under administrative controls.	Prior to entering MODE 4 from MODE 5 if not performed within the previous 92 days
SR 3.6.3.4	Verify the isolation time of each power operated and each automatic containment isolation valve is within limits.	In accordance with the Surveillance Frequency Control Program

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.6.1	Verify each containment spray manual, power operated, and automatic valve in the flow path that is not locked, sealed, or otherwise secured in position is in the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.2	Verify each containment spray pump's developed head at the flow test point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.3	Verify each automatic containment spray valve in the flow path that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.4	Verify each containment spray pump starts automatically on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.6.6.5	Verify each spray nozzle is unobstructed.	At first refueling <u>AND</u> In accordance with the Surveillance Frequency Control Program
SR 3.6.6.6	Perform SR 3.5.2.2 and SR 3.5.2.4 for the RHR spray system.	In accordance with Applicable SRs

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time not met. <u>OR</u> One or more steam generators with ≥ 4 MSSVs inoperable.	C.1 Be in MODE 3.	6 hours
	<u>AND</u> C.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.1.1 -----NOTE----- Only required to be performed in MODES 1 and 2. ----- Verify each required MSSV lift setpoint per Table 3.7.1-2. Following testing, lift settings shall be within $\pm 1\%$.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and associated Completion Time of Condition C not met.	D.1 Be in MODE 3.	6 hours
	<u>AND</u>	
	D.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.7.2.1 -----NOTE----- Required to be performed in MODES 1 & 2. ----- Verify closure time of each MSIV is ≤ 6.0 seconds on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Two valves in the same flow path inoperable.	D.1 Isolate affected flow path.	8 hours
E. Required Action and associated Completion Time not met.	E.1 Be in MODE 3. <u>AND</u>	6 hours
	E.2 Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.3.1	Verify the isolation time of each MFIV, MFRV, and associated bypass valve is within limits.	In accordance with the Surveillance Frequency Control Program
SR 3.7.3.2	Verify each MFIV, MFRV, and associated bypass valves actuates to the isolation position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.7.5.1	Verify each AFW manual, power operated, and automatic valve in each water flow path, and in both steam supply flow paths to the steam turbine driven pump, that is not locked, sealed, or otherwise secured in position, is in the correct position.	In accordance with the Surveillance Frequency Control Program
SR 3.7.5.2	-----NOTE----- Not required to be performed for the turbine driven AFW pump until 24 hours after ≥ 1092 psig in the steam generator. ----- Verify the developed head of each AFW pump at the flow test point is greater than or equal to the required developed head.	In accordance with the Surveillance Frequency Control Program
SR 3.7.5.3	-----NOTE----- Not applicable in MODE 4 when steam generator is relied upon for heat removal. ----- Verify each AFW automatic valve that is not locked, sealed, or otherwise secured in position, actuates to the correct position on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program
SR 3.7.5.4	-----NOTE----- - Not required to be performed for the turbine driven AFW pump until 24 hours after ≥ 1092 psig in the steam generator. - Not applicable in MODE 4 when steam generator is relied upon for heat removal. ----- Verify each AFW pump starts automatically on an actual or simulated actuation signal.	In accordance with the Surveillance Frequency Control Program

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.3 Offsite Dose Calculation Manual (ODCM) (continued)

- c. Shall be submitted to the NRC in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Radioactive Effluent Release Report for the period of the report in which any change in the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (i.e., month and year) the change was implemented.

5.7.2.4 Primary Coolant Sources Outside Containment

This program provides controls to minimize leakage from those portions of systems outside containment that could contain highly radioactive fluids during a serious transient or accident to levels as low as practicable. The systems include Containment Spray, Safety Injection, Residual Heat Removal, Chemical and Volume Control, Reactor Coolant System Sampling, and Waste Gas. The program shall include the following:

- a. Preventive maintenance and periodic visual inspection requirements; and
- b. Integrated leak test requirements for each system at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 are applicable.

5.7.2.5 Reserved for Future Use

5.7.2.6 Reserved for Future Use

(continued)

5.7 Procedures, Programs, and Manuals (continued)

5.7.2.11 Inservice Testing Program (see Section 5.7.2.23)

The Inservice Testing Program is the licensee program that fulfills the requirements of 10 CFR 50.55a(f). The provisions of SR 3.0.3 are applicable to inservice testing activities.

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.14 Ventilation Filter Testing Program (VFTP)

A program shall be established to implement the following required testing of Engineered Safety Feature (ESF) filter ventilation systems at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and the event-based Frequencies in accordance with Regulatory Guide 1.52, Revision 2; ASME N510-1989, and the exceptions noted for each ESF system in Tables 6.5-1, 6.5-2, and 6.5-4 of the FSAR.

- a. Demonstrate for each of the ESF systems that an inplace test of the high efficiency particulate air (HEPA) filters shows a penetration and system bypass within acceptance criterion when tested in accordance with Regulatory Guide 1.52, Revision 2, the exceptions noted for each ESF system in Tables 6.5-1, 6.5-2, and 6.5-4 of the FSAR, and ASME N510-1989 at the system flowrate specified below. An alternative challenge aerosol may be used in accordance with Regulatory Guide 1.52, Revision 3, Section 6.3.

ESF VENTILATION SYSTEM	ACCEPTANCE CRITERIA	FLOW RATE
Emergency Gas Treatment	< 0.05%	4,000 cfm $\pm$ 10%
Auxiliary Building Gas Treatment	< 0.05%	9,000 cfm $\pm$ 10%
Control Room Emergency	< 1.00%	4,000 cfm $\pm$ 10%

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.15 Explosive Gas and Storage Tank Radioactivity Monitoring Program
(continued)

- b. A surveillance program to ensure that the quantity of radioactivity contained in each gas storage tank is less than the amount that would result in a whole body exposure of > 0.5 rem to any individual in an unrestricted area, in the event of an uncontrolled release of the tanks' contents; and
- c. A surveillance program to ensure that the quantity of radioactivity contained in all outdoor liquid radwaste tanks that are not surrounded by liners, dikes, or walls, capable of holding the tanks' contents and that do not have tank overflows and surrounding area drains connected to the Liquid Radwaste Treatment System is less than the amount that would result in concentrations less than the limits of 10 CFR 20.1302(b)(2)(i), at the nearest potable water supply and the nearest surface water supply in an unrestricted area, in the event of an uncontrolled release of the tanks' contents.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Explosive Gas and Storage Tank Radioactivity Monitoring Program surveillance frequencies.

5.7.2.16 Diesel Fuel Oil Testing Program

A diesel fuel oil testing program to implement required testing of both new fuel oil and stored fuel oil shall be established. The program shall include sampling and testing requirements, and acceptance criteria, all in accordance with applicable ASTM Standards. The purpose of the program is to establish the following:

- a. Acceptability of new fuel oil for use prior to addition to the 7 day storage tanks by determining that the fuel oil has:
 - 1. an API gravity or an absolute specific gravity within limits,
 - 2. a flash point and kinematics viscosity within limits for ASTM 2D fuel oil, and
 - 3. a clear and bright appearance with proper color or a water and sediment content within limits;

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.16 Diesel Fuel Oil Testing Program (continued)

- b. Other properties for ASTM 2D fuel oil are within limits within 31 days following sampling and addition to the 7 day storage tanks; and
- c. Total particulate concentration of the fuel oil in each of the four interconnected tanks which constitute a 7 day storage tank is ≤ 10 mg/l when tested at a Frequency in accordance with the Surveillance Frequency Control Program.

The provisions of SR 3.0.2 and SR 3.0.3 are applicable to the Diesel Fuel Oil Testing Program test frequencies.

5.7.2.17 (removed from Technical Specifications)

5.7.2.18 Safety Function Determination Program (SFDP)

This program ensures loss of safety function is detected and appropriate actions taken. Upon entry into LCO 3.0.6, an evaluation shall be made to determine if loss of safety function exists. Additionally, other appropriate actions may be taken as a result of the support system inoperability and corresponding exception to entering supported system Condition and Required Actions. This program implements the requirements of LCO 3.0.6. The SFDP shall contain the following:

- a. Provisions for cross train checks to ensure a loss of the capability to perform the safety function assumed in the accident analysis does not go undetected;
- b. Provisions for ensuring the plant is maintained in a safe condition if a loss of function condition exists;
- c. Provisions to ensure that an inoperable supported system's Completion Time is not inappropriately extended as a result of multiple support system inoperabilities; and
- d. Other appropriate limitations and remedial or compensatory actions.

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.20 Control Room Envelope Habitability Program (continued)

- c. Requirements for (i) determining the unfiltered air leakage past the CRE boundary into the CRE at the periodic Frequencies in accordance with the Surveillance Frequency Control Program and in accordance with the testing methods and at the event-driven Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, "Demonstrating Control Room Envelope Integrity at Nuclear Power Reactors," Revision 0, May 2003, and (ii) assessing CRE habitability at the Frequencies specified in Sections C.1 and C.2 of Regulatory Guide 1.197, Revision 0.
- d. Measurement, at designated locations, of the CRE pressure relative to all external areas adjacent to the CRE boundary during the pressurization mode of operation by one train of the CREVS, operating at the flow rate defined in the Ventilation Filter Testing Program (VFTP), at a Frequency in accordance with the Surveillance Frequency Control Program. The results shall be trended and used as part of the periodic assessment of the CRE boundary.
- e. The quantitative limits on unfiltered air leakage into the CRE. These limits shall be stated in a manner to allow direct comparison to the unfiltered air leakage measured by the testing described in paragraph c. The unfiltered air leakage limit for radiological challenges is the leakage flow rate assumed in the licensing basis analyses of DBA consequences. Unfiltered air leakage limits for hazardous chemicals must ensure that exposure of CRE occupants to these hazards will be within the assumptions in the licensing basis.
- f. The provisions of SR 3.0.2 are applicable to the frequencies for assessing CRE habitability, determining CRE unfiltered leakage, and measuring CRE pressure and assessing the CRE boundary as required by paragraphs c and d, respectively.

5.7.2.21 Spent Fuel Storage Rack Monitoring Program

This Program provides controls for monitoring the condition of the neutron absorber used in the spent fuel pool storage racks to verify the Boron-10 areal density is consistent with the assumptions in the spent fuel pool criticality analysis. The Program shall be in accordance with NEI 16-03-A, "Guidance for Monitoring of Fixed Neutron Absorbers in Spent Fuel Pools," Revision 0, May 2017.

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.22 Battery Monitoring and Maintenance Program (continued)

3. Actions to equalize and test battery cells that had been discovered with electrolyte level below the top of the plates;
4. Limits on average electrolyte temperature, battery connection resistance, and battery terminal voltage; and
5. A requirement to obtain specific gravity readings of all cells at each discharge test, consistent with manufacturer recommendations.

5.7.2.23 Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies and the frequencies of other periodic testing required by these Technical Specifications (herein also referred to as Surveillance Frequencies). The program shall ensure that Surveillance Requirements and other required testing specified in these Technical Specifications are performed at intervals sufficient to assure the associated Limiting Conditions for Operation and other requirements are met.

- a. The Surveillance Frequency Control Program shall contain a list of Frequencies controlled by the programs that govern the activities listed in items b.1, b.2, and b.3.
- b. Changes to the Frequencies listed in the Surveillance Frequency Control Program shall be made:
 1. In accordance with the requirements of 10 CFR 50.55a for the Inservice Testing Program required by 10 CFR 50.55a(f);
 2. In accordance with the requirements of 10 CFR 50.69(d)(2) for testing permitted under 10 CFR 50.69(b)(1)(v);
 3. Otherwise, in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1, or as specifically approved by the NRC.

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.23 Surveillance Frequency Control Program (continued)

- c. The provisions of Surveillance Requirements 3.0.2 and 3.0.3 are applicable to the Frequencies established in the Surveillance Frequency Control Program, except:
 - 1. The SR 3.0.2 test interval extensions for components in the Inservice Testing Program cannot exceed those allowed by the ASME OM Code as incorporated by reference in 10 CFR 50.55a; and
 - 2. Surveillance Requirement 3.0.3 is only applicable to other required testing when invoked by a Chapter 5 Specification.

5.7.2.24 Online Monitoring Program

This program provides controls to determine the need for calibration of pressure, level, and flow transmitters using condition monitoring based on drift analysis. It also provides a means for in-situ dynamic response assessment using the noise analysis technique to detect failure modes that are not detectable by drift monitoring.

The Online Monitoring Program must be implemented in accordance with AMS-TR-0720R2-A, "Online Monitoring Technology to Extend Calibration Intervals of Nuclear Plant Pressure Transmitters" (proprietary version). The program shall include the following elements:

- a. Implementation of online monitoring for transmitters that have been evaluated in accordance with an NRC approved methodology during the plant operating cycle.
 - 1. Analysis of online monitoring data to identify those transmitters that require a calibration check and those that do not require a calibration check,
 - 2. Performance of online monitoring using noise analysis to assess in-situ dynamic response of transmitters that can affect response time performance,
 - 3. Calibration checks of identified transmitters no later than during the next scheduled refueling outage, and
 - 4. Documentation of the results of the online monitoring data analysis.

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.24 Online Monitoring Program (continued)

- b. Performance of a calibration check for any transmitter where the online monitoring was not implemented during the plant operating cycle no later than during the next scheduled refueling outage.
- c. Performance of calibration checks for transmitters at the specified backstop frequencies.
- d. The provisions of Surveillance Requirement 3.0.3 are applicable to the required calibration checks specified in items a.3, b, and c above.

5.7.2.25 Risk Informed Completion Time Program

This program provides controls to calculate a Risk Informed Completion Time (RICT) and must be implemented in accordance with NEI 06-09-A, Revision 0, "Risk-Managed Technical Specifications (RMTS) Guidelines." The program shall include the following:

- a. The RICT may not exceed 30 days;
- b. A RICT may only be utilized in MODES 1 and 2;
- c. When a RICT is being used, any change to the plant configuration, as defined in NEI 06-09-A, Appendix A, must be considered for the effect on the RICT
 - 1. For planned changes, the revised RICT must be determined prior to implementation of the change in configuration.
 - 2. For emergent conditions, the revised RICT must be determined within the time limits of the Required Action Completion Time (i.e., not the RICT) or 12 hours after the plant configuration change, whichever is less.
 - 3. Revising the RICT is not required if the plant configuration change would lower plant risk and would result in a longer RICT.

(continued)

5.7 Procedures, Programs, and Manuals

5.7.2.25 Risk Informed Completion Time Program (continued)

- d. For emergent conditions, if the extent of condition evaluation for inoperable structures, systems, or components (SSCs) is not complete prior to exceeding the Completion Time, the RICT shall account for the increased possibility of common cause failure (CCF) by either:
 - 1. Numerically accounting for the increased possibility of CCF in the RICT calculation; or
 - 2. Risk Management Actions (RMAs) not already credited in the RICT calculation shall be implemented that support redundant or diverse SSCs that perform the function(s) of the inoperable SSCs, and, if practicable, reduce the frequency of initiating events that challenge the function(s) performed by the inoperable SSCs.
 - e. A RICT calculation must include the following hazard groups: internal flood and internal events PRA model, internal fire PRA model, and seismic PRA model. Changes to these means of assessing the hazard groups require prior NRC approval.
 - f. The PRA models used to calculate a RICT shall be maintained and upgraded in accordance with the processes endorsed in the regulatory positions of Regulatory Guide 1.200, Revision 3, "Acceptability of Probabilistic Risk Assessment Results for Risk Informed Activities."
 - g. A report shall be submitted in accordance with Specification 5.9.10 before a newly developed method is used to calculate a RICT.
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UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
RELATED TO AMENDMENT NO. 346 TO
SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-33
AMENDMENT NO. 369 TO
SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-52
AMENDMENT NO. 329 TO
SUBSEQUENT RENEWED FACILITY OPERATING LICENSE NO. DPR-68
AMENDMENT NO. 380 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-77
AMENDMENT NO. 375 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-79
AMENDMENT NO. 185 TO FACILITY OPERATING LICENSE NO. NPF-90
AMENDMENT NO. 90 TO FACILITY OPERATING LICENSE NO. NPF-96
TENNESSEE VALLEY AUTHORITY
BROWNS FERRY NUCLEAR PLANT, UNITS 1, 2, AND 3
SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2
WATTS BAR NUCLEAR PLANT, UNITS 1 AND 2
DOCKET NOS. 50-259, 50-260, 50-296, 50-327, 50-328, 50-390, AND 50-391

1.0 INTRODUCTION

By application dated February 17, 2026 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML26048A396), as supplemented by letter dated July 21, 2026 (ML26202A344), the Tennessee Valley Authority (TVA or the licensee) requested changes to the technical specifications (TSs) for Browns Ferry Nuclear Plant, Units 1, 2, and 3 (BFN), Sequoyah Nuclear Plant, Units 1 and 2 (SQN), and Watts Bar Nuclear Plant, Units 1 and 2 (Watts Bar or WBN). In its application, TVA requested that the U.S. Nuclear Regulatory Commission (NRC, the Commission) process the proposed license amendment request (LAR) under the Consolidated Line Item Improvement Process.

The proposed amendments would revise the BFN, SQN and WBN TSs based on Technical Specifications Task Force (TSTF) Traveler TSTF-596, Revision 2, "Expand the Applicability of the Surveillance Frequency Control Program (SFCP)" (TSTF-596) (ML24282B020) and the associated NRC staff safety evaluation (SE) of TSTF-596, Revision 2 (ML24362A056). These proposed changes are evaluated in section 3.1 of this SE.

The proposed changes would:

- (1) expand the applicability of the SFCP to include other periodic testing frequencies in the "Programs and Manuals" TS section;
- (2) revise the SFCP to reference additional regulatory mechanisms that may be used to control surveillance frequencies (i.e., Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.55a, "Codes and standards"; and 10 CFR 50.69, "Risk-informed categorization and treatment of structures, systems and components for nuclear power reactors"); and
- (3) revise surveillance requirements (SRs) that reference the Inservice Testing (IST) Program to instead reference the SFCP or to describe the required test.

TVA also requested changes that are only applicable to WBN. The proposed amendments for WBN would revise its TSs to adopt changes in accordance with TSTF-374, Revision 0, "Revision to TS 5.5.13 and associated TS Bases for Diesel Fuel Oil" (ML011340449); TSTF-118, Revision 0, "Administrative Controls Program Exceptions" (ML040480060); and TSTF-600, Revision 2, "Revise the Reactor Coolant System (RCS) Pressure Isolation Valve (PIV) Leakage Testing Frequency" (ML25083A153). The NRC staff's SE of Traveler TSTF-374 is documented in a letter dated January 13, 2005 (ML050130309), and the SE of Traveler TSTF-600 is documented in a letter dated September 8, 2025 (ML25247A240). These proposed changes are evaluated in section 3.2 of this SE.

The proposed changes would:

- (1) add the provisions of SR 3.0.2 and SR 3.0.3 to TS 5.7.2.16 "Diesel Fuel Oil Testing Program" per TSTF-118;
- (2) relocate a reference to the American Society of Testing and Materials (ASTM) standards from TS 5.7.2.16 to the TS Bases and add an alternate criterion for new fuel oil acceptance per TSTF-374; and
- (3) revise the frequency for SR 3.4.14.1 in TS 3.4.14, "RCS Pressure Isolation Valve (PIV) Leakage," to eliminate time- and event-based frequencies for PIV testing per TSTF-600. The frequency for SR 3.4.14.1 would be "In accordance with the Surveillance Frequency Control Program" per TSTF-596 as discussed in section 2.2 of the LAR and evaluated in section 3.2 of this SE.

A notice of consideration of proposed issuance of amendments with proposed no significant hazards consideration (NSHC) determinations was published in the *Federal Register* on April 14, 2026 (91 FR 19207), and there has been no public comment on such determination. The supplemental letter dated July 21, 2026, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed NSHC determination as published in this notice.

2.0 REGULATORY EVALUATION

2.1 System or Program Description

2.1.1 Description of the Surveillance Frequency Control Program (TSTF-596)

The SFCP is a program that controls the frequencies of TS SRs for structures, systems, and components (SSCs). The SFCP uses a risk-informed process to allow licensees to optimize the frequency of surveillance activities. The goal of the SFCP is to reduce the number of surveillances performed which can reduce the risk of plant transients. The SFCP is part of the "Administrative Controls" TS section. Changes to surveillance frequencies are made in accordance with Nuclear Energy Institute (NEI) 04-10, "Risk-Informed Technical Specifications Initiative 5b, Risk-Informed Method for Control of Surveillance Frequencies," Revision 1 (ML071360456).

2.1.2 System Description (TSTF-118, TSTF-374, and TSTF-600)

2.1.2.1 Diesel Generator Fuel Oil Testing Program (TSTF-118 and TSTF-374)

The Diesel Generator (DG) Fuel Oil Testing Program (WBN TS 5.7.2.16) is a TS program that contains the test requirements for maintaining the quality of the DG fuel oil to ensure that the DGs can perform their safety functions when required. The program refers to ASTM test standards for required testing and provides criteria for acceptable test results.

2.1.2.2 Reactor Coolant System Pressure Isolation Valves (TSTF-600)

The RCS PIVs are two valves in series that isolate the high-pressure RCS from lower pressure systems. The purpose of the valves is to prevent RCS fluid from over pressurizing piping that is not designed for the high RCS pressure. This is intended to prevent an intersystem loss-of-coolant accident (ISLOCA) which could release RCS fluid outside of containment. The TS Limiting Condition for Operation (LCO) for PIVs limits this leakage to amounts that do not compromise safety. Each PIV has a leakage limit. In addition, leakage through two PIVs associated with a single penetration is included as "identified leakage" as defined in the RCS Operational Leakage TS. All RCS PIVs have testing required by both the TS and the American Society of Mechanical Engineers (ASME) *Operation and Maintenance of Nuclear Power Plants* (OM Code), IST Program.

2.2 Requested Changes

2.2.1 Proposed TS Changes to Adopt TSTF-596

2.2.1.1 Proposed Changes to the SFCP

In accordance with TSTF-596, TVA proposed changes that would revise the "Surveillance Frequency Control Program," which is found at TS 5.5.15 for BFN, TS 5.5.17 for SQN, and TS 5.7.2.23 for WBN.

Specifically, TVA proposed the following SFCP changes to adopt TSTF-596 (added text shown in **bold**, deleted text shown in ~~**bold**~~). Note, TVA proposed variations to the SFCP as outlined in TSTF-596 and these variations are captured here in square brackets shown in **[bold text or strikethrough]** for ease of presentation.

Surveillance Frequency Control Program

This program provides controls for Surveillance Frequencies **and the frequencies of other periodic testing required by these Technical Specifications (herein also referred to as Surveillance Frequencies)**. The program shall ensure that Surveillance Requirements **and other required testing** specified in these Technical Specifications are performed at intervals sufficient to assure the associated Limiting Conditions for Operation **and other requirements** are met.

- a. The Surveillance Frequency Control Program shall contain a list of Frequencies ~~[of those Surveillance Requirements for which the Frequency is]~~ controlled by the program **[s that govern the activities listed in items b.1, b.2 and b.3.]**
- b. Changes to the Frequencies listed in the Surveillance Frequency Control Program shall be made:
 1. **In accordance with the requirements of 10 CFR 50.55a for [the Inservice Testing Program][testing] required by 10 CFR 50.55a(f);**
 2. **In accordance with the requirements of 10 CFR 50.69(d)(2) for testing permitted under 10 CFR 50.69(b)(1)(v)[in lieu of testing required by 10 CFR 50.55a(f)];**
 3. **Otherwise, in accordance with NEI 04-10, "Risk-Informed Method for Control of Surveillance Frequencies," Revision 1[~~2~~], or as specifically approved by the NRC.]**
- c. The provisions of Surveillance Requirements 3.0.2 and 3.0.3 are applicable to the Frequencies established in the Surveillance Frequency Control Program, **except:**
 1. **[The SR 3.0.2 test interval extensions for components in the Inservice Testing Program cannot exceed those allowed by the ASME OM Code as incorporated by reference in 10 CFR 50.55a; ~~For testing required by 10 CFR 50.55a(f), Surveillance Requirement 3.0.2 is only applicable as permitted by that regulation];~~ and**
 2. **Surveillance Requirement 3.0.3 is only applicable to other required testing when invoked by a Chapter 5 Specification.**

The SFCP has been incorporated into several SR test frequencies as reflected in the TS markups (attachments 1 through 7 to the LAR, and enclosure 2 of the supplemental letter dated July 21, 2026).

2.2.1.2 Proposed Changes Related to the Inservice Testing Program

In accordance with the NRC staff-approved TSTF-596, TVA proposed the following changes related to how the IST program is referenced:

- All SR test frequencies that state “In accordance with the Inservice Testing Program” would be revised to state “In accordance with the Surveillance Frequency Control Program.”
- All SRs that reference the IST Program would be revised to replace the reference with a description of the required test.

2.2.1.3 Proposed Changes to Periodic Testing Requirements in Section 5.5 Programs

In accordance with TSTF-596, TVA proposed that the periodic testing frequencies required by four programs in the “Program and Manuals” TS section be revised to reference the SFCP. The bracketed number will change based on the plant design (i.e., Westinghouse (shown) or General Electric boiling water reactor type 4 (BWR/4)). Note, for these programs, the WBN TSs do not follow the standard numbering convention.

- TS 5.5.2 “Primary Coolant Sources Outside Containment”
- TS 5.5.[10] “Ventilation Filter Testing Program (VFTP)”
- TS 5.5.[12] “Diesel Fuel Oil Testing Program”
- TS 5.5.[17] “Control Room Envelope (CRE) Habitability Program”

The periodic testing frequencies for these programs would be revised to state that they are tested at a Frequency in accordance with the Surveillance Frequency Control Program.

2.2.1.4 Proposed Variations to TSTF-596

In general, the TS variations that TVA described in the LAR and supplemental letter, in some cases, have different TS numbers, titles, language, or content as compared to TSTF-596. In addition, there are instances where no plant-specific change is needed because the TS is already consistent with TSTF-596 or the plant does not have a similar requirement to the Standard Technical Specifications (STS) on which TSTF-596 is based. There are also cases where a plant-specific change is applicable but does not exist in the STS on which TSTF-596 is based. In other instances, all uses of a defined term are eliminated; therefore, the term is removed from the definitions section of TSs. Finally, in some cases, TVA proposed variations to avoid numerous procedure changes in the IST Program. The proposed variations are dispositioned in SE section 3.1.1.

TVA proposed plant-specific variations regarding TSTF-596 adoption that can be found in the enclosure to the LAR, Section 2.2, “Variations for TSTF-596.”

- For BFN, refer to LAR enclosure pages E2 through E4.
- For SQN, refer to LAR enclosure pages E4 through E5.
- For WBN, refer to LAR enclosure pages E5 through E6.

TVA also proposed additional changes regarding TSTF-596 that can be found in the enclosure to the LAR, Section 2.3, "Additional Changes Regarding TSTF-596."

- For BFN, SQN, and WBN, refer to LAR enclosure pages E7 through E8

Additionally, in the supplemental letter, TVA proposed TS changes in response to requests for additional information. Of note, TVA proposed changes to the SFCP TSs for all three sites (see SFCP proposed changes in SE section 2.2.1.1 above)

2.2.2 Proposed TS Changes to Adopt TSTF-118

In accordance with the NRC staff-approved TSTF-118, TVA proposed changes that would revise WBN TS 5.7.2.16, "Diesel Fuel Oil Testing Program." Specifically, TVA proposed adding the following at the end of the TS:

The provisions of SR 3.0.2 and 3.0.3 are applicable to the Diesel Fuel Oil Testing Program.

2.2.2.1 Proposed Variations to TSTF-118

Editorial Variations

Traveler TSTF-118 revises the current Westinghouse STS 5.5.12. The equivalent WBN TS that is proposed for revision is TS 5.7.2.16.

Traveler TSTF-118 is also applicable to current Westinghouse STS 5.5.8. "Steam Generator Tube Surveillance Program"; however, that change was removed from the STS by TSTF-449, Revision 4, "Steam Generator Tube Integrity" (ML051090200). Therefore, only the change to WBN TS 5.7.2.16 is proposed for adoption.

Other Variations

TVA did not cite other variations for TSTF-118.

2.2.3 Proposed Changes to Adopt TSTF-374

In accordance with the NRC staff-approved TSTF-374, TVA proposed changes that would revise WBN TS 5.7.2.16. Specifically, TVA proposed the addition of an acceptance criterion that may be used in place of the current requirement for the appearance of diesel fuel oil. The new criterion allows the fuel to have a water and sediment content within limits. In addition, TVA proposed changes to the Bases for SR 3.8.3.3 to provide explanatory information for the change to TS 5.7.2.16. The change to the Bases also adds or revises ASTM standards for test methods and acceptance criteria for diesel fuel oil. TVA stated that the ASTM standards being proposed are either already specified in plant procedures or will be added as part of the implementation of the amendments.

2.2.3.1 Proposed Variations to TSTF-374

Editorial Variations

TSTF-374 revises the current Westinghouse STS 5.5.12. The equivalent WBN TS that is proposed for revision is TS 5.7.2.16.

Other Variations

TVA proposed using different ASTM standards than those specified in the traveler.

One of the ASTM standard variations is the replacement of ASTM D2276, "Standard Test Method for Particulate Contaminant in Aviation Fuel by Line Sampling," which is currently cited for replacement in the traveler, with ASTM D6217, "Standard Test Method for Particulate Contamination in Middle Distillate Fuels by Laboratory Filtration." This is identified as a variation because TSTF-374 specifies ASTM D5452, "Standard Test Method for Particulate Contamination in Aviation Fuels by Laboratory Filtration," as the replacement standard. TVA cited a previous NRC approval of use of ASTM 6217 as an appropriate standard during the SQN conversion to STS (ML13330A922).

2.2.4 Proposed Changes to Adopt TSTF-600

In accordance with TSTF-600, TVA proposed changes that would revise the frequency for WBN TS SR 3.4.14.1, for RCS PIV testing, to be "In accordance with the Surveillance Frequency Control Program." TVA also proposed to remove Notes 2 and 3 from SR 3.4.14.1 per the traveler.

In its LAR, TVA stated, in part:

TVA confirms that the RCS PIVs required to be tested by the WBN Units 1 and 2 TS are included in the WBN Units 1 and 2 IST Program.

TVA confirms that an Intersystem Loss-of-Coolant Accident (ISLOCA) is not a risk-significant contributor to core damage or large early release in the plant's probabilistic risk assessment evaluation, and that the event-driven and 9-month Frequency RCS PIV testing is not necessary to ensure the risk associated with an ISLOCA is acceptable.

TVA confirms that the RCS PIV leakage testing Frequencies proposed to be removed from the WBN Units 1 and 2 TS are not credited for satisfying any other requirements described in the Updated Final Safety Analysis Report or any commitments for reasons other than being a TS requirement.

2.2.4.1 Proposed Variations to TSTF-600

Editorial Variations

No editorial variations were identified for TSTF-600.

Other Variations

TVA is adopting TSTF-596 into the WBN TSs. These changes are evaluated in section 3.1 of this SE. As noted in TSTF-600, Section 2.4, "Description of the Proposed Change," the Frequency for SR 3.4.14.1 may be revised to state, "In accordance with the Surveillance Frequency Control Program," which incorporates the SFCP as allowed by TSTF-596. Therefore, TVA has proposed the frequency for SR 3.4.14.1 to be in accordance with the SFCP instead of in accordance with the Inservice Testing Program as denoted in the TSTF-600 TS markup.

2.3 Regulatory Requirements

As described in 10 CFR 50.92(a), in determining whether an amendment to a license will be issued to the applicant, the Commission will be guided by the considerations which govern the issuance of initial licenses to the extent applicable and appropriate. The general considerations that guide the Commission include, as stated in 10 CFR 50.40(a), how the TSs, in combination with other listed documents, provide reasonable assurance that the health and safety of the public will not be endangered. Also, to issue an operating license, of which TSs are a part, the Commission must make the findings of 10 CFR 50.57, "Issuance of operating license," including the 10 CFR 50.57(a)(3)(i) finding that there is reasonable assurance that the activities authorized by the operating license can be conducted without endangering the health and safety of the public.

Section 182(a) of the Atomic Energy Act of 1954, as amended, (AEA) requires applicants for nuclear power plant operating licenses to include TSs as part of the application. Section 182(a) of the AEA further states that TSs shall be part of the license. The Commission's regulatory requirements related to the content of TSs are set forth in 10 CFR 50.36, "Technical specifications," which require, in pertinent part, that the TSs include: (1) safety limits, limiting safety system settings, and limiting control settings; (2) LCOs; (3) SRs; (4) design features; and (5) administrative controls.

The regulation at 10 CFR 50.36(c)(3) states that SRs are "requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCOs will be met."

The regulation at 10 CFR 50.36(c)(5) states, in part, that TS requires administrative controls, which "are the provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner."

The regulations at 10 CFR 50.55a(f), "Preservice and inservice testing requirements," meet the IST requirements of the ASME *Boiler and Pressure Vessel Code* (ASME Code) and the ASME OM Code, Section IST, "Rules for Inservice Testing of Light-Water Reactor Power Plants."

The regulations at 10 CFR 50.69, "Risk-informed categorization and treatment of structures, systems and components for nuclear power reactors," provides alternative requirements for some SSCs in lieu of the requirements of 10 CFR 50.55a(f). However, 10 CFR 50.69(b)(2) states that, "[a] licensee voluntarily choosing to implement this section shall submit an application for license amendment under [10 CFR] 50.90..."

The regulation at 10 CFR 50.69(b)(3) states that, "[t]he Commission will approve a licensee's implementation of this section if it determines that the process for categorization of [risk-informed safety classification (RISC)] RISC-1, RISC-2, RISC-3, and RISC-4 SSCs satisfies the

requirements of [10 CFR] 50.69(c) by issuing a license amendment approving the licensee's use of this section." Therefore, a licensee may not implement the requirements of 10 CFR 50.69 without prior NRC approval.

The NRC staff's guidance for the review of TSs is provided in NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR [Light-Water Reactor] Edition" (SRP), Chapter 16, Section 16.0, "Technical Specifications," Revision 3, March 2010 (ML100351425). As described therein, as part of the regulatory standardization effort, the NRC staff has prepared STS for each of the LWR nuclear designs. The STS relevant to the TVA sites are listed below:

- NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Volume 1, "Specifications," and Volume 2, "Bases," Revision 5, September 2021 (ML21259A155 and ML21259A159, respectively).
- NUREG-1433, "Standard Technical Specifications, General Electric BWR/4 Plants," Volume 1, "Specifications," and Volume 2, "Bases," Revision 5, September 2021 (ML21272A357 and ML21272A358, respectively).

3.0 TECHNICAL EVALUATION

The NRC staff evaluated TVA's application to determine whether the proposed changes are consistent with the regulations and guidance discussed in section 2.3 of this SE. Additionally, the NRC staff reviewed the request by comparing TVA's proposal against the changes described in TSTF travelers. In particular, the NRC staff assessed whether or not the variations identified by TVA affect the applicability of NRC-approved TSTF travelers. To make this assessment, the staff compared the plants' designs and existing TSs with the design and TSs presumed in the TSTF travelers and corresponding STS.

3.1 Evaluation of Proposed TS Changes to Adopt TSTF-596

The NRC staff compared TVA's proposed TS changes summarized in section 2.2.1 of this SE against the changes approved in TSTF-596. Consistent with SRP Section 16.0, the NRC staff determined that the STS changes approved in TSTF-596 are applicable to the TSs for BFN, SQN, and WBN, because they are BWR/4, Westinghouse, and Westinghouse designs, respectively. The NRC staff approved the TSTF-596 changes for these designs.

TSTF-596 expanded the SFCP to include other TS periodic tests that had not previously been included in the SFCP. Specifically, the proposed SFCP expansion would include: (1) surveillance testing required by the IST Program; (2) surveillance testing in which an alternative treatment has been applied in accordance with 10 CFR 50.69(b)(1)(v) in lieu of the IST Program requirements; and (3) periodic testing that is required by the TS section 5.5 programs for Primary Coolant Sources Outside Containment, Ventilation Filter Testing, Diesel Fuel Oil Testing, and Control Room Envelope Habitability. To accomplish this, additional methodologies for the control of surveillance frequencies are added to the SFCP to provide the methodologies necessary for controlling the IST Program and 10 CFR 50.69 alternative treatment surveillance frequencies.

By revising the SR frequencies that reference the IST Program to reference the SFCP and revising the SFCP to describe control mechanisms for both IST Program testing and 10 CFR 50.69 alternative treatment, the proposed changes would, if approved, clarify that a licensee

that has an approved license amendment from the NRC to utilize the provisions of 10 CFR 50.69 may control a surveillance frequency of a RISC-3 SSC that had previously been tested in accordance with the IST Program, where the IST Program had previously been defined as the program that fulfilled the requirements of 10 CFR 50.55a(f), without needing an additional license amendment to revise its TSs. The NRC staff's evaluation of these proposed changes is provided in sections 3.2, 3.3, 3.4, and 3.5 of the NRC staff SE for TSTF-596. As noted in the NRC staff's evaluation of the proposed changes, for a licensee for which the staff has not yet approved a license amendment to adopt 10 CFR 50.69, the proposed changes would not remove the need for the licensee to apply for that amendment before using the language that references the 10 CFR 50.69 alternative.

In the SE of TSTF-596, the NRC staff concluded that the proposed changes would ensure that the requirements of 10 CFR 50.55a(f) and 10 CFR 50.69 that are applicable to TS SRs would be met. The NRC staff reviewed the proposed changes to the SRs and concluded that the modified SRs would continue to assure that the necessary quality of systems and components is maintained that facility operation will be within safety limits, that the LCOs will be met, and the changes would continue to provide reasonable assurance of adequate protection of the health and safety of the public. For these same reasons, the NRC staff finds that the corresponding proposed changes to the TSs for BFN, SQN and WBN, as described in section 2.2.1 of this SE, continue to meet the requirements of 10 CFR 50.36(c)(3).

3.1.1 Evaluation of Variations

In addition to the proposed changes that are consistent with TSTF-596, TVA proposed variations referenced in SE section 2.2.1.4 above. For example, in some instances, the TSs proposed to be revised in the LAR had different TS numbers, titles, language, or content as compared to TSTF-596. In addition, there were instances where no plant-specific change was needed because it was already consistent with TSTF-596 or the plant did not have a similar requirement to the STS. In both cases no changes were necessary.

There were also cases where a plant-specific change was applicable but did not exist in the STS on which TSTF-596 was based. For these cases, plant-specific changes were made to remain consistent with TSTF-596 and, if appropriate, the guidance in the applicable STS as updated by NRC-approved travelers. In other instances, all uses of a defined term (e.g., STAGGERED TEST BASIS) were eliminated, therefore, the term was removed from the definition section of TSs.

The NRC staff finds that these types of variations described above are acceptable because the variations are consistent with the intent of TSTF-596 or do not substantively alter the TS requirements. In the following subsections, the NRC staff evaluates the variations regarding the SFCP and the IST Program.

3.1.2 Evaluation of SFCP Variations

As identified in the enclosure to the LAR, section 2.2, TVA proposed variations to the SFCP TS for BFN, SQN, and WBN. In particular, TVA proposed the following:

- SFCP part a:
 - clarifying that the SFCP shall contain a list of Frequencies controlled by the programs "that govern the activities listed in items b.1, b.2, and b.3"

- SFCP part b:
 - clarifying that the SFCP refers to the Inservice Testing Program in item b.1 and removing “in lieu of testing required by 10 CFR 50.55a(f)” in item b.2
 - adding “or as specifically approved by the NRC.” at the end of item b.3
- SFCP part c:
 - clarifying SR 3.0.2 applicability in item c.1

The NRC staff reviewed the SFCP variations provided in the LAR, as modified in the supplemental letter. In the enclosure to the LAR, section 2.3, TVA explained that SFCP ‘part a’ proposed changes that supported plant-specific development of implementing procedures. For part b.1, TVA explained that the change is to clearly state that the requirement applies to the IST Program required by 10 CFR 50.55a(f) versus simply stating testing required by 10 CFR 50.55a(f). For part b.2, TVA explained that removing reference to the IST Program is appropriate based on the proposed change to specify the IST Program in item b.1. Based on the information provided in the LAR, the NRC staff finds that the proposed changes are clarifications that are consistent with the intent of TSTF-596 and do not substantively alter the TS requirements.

In the supplemental letter to the LAR, TVA explained proposed changes to SFCP items b.3 and c.1. The proposed change to item b.3 reflects the changes in TSTF-614-T, Revision 0, “Revise the Surveillance Frequency Control Program to Acknowledge NRC-Approved Frequency Changes,” dated April 13, 2026. This “T-traveler” made a minor change to BWR/4 STS 5.5.16, “Surveillance Frequency Control Program,” and Westinghouse STS 5.5.19, “Surveillance Frequency Control Program,” to add “or as specifically approved by the NRC,” to BWR/4 STS 5.5.16.b.3 and Westinghouse STS 5.5.19.b.3 regarding the reference to NEI 04-10, Revision 1. TVA explained that the proposed change to item b.3 is consistent with prior NRC approvals of amendments to change surveillance frequencies (e.g., ML26023A004 and ML25043A428). Based on the information provided in the supplement, the NRC staff finds that the proposed change to SFCP item b.3 is applicable to TVA because it is generically applicable and consistent with prior NRC approvals.

The proposed change to item c.1 clarified that the regulations take precedence over the TS. In particular, the “c.1” exception ensures regulations take precedence over TS by clarifying that SR 3.0.2 applies to IST surveillances only if allowed by the regulation, which may change. Based on the information provided in the supplement, the NRC staff finds that the proposed changes are clarifications that do not substantively alter TS requirements.

Based on the discussion above, the NRC staff finds the variations to the SFCP TS are acceptable because the SFCP, as revised, will continue to assure operation of the facility in a safe manner in accordance with 10 CFR 50.36(c)(5).

3.1.3 Evaluation of Inservice Testing Program Variations

3.1.3.1 BFN Variations

TVA described in the enclosure to the LAR that TSTF-596 deletes, in its entirety, the NUREG-1433 BWR/4 STS 1.1 Definition for the IST Program. As stated below, the deletion of this definition causes some hardship for TVA; therefore, TVA desires to retain the definition.

In order to avoid the numerous procedure changes with the STS revision and avoid any potential misunderstandings regarding the TS requirements for the IST program, TVA is retaining the TS 1.1 Definition for the BFN Units 1, 2 and 3 IST program and adding a reference to TS 5.5.15 (i.e., the SFCP). In accordance with TSTF-596, BFN Units 1, 2 and 3 TS 5.5.15 [SFCP] is being revised to include the IST Program per 10 CFR 50.55a.

The NRC staff evaluated this variation and noted that the reason TSTF-596 deleted the STS definition for the IST Program is because all uses of this defined term in the STS are eliminated and replaced with the SFCP. The same is true for BFN; that is, with the proposed adoption of TSTF-596, BFN TSs will not use the defined term in the TSs. Therefore, it appears that the defined term has no practical TS application and, as such, would ordinarily be removed from definitions section of the TS. However, retaining the definition benefits TVA by avoiding numerous procedure changes. In the supplement to the LAR, TVA confirmed that no potential inadvertent impacts to safe plant operations might be imposed by retaining the defined term. Although the NRC staff prefers the standard approach developed in TSTF-596, the NRC staff finds that this variation from STS guidance, as modified by TSTF-596, does not substantively alter the TS requirements. Therefore, the NRC staff finds that retaining the TS definition for the IST program and adding a pointer to the SFCP does not impact TVA's ability to continue to assure operation of the facility in a safe manner in accordance with 10 CFR 50.36(c)(5).

3.1.3.2 SQN and WBN Variations

TVA described in the enclosure to the LAR that the removal of the IST Program from SQN and WBN TSs causes some hardship (numerous procedure changes) and TVA justifies retaining some aspects of the current TS Chapter 5 IST Program as follows:

The SQN and WBN Units 1 and 2 TS do not include the changes in TSTF-545 ["TS Inservice Testing Program Removal & Clarify SR Usage Rule Application to Section 5.5 Testing"]. As a result, the SQN and WBN Units 1 and 2 TS include a TS Section 5.5.6 (SQN) and 5.7.2.11 (WBN), "Inservice Testing Program," and do not include a defined term, "INSERVICE TESTING PROGRAM." As discussed in TSTF-596-A, the proposed change removes TS Section 5.5, "Inservice Testing Program," in addition to making the other changes described in the traveler. Furthermore, Section 3.6 of TSTF-596-A states, "Licensees that have not adopted TSTF-545 may adopt the proposed traveler and delete the TS Section 5.5, "Inservice Testing Program" without adding the defined term, "INSERVICE TESTING PROGRAM," that is eliminated by this traveler.

Therefore, the contents of SQN Units 1 and 2 TS 5.5.6 and WBN Units 1 and 2 TS 5.7.2.11, "Inservice Testing Program," are being deleted consistent with the Westinghouse STS and TSTF-545, Revision 3. However, the titles of SQN Units 1 and 2 TS 5.5.6 and WBN Units 1 and 2 TS 5.7.2.11 are being revised to

state, “Inservice Testing Program (see Section 5.5.17),” and “Inservice Testing Program (see Section 5.7.2.23),” respectively, in order to avoid the numerous procedure changes with the STS revision. Additionally, the definition of the IST program is being added to SQN Units 1 and 2 TS 5.5.6 and WBN Units 1 and 2 TS 5.7.2.11 for consistency with BFN TS 1.1. Furthermore, the verbiage that SR 3.0.3 is applicable to inservice testing activities is being retained in SQN Units 1 and 2 TS 5.5.6 and WBN Units 1 and 2 TS 5.7.2.11. This is a non-technical variation to the traveler.

Traveler TSTF-596 discusses plant-specific variations for licensees that have not adopted TSTF-545 because TSTF-596 revised the same SRs that were affected by TSTF-545 to point to the SFCP. The practical result of both TSTF-545 and the TSTF-596 plant-specific variation is the same—to point to 10 CFR 50.55a(f) for the applicable testing requirements. The same is true for SQN and WBN; that is, with the proposed adoption of TSTF-596, the SQN SRs and WBN SRs will no longer refer to the IST program and instead refer to the SFCP. Therefore, since it is evident that there is no SQN or WBN TS SR that will refer to the IST Program, it may be removed from the TSs. However, it appears that retaining some semblance of the IST Program in SQN and WBN Chapter 5 TSs benefits TVA by avoiding numerous procedure changes. In the supplement to the LAR, TVA confirmed that no potential inadvertent impacts to safe plant operations might be imposed by retaining the portions of this administrative control (IST Program). Although the NRC staff prefers the standard approach developed in TSTF-596, the NRC staff finds that this SQN and WBN variation from STS guidance, as modified by TSTF-596, does not substantively alter the TS requirements. Therefore, the NRC staff finds that the SQN and WBN revisions to the Chapter 5 TS for the IST Program, which include adding a pointer to the SFCP and stating that the IST Program is the TVA program that fulfills the requirements of 10 CFR 50.55a(f), does not impact TVA’s ability to continue to assure operation of the facility in a safe manner in accordance with 10 CFR 50.36(c)(5). In addition, retaining the verbiage that SR 3.0.3 is applicable to IST activities is acceptable because it is consistent with the SFCP item c and, as such, does not impact the TVA’s ability to continue to assure operation of the facility in a safe manner in accordance with 10 CFR 50.36(c)(5).

3.1.4 TS Change Consistency

The NRC staff reviewed the proposed TS changes for technical clarity and consistency with the existing guidance for customary terminology and formatting. The NRC staff finds that the proposed changes regarding the adoption of TSTF-596 are consistent with Section 16.0 of the SRP and are, therefore, acceptable.

3.2 Evaluation of Proposed Changes to Adopt TSTF-118, TSTF-374, and TSTF-600 (WBN Only)

TVA stated that it had reviewed TSTF-118, TSTF-374, and TSTF-600 and the associated NRC staff SEs and confirmed that the travelers are applicable to WBN. The NRC staff notes that TSTF-118 does not have an associated SE, but it was approved by the NRC and incorporated into Revision 2 of the Westinghouse STS.

3.2.1 Evaluation of Proposed TS Changes to Adopt TSTF-118

TSTF-118 adds a sentence to TS 5.7.2.16, “Diesel Fuel Oil Testing Program” that states, “The provisions of SR 3.0.2 and 3.0.3 are applicable to the Diesel Fuel Oil Testing Program testing frequencies.” SR 3.0.2 permits the surveillance to be performed within 25 percent of the

surveillance interval. SR 3.0.3 permits a surveillance to be performed within 24 hours if it is discovered that the surveillance was not performed within the required frequency.

The lack of a statement in TS 5.7.2.16 that designates the applicability of these SRs to the diesel fuel oil testing program introduces confusion because the SRs are located in TS 3.8.3, "Diesel Fuel Oil, Lube Oil, and Starting Air," to which SR 3.0.2 and 3.0.3 apply. Additionally, the applicability of SRs 3.0.2 and 3.0.3 to the program surveillance is consistent with the current licensing basis for plants, including WBN. The addition of the statement to the Diesel Generator Lube Oil Program adds consistency and clarifies the TS requirements.

TSTF-118 was incorporated into the STS in Revision 2. This demonstrates NRC staff acceptance of the changes proposed in the traveler.

Variations for TSTF-118

In addition to the changes proposed consistent with the traveler discussed in SE section 2.2.2, TVA proposed the variations below.

Traveler TSTF-118 revises the current Westinghouse STS 5.5.12. The equivalent WBN TS which is proposed for revision is TS 5.7.2.16. The traveler is also applicable to current Westinghouse STS 5.5.8. "Steam Generator Tube Surveillance Program," however that was removed from the STS by TSTF-449. Therefore, only the change to TS 5.7.2.16 is proposed for adoption. These variations from the original traveler are editorial or have been addressed by another traveler and do not affect the requirements of the TS. The NRC staff finds that the proposed variations are, therefore, acceptable.

Conclusions for TSTF-118

The NRC staff finds that TSTF-118 is applicable to WBN and that the proposed changes are consistent with the traveler. The addition of SRs 3.0.2 and 3.0.3 to TS 5.7.2.16 ensures that the SRs in TS 3.8.3 and the DG Fuel Oil Program are implemented consistently. The incorporation of these SRs into TS 5.7.2.16 does not inhibit the TS from assuring safe operation of the facility. Therefore, the regulation in 10 CFR 50.36(c)(5) continues to be met. The staff finds that the proposed change to incorporate the TSTF-118 changes into WBN TS 5.7.2.16 is acceptable.

3.2.2 Evaluation of Proposed TS Changes to Adopt TSTF-374

The proposed changes would revise TS 5.7.2.16, "Diesel Fuel Oil Testing Program." Specifically, TVA proposed addition of an acceptance criterion that may be used in place of the current requirement for appearance of diesel fuel oil. The new criterion allows the fuel to have a water and sediment content within limits. TVA also proposed to remove a reference to an ASTM method for testing from TS 5.7.2.16. The reference has been moved to the Bases for TS 3.8.3. In addition, TVA proposed additions to the Bases for SR 3.8.3.3 to provide explanatory information for the change to TS 5.7.2.16 and document the ASTM standards used by the site for fuel oil testing. The change to the Bases also adds or revises ASTM standards that provide acceptable test methods and acceptance criteria for diesel fuel oil.

Variations for TSTF-374

In addition to the changes proposed consistent with the traveler discussed in SE section 2.2.3, TVA proposed the variations below.

Editorial Variations

TSTF-374 revises the current Westinghouse STS 5.5.12. The equivalent WBN TS which is proposed for revision is TS 5.7.2.16. This variation is a numbering difference between the traveler and the WBN TS. The NRC staff finds that it has no effect on TS requirements and is, therefore, acceptable.

Other Variations

TVA proposed to use different ASTM standards than those specified in the traveler. TSTF-374 specifies that ASTM D2276 should be replaced with ASTM 5452. Instead, TVA proposed to replace ASTM D2276 with ASTM D6217. TVA cited the previous NRC approval of the use of ASTM D6217 during the SQN conversion to STS (ML13330A922). The NRC staff also confirmed, as part of this review, that ASTM D6217 is an acceptable standard to ensure the necessary quality of the fuel oil. Therefore, the NRC staff finds the proposed change acceptable.

In its Bases markup, TVA also made a variation by proposing to replace ASTM D2709 as cited in TSTF-374 with ASTM D1796. The NRC staff confirmed that ASTM D1796 is an acceptable standard to ensure the necessary quality of the fuel oil.

Conclusions for TSTF-374

The NRC staff documented its approval of TSTF-374-A, Revision 0, in a letter to the TSTF dated January 13, 2005 (ML050130309). The NRC also included this change to STS Revision 2 on April 14, 2006. TVA has adopted the traveler as approved except as described by the variations that have been reviewed above.

By this change, TVA is documenting the appropriate ASTM standards for testing the fuel oil in the TS Bases and in site procedures. Future changes to standards different than those cited in these documents require 10 CFR 50.59 review, which will ensure that safety is maintained.

Based on the above, the NRC staff finds that the proposed changes to TS 5.7.2.16 are consistent with the approved TSTF-374. Additionally, the NRC staff finds that the requirements of 10 CFR 50.36(c)(3) and 10 CFR 50.36(c)(5) continue to be met because the revised TSs provide the appropriate administrative controls and Bases information to ensure the necessary quality of the fuel oil is maintained. Therefore, the staff finds that the proposed change to incorporate the TSTF-374 changes into WBN TS 5.7.2.16 is acceptable.

3.2.3 Evaluation of Proposed TS Changes to Adopt TSTF-600

TVA proposed to revise the frequency for TS SR 3.4.14.1, for RCS PIV testing, to be "In accordance with the Surveillance Frequency Control Program." TVA also proposed to remove Notes 2 and 3 from SR 3.4.14.1 per TSTF-600. TVA proposed changes to the Bases for TS 3.4.14, "RCS Pressure Isolation Valve (PIV) Leakage" to reflect the changes made to the SR.

TSTF-600 proposed elimination of the time and event-based frequencies for the RCS PIV SR and replacement of these frequencies with requirements from 10 CFR 50.55a(f), which invokes ASME OM Code requirements and associated IST. The frequency is proposed to be, "In

accordance with the Inservice Test Program.” As discussed in the variation section below, TVA proposed to adopt a frequency of, “In accordance with the Surveillance Frequency Control Program.”

TSTF-600 also proposed to remove two notes, Notes 2 and 3, from SR 3.4.14.1. The notes provide exceptions that allowed the SR not to be performed under conditions specified in the notes. With the change in frequency proposed by the traveler, the notes are no longer required.

TSTF-600 recommended changes to the TS Bases associated with SR 3.4.14.1 to reflect the change to the frequency and to remove references to outdated studies regarding the risk associated with RCS PIVs.

In the SE for TSTF-600 (ML25247A240), the NRC staff found that frequencies based on the ASME Code requirements provide assurance of adequate safety when used in place of the current time and event-based frequencies.

Although a survey of plants found that ISLOCA is not a significant contributor to plant risk even when the event- and time-based frequencies are replaced by ASME Code required testing, the NRC staff SE notes that the model application requires each applicant to confirm that this is true for their specific plant(s) and that this assures that plant-specific risk is acceptable.

The model application also requires plants to confirm that there are no other licensing basis requirements that would require more frequent testing of the PIVs than required by the ASME Code.

TVA confirmed that ISLOCA is not a significant contributor to plant risk and that there are no other licensing basis requirements for the PIVs to be tested more frequently. TVA also confirmed that the PIVs at WBN are included in the IST Program.

The NRC staff SE for TSTF-600 also discussed the potential for plants to adopt TSTF-596, as was done for WBN, so that the frequency is determined per the SFCP. The adoption of the SFCP, per TSTF-596, for SRs required by the ASME Code, refers to the requirements of 10 CFR 50.55a when establishing the TS frequencies. This ensures that the ASME Code requirements are met.

The NRC staff final SE for TSTF-600 concludes that Notes 2 and 3 for SR 3.4.14.1 are no longer needed once the event- and time-based frequencies are deleted. In addition, the Notes provide relief from performing the SR under specified conditions, so their removal is conservative.

Variations for TSTF-600

Editorial Variations

No editorial variations were proposed for TSTF-600.

Other Variations

In addition to the changes proposed consistent with the traveler discussed in section 2.2.4, TVA proposed a single variation. The variation is a result of the LAR implementing changes from both TSTF-600 and TSTF-596. TSTF-600 sets the frequency of SR 3.4.14.1 to be “In

accordance with the Inservice Test Program.” TVA proposed setting the frequency for the SR to be “In accordance with the Surveillance Frequency Control Program.” This variation is acceptable because TSTF-596 incorporates the IST Program frequencies into the SFCP and replaces all frequencies that are in accordance with the IST Program with the SFCP. The acceptability of the incorporation of changes based on TSTF-596 were found acceptable in this SE. In addition, this possibility was discussed in section 2.4 of TSTF-600, and in the NRC staff SE on TSTF-600.

Conclusion for TSTF-600 Adoption

The NRC staff SE for TSTF-600 concluded that the traveler contains sufficient justification for the proposed change. In addition, TSTF-600 provides sufficient detail to ensure that plant-specific requests for changes to the SR Frequency for PIV testing will be appropriately evaluated. The requirements in 10 CFR 50.36(c)(3) for SRs will continue to be met with the changes to the required Frequency and the proposed deletion of the SR Notes. Therefore, the NRC staff finds that changing from the current surveillance frequencies to a surveillance frequency that is in accordance with the IST Program (or SFCP if adopted by TSTF-596) is acceptable.

3.2.4 Technical Conclusions for WBN Adoption of TSTF-118, TSTF-374, and TSTF-600

The regulatory framework that the NRC staff used to determine the acceptability of TVA's proposed changes consisted of the requirements and guidance listed in section 2.3 of this SE. The NRC staff compared TVA's proposed TS changes summarized in sections 2.2.2, 2.2.3, and 2.2.4 of this SE against the changes approved in TSTF-118, 374, and 600. Consistent with SRP Section 16.0, the NRC staff determined that the STS changes approved in the travelers are applicable to WBN TSs because the WBN units are PWR designs and the NRC staff approved TSTF-118, TSTF-374, and TSTF-600 changes for this design.

The changes, based on the TSTF travelers, were reviewed to ensure that the compliance with the specified requirements of 10 CFR 50.36 would be maintained.

For TSTF-118, the Diesel Generator Fuel Oil Program is proposed to be changed. The proposed change affects the frequency of SRs that provide assurance that the fuel oil will support operation of the diesel generators.

For TSTF-374, TVA proposed revising the Diesel Generator Fuel Oil Program by specifying an alternate acceptance criterion for the appearance of the fuel oil. The alternative is to measure the sediment and water in the fuel oil to assure its quality. TVA also proposed alternate ASTM standards for various measures of fuel oil quality. This proposed change affects the acceptance criteria for SRs that provide assurance that the fuel oil will support operation of the diesel generators.

The regulations at 10 CFR 50.36(c)(3) and (c)(5) place requirements on SRs and administrative controls, respectively. The Diesel Generator Fuel Oil Program is an administrative program within TSs. TS administrative programs are provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner. Surveillance requirements are requirements relating to test, calibration, or inspection to ensure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCOs will be met. The proposed changes meet these requirements by providing assurance that SRs will be

performed to maintain necessary quality and will be performed at a frequency that ensures quality. Therefore, the NRC staff concludes that the proposed changes associated with TSTF-118 and TSTF-374 meet the requirements of 10 CFR 50.36(c)(3) for SRs and 10 CFR 50.36(c)(5) for Administrative controls.

For TSTF-600, TVA proposed to change the frequency of the SR for PIV testing. As discussed above, the change in frequency continues to provide assurance that the necessary quality of the PIVs is maintained and the LCO will be met. Therefore, the requirements of 10 CFR 50.36(c)(3) continue to be met.

3.2.5 TS Change Consistency

The NRC staff reviewed the proposed TS changes for technical clarity and consistency with the existing guidance for customary terminology and formatting. The NRC staff finds that the proposed changes are consistent with Section 16.0 of the SRP and are therefore acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Alabama State official and Tennessee State official were notified of the proposed issuance of the amendments on August 26, 2026. The State officials had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

This action relates to authorizations under 10 CFR Part 50 with respect to changes to inspection of surveillance requirements and installation or use of a facility component. The NRC staff has determined that any ground disturbance is limited to previously disturbed areas. Additionally, the NRC staff has determined that the action involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, no significant increase in individual or cumulative public or occupational radiation exposure, and no significant increase in the potential for or consequences from radiological accidents. Finally, the NRC staff has determined that a categorical exclusion applies and that special circumstances under 10 CFR 51.22, "Categorical exclusions," are not present that would preclude reliance on the categorical exclusion. Accordingly, this action meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(d)(1) and (d)(8). Pursuant to 10 CFR 51.22, no environmental impact statement or environmental assessment need be prepared in connection with the action.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: September 25, 2026

SUBJECT: BROWNS FERRY NUCLEAR PLANT, UNITS 1, 2, AND 3; SEQUOYAH NUCLEAR PLANT, UNITS 1 AND 2; WATTS BAR NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENT NOS. 346; 369, AND 329; 380 AND 375; AND 186 AND 91 REGARDING ADOPTION OF TSTF-596, REVISION 2; AND ADOPTION OF TSTF-374-A, REVISION 0, TSTF-118-A, REVISION 0, AND TSTF-600-A, REVISION 2, FOR WATTS BAR ONLY (EPID L-2026-LLA-0034) DATED SEPTEMBER 25, 2026

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