



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

September 28, 2026

Mr. Shawn Hafen
Site Vice President
Northern States Power Company
- Minnesota
Monticello Nuclear Generating Plant
2807 West County Road 75
Monticello, MN 55362

SUBJECT: MONTICELLO NUCLEAR GENERATING PLANT, UNIT 1 - ISSUANCE OF
AMENDMENT NO. 218 RE: ADOPTION OF TSTF-585, "REVISE LCO 3.0.3 TO
REQUIRE MANAGING RISK" AND TSTF-597, "ELIMINATE LCO 3.0.3 MODE 2
REQUIREMENTS" (EPID L-2026-LLA-0096)

Dear Mr. Hafen:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment No. 218 to Renewed Facility Operating License No. DPR-22, for the Monticello Nuclear Generating Plant, Unit 1. The amendment consists of changes to the Technical Specifications in response to your application dated May 13, 2026, as supplemented by letter dated June 22, 2026.

The amendment adopts Technical Specification Task Force (TSTF) Traveler TSTF-585, "Revise LCO [Limiting Condition for Operation] 3.0.3 to Require Managing Risk," and TSTF-597, "Eliminate LCO 3.0.3 Mode 2 Requirements."

A copy of the safety evaluation is also enclosed. A Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Robert Kuntz, Senior Project Manager
Operating Reactor Licensing Branch 3
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Docket No. 50-263

Enclosure:

1. Amendment No. 218 to DPR-22
2. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY - MINNESOTA

DOCKET NO. 50-263

MONTICELLO NUCLEAR GENERATING PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 218
Renewed License No. DPR-22

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company – Minnesota dated May 13, 2026, as supplemented by letter dated June 22, 2036 complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission’s rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission’s regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission’s regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-22 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 218, are hereby incorporated in the license. NSPM shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 12 months of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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BERRIOS

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BERRIOS
Date: 2026.09.28
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Ilka Berrios, Chief
Operating Reactor Licensing Branch 3
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and Technical
Specifications

Date of Issuance: September 28, 2026

ATTACHMENT TO LICENSE AMENDMENT NO. 218
MONTICELLO NUCLEAR GENERATING PLANT, UNIT 1
AMENDMENT TO RENEWED FACILITY OPERATING LICENSE
DOCKET NO. 50-263

Renewed Facility Operating License No. DPR-22

Replace the following page of the Renewed Facility Operating License No. DPR-22 with the attached revised page. The revised page is identified by amendment number and contains marginal lines indicating the area of change.

INSERT
Page 3

REMOVE
Page 3

Technical Specifications

Replace the following pages of Appendix A, Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

INSERT
3.0-1
3.0-2
3.0-3
3.0-4
3.0-5

3.6.4.3-2
3.7.4-2
3.8.4-2
3.8.7-2

REMOVE
3.0-1
3.0-2
3.0-3
3.0-4
3.0-5
3.0-6
3.6.4.3-2
3.7.4-2
3.8.4-2
3.8.7-2

2. Pursuant to the Act and 10 CFR Part 70, NSPM to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operations, as described in the Final Safety Analysis Report, as supplemented and amended, and the licensee's filings dated August 16, 1974 (those portions dealing with handling of reactor fuel);
 3. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 4. Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 5. Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to possess, but not separate, such byproduct and special nuclear material as may be produced by operation of the facility.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission, now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
1. Maximum Power Level

NSPM is authorized to operate the facility at steady state reactor core power levels not in excess of 2004 megawatts (thermal).
 2. Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 218, are hereby incorporated in the license. NSPM shall operate the facility in accordance with the Technical Specifications.
 3. Physical Protection

NSPM shall implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search

3.0 LIMITING CONDITION FOR OPERATION (LCO) Applicability

LCO 3.0.1	LCOs shall be met during the MODES or other specified conditions in the Applicability, except as provided in LCO 3.0.2, LCO 3.0.7, LCO 3.0.8, and LCO 3.0.9.
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LCO 3.0.2	Upon discovery of a failure to meet an LCO, the Required Actions of the associated Conditions shall be met, except as provided in LCO 3.0.5 and LCO 3.0.6.
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If the LCO is met or is no longer applicable prior to expiration of the specified Completion Time(s), completion of the Required Action(s) is not required, unless otherwise stated.

LCO 3.0.3	When an LCO is not met and the associated ACTIONS are not met, an associated ACTION is not provided, or if directed by the associated ACTIONS, the unit shall be placed in a MODE or other specified condition in which the LCO is not applicable. Within 6 hours perform a risk assessment addressing inoperable systems and components and implement appropriate risk management actions. If the risk assessment determines that continuing operation is acceptable, the appropriate risk management actions are implemented, and entry into LCO 3.0.3 was unplanned, then action shall be initiated within 24 hours to place the unit, as applicable, in:
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- a. MODE 3 within 36 hours, and
- b. MODE 4 within 60 hours.

If the risk assessment determines that continuing operation is not acceptable, the risk assessment was not completed, the appropriate risk management actions were not implemented, or entry into LCO 3.0.3 was planned, then action shall be initiated to place the unit, as applicable, in:

- a. MODE 3 within 18 hours, and
- b. MODE 4 within 42 hours.

All times are determined from entry into LCO 3.0.3.

Exceptions to this Specification are stated in the individual Specifications.

If corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, then completion of the actions required by LCO 3.0.3 is not required.

LCO 3.0.3 is only applicable in MODES 1, 2, and 3.

LCO Applicability

LCO 3.0.4	When an LCO is not met, entry into a MODE or other specified condition in the Applicability shall only be made: a. When the associated ACTIONS to be entered permit continued operation in the MODE or other specified condition in the Applicability for an unlimited period of time; b. After performance of a risk assessment addressing inoperable systems and components, consideration of the results, determination of the acceptability of entering the MODE or other specified condition in the Applicability, and establishment of risk management actions, if appropriate (exceptions to this Specification are stated in the individual Specifications); or c. When an allowance is stated in the individual value, parameter, or other Specification. This Specification shall not prevent changes in MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.
LCO 3.0.5	Equipment removed from service or declared inoperable to comply with ACTIONS may be returned to service under administrative control solely to perform testing required to demonstrate its OPERABILITY or the OPERABILITY of other equipment. This is an exception to LCO 3.0.2 for the system returned to service under administrative control to perform the testing required to demonstrate OPERABILITY.
LCO 3.0.6	When a supported system LCO is not met solely due to a support system LCO not being met, the Conditions and Required Actions associated with this supported system are not required to be entered. Only the support system LCO ACTIONS are required to be entered. This is an exception to LCO 3.0.2 for the supported system. In this event, an evaluation shall be performed in accordance with Specification 5.5.10, "Safety Function Determination Program (SFDP)." If a loss of safety function is determined to exist by this program, the appropriate Conditions and Required Actions of the LCO in which the loss of safety function exists are required to be entered. When a support system's Required Action directs a supported system to be declared inoperable or directs entry into Conditions and Required Actions for a supported system, the applicable Conditions and Required Actions shall be entered in accordance with LCO 3.0.2.

LCO Applicability

LCO 3.0.7	Special Operations LCOs in Section 3.10 allow specified Technical Specifications (TS) requirements to be changed to permit performance of special tests and operations. Unless otherwise specified, all other TS requirements remain unchanged. Compliance with Special Operations LCOs is optional. When a Special Operations LCO is desired to be met but is not met, the ACTIONS of the Special Operations LCO shall be met. When a Special Operations LCO is not desired to be met, entry into a MODE or other specified condition in the Applicability shall only be made in accordance with the other applicable Specifications.
LCO 3.0.8	When one or more required snubbers are unable to perform their associated support function(s), any affected supported LCO(s) are not required to be declared not met solely for this reason if risk is assessed and managed, and: a. The snubbers not able to perform their associated support function(s) are associated with only one subsystem of a multiple subsystem supported system or are associated with a single subsystem supported system and are able to perform their associated support function within 72 hours; or b. The snubbers not able to perform their associated support function(s) are associated with more than one subsystem of a multiple subsystem supported system and are able to perform their associated support function within 12 hours. At the end of the specified period the required snubbers must be able to perform their associated support function(s), or the affected supported system LCO(s) shall be declared not met.
LCO 3.0.9	When one or more required barriers are unable to perform their related support function(s), any supported system LCO(s) are not required to be declared not met solely for this reason for up to 30 days provided that at least one division or subsystem of the supported system is OPERABLE and supported by barriers capable of providing their related support function(s), and risk is assessed and managed. This specification may be concurrently applied to more than one division or subsystem of a multiple division or subsystem supported system provided at least one division or subsystem of the supported system is OPERABLE and the barriers supporting each of these divisions or subsystems provide their related support function(s) for different categories of initiating events. For the purposes of this specification, the High Pressure Coolant Injection (HPCI) system, the Reactor Core Isolation Cooling (RCIC) system, and the Automatic Depressurization System (ADS) are considered independent subsystems of a single system.

LCO Applicability

LCO 3.0.9 (continued)

If the required OPERABLE division or subsystem becomes inoperable while this specification is in use, it must be restored to OPERABLE status within 24 hours or the provisions of this specification cannot be applied to the divisions or subsystems supported by the barriers that cannot perform their related support function(s).

At the end of the specified period, the required barriers must be able to perform their related support function(s) or the supported system LCO(s) shall be declared not met.

3.0 SURVEILLANCE REQUIREMENT (SR) Applicability

SR 3.0.1	SRs shall be met during the MODES or other specified conditions in the Applicability for individual LCOs, unless otherwise stated in the SR. Failure to meet a Surveillance, whether such failure is experienced during the performance of the Surveillance or between performances of the Surveillance, shall be failure to meet the LCO. Failure to perform a Surveillance within the specified Frequency shall be failure to meet the LCO except as provided in SR 3.0.3. Surveillances do not have to be performed on inoperable equipment or variables outside specified limits.
SR 3.0.2	The specified Frequency for each SR is met if the Surveillance is performed within 1.25 times the interval specified in the Frequency, as measured from the previous performance or as measured from the time a specified condition of the Frequency is met. For Frequencies specified as "once," the above interval extension does not apply. If a Completion Time requires periodic performance on a "once per . . ." basis, the above Frequency extension applies to each performance after the initial performance. Exceptions to this Specification are stated in the individual Specifications.
SR 3.0.3	If it is discovered that a Surveillance was not performed within its specified Frequency, then compliance with the requirement to declare the LCO not met may be delayed, from the time of discovery, up to 24 hours or up to the limit of the specified Frequency, whichever is greater. This delay period is permitted to allow performance of the Surveillance. The delay period is only applicable when there is a reasonable expectation the surveillance will be met when performed. A risk evaluation shall be performed for any Surveillance delayed greater than 24 hours and the risk impact shall be managed. If the Surveillance is not performed within the delay period, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered. When the Surveillance is performed within the delay period and the Surveillance is not met, the LCO must immediately be declared not met, and the applicable Condition(s) must be entered.
SR 3.0.4	Entry into a MODE or other specified condition in the Applicability of an LCO shall only be made when the LCO's Surveillances have been met within their specified Frequency, except as provided by SR 3.0.3. When an LCO is not met due to Surveillances not having been met, entry into a MODE or other specified condition in the Applicability shall only be made in accordance with LCO 3.0.4.

SR Applicability

SR 3.0.4 (continued)

This provision shall not prevent entry into MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Two SGT subsystems inoperable in MODE 1, 2, or 3.	D.1 Be in MODE 3. <u>AND</u>	12 hours
	D.2 Be in MODE 4.	36 hours
E. Two SGT subsystems inoperable during movement of recently irradiated fuel assemblies in the secondary containment.	E.1 -----NOTE----- LCO 3.0.3 is not applicable. ----- Suspend movement of recently irradiated fuel assemblies in secondary containment.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.6.4.3.1	Operate each SGT subsystem for ≥ 15 continuous minutes.	In accordance with the Surveillance Frequency Control Program
SR 3.6.4.3.2	Perform required SGT filter testing in accordance with the Ventilation Filter Testing Program (VFTP).	In accordance with the VFTP
SR 3.6.4.3.3	Verify each SGT subsystem actuates on an actual or simulated initiation signal.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time of Condition A or B not met in MODE 1, 2, or 3.	C.1 Be in MODE 3.	12 hours
	<u>AND</u> C.2 Be in MODE 4.	36 hours
D. Required Action and associated Completion Time of Condition A not met during movement of recently irradiated fuel assemblies in the secondary containment.	-----NOTE----- LCO 3.0.3 is not applicable. ----- D.1 Place OPERABLE CREF subsystem in pressurization mode.	Immediately
	<u>OR</u> D.2 Suspend movement of recently irradiated fuel assemblies in the secondary containment.	Immediately
E. Two CREF subsystems inoperable in MODE 1, 2, or 3 for reasons other than Condition B.	E.1 Be in MODE 3.	12 hours
	<u>AND</u> E.2 Be in MODE 4.	36 hours

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time of Condition A or B not met.	C.1 Be in MODE 3.	12 hours
	<u>AND</u>	
	C.2 Be in MODE 4.	36 hours
D. Two Division 1 or Division 2 DC electrical power subsystems inoperable.	D.1 Be in MODE 3.	12 hours
	<u>AND</u>	
	D.2 Be in MODE 4.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.8.4.1	Verify battery terminal voltage is greater than or equal to the minimum established float voltage.	In accordance with the Surveillance Frequency Control Program
SR 3.8.4.2	Verify each required battery charger supplies the following: • ≥ 150 amps for 250 VDC Div 1 • ≥ 110 amps for 250 VDC Div 2 • ≥ 75 amps for 125 VDC subsystems, at greater than or equal to the minimum established float voltage for ≥ 4 hours.	In accordance with the Surveillance Frequency Control Program

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time of Condition A or B not met.	C.1 Be in MODE 3.	12 hours
	<u>AND</u>	
	C.2 Be in MODE 4.	36 hours
D. Two or more electrical power distribution subsystems inoperable that result in a loss of function.	D.1 Be in MODE 3.	12 hours
	<u>AND</u>	
	D.2 Be in MODE 4.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.8.7.1 Verify correct breaker alignments and voltage to required AC and DC electrical power distribution subsystems.	In accordance with the Surveillance Frequency Control Program



UNITED STATES
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WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 218 TO RENEWED

FACILITY OPERATING LICENSE NO. DPR-22

NORTHERN STATES POWER COMPANY - MINNESOTA

MONTICELLO NUCLEAR GENERATING PLANT, UNIT 1

DOCKET NO. 50-263

1.0 INTRODUCTION

Northern States Power Company – Minnesota (the licensee) requested changes to the technical specifications (TSs) for Monticello Nuclear Generating Plant (Monticello), Unit 1 by license amendment request (LAR, application) dated May 13, 2026 (Agencywide Documents Access and Management System Accession No. ML26133A276), as supplemented by letter dated June 22, 2026 (ML26174A091). In its application, the licensee requested that the U.S. Nuclear Regulatory Commission (NRC, the Commission) process the proposed amendment under the Consolidated Line Item Improvement Process (CLIIP). The proposed changes would revise the TS based on Technical Specifications Task Force (TSTF) Traveler TSTF-585, Revision 5, “Revise LCO [Limiting Condition for Operation] 3.0.3 to Require Managing Risk,” (TSTF-585, ML25260A487) and the associated NRC staff safety evaluation (SE) of Traveler TSTF-585 (ML26005A205).

The proposed changes would revise the shutdown requirements in LCO 3.0.3 by requiring the assessment and management of plant risk whenever LCO 3.0.3 is entered. The licensee proposed that the risk assessment and implementation of any appropriate risk management actions must be completed within six hours. The proposed change also revises or adds TS Required Actions to require a plant shutdown instead of entry into LCO 3.0.3.

The LAR also requested the Monticello TSs be modified to adopt TSTF-597, Revision 0, “Eliminate LCO 3.0.3 Mode 2 Requirement” (TSTF-597, ML24075A080), correction letter (ML24260A221), and the associated NRC staff final SE of Traveler TSTF-597 (ML24358A228).

The proposed change to adopt TSTF-597 would revise the shutdown requirements in TS LCO 3.0.3 by removing the requirement to place the unit in MODE 2 prior to entering MODE 3.

The current TS LCO 3.0.3 requires action to be initiated within 1-hour to place the unit, as applicable, in Mode 2 within 7 hours, Mode 3 within 13 hours, and Mode 4 within 37 hours when:

- 1) an LCO is not met and the associated actions are not met,
- 2) an associated action is not provided, or
- 3) if directed to enter LCO 3.0.3 by the associated actions.

LCO 3.0.3 is applicable in Modes 1, 2, and 3.

The supplement dated June 22, 2026, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the *Federal Register* on June 9, 2026 (91 FR 34842). There has been no public comment on such finding.

1.1 Proposed TS Changes to Adopt TSTF-597

In accordance with the NRC staff approved TSTF-597, the licensee proposed changes that would revise LCO 3.0.3. Specifically, the licensee proposed to delete the requirement to be in Mode 2 within 7 hours and renumber the subsequent requirements.

1.2 Proposed TS Changes to Adopt TSTF-585

In accordance with the NRC staff-approved TSTF-585, the licensee proposed changes that restructure LCO 3.0.3. The 1-hour shutdown preparation time is proposed to be replaced with a requirement to assess the risk and implement appropriate risk management actions (RMAs) within 6 hours. In addition, if risk is assessed and managed, and is subsequently determined to be acceptable, operation may continue for a total of 24 hours after unplanned entry into LCO 3.0.3 (i.e., an additional 18 hours beyond the initial 6 hours for the risk assessment and implementation of RMAs). If the plant is still in LCO 3.0.3 at the end of the 24-hour period, the initiation of a shutdown is required. In addition, a shutdown is required to be initiated at the end of the 6-hour period "[i]f the risk assessment determines that continuing operation is not acceptable, the risk assessment was not completed, the appropriate risk management actions were not implemented, or entry into LCO 3.0.3 was planned."

The licensee also proposes to change the following sentence in LCO 3.0.3:

Where corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, completion of the actions required by LCO 3.0.3 is not required.

to read:

If corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, then completion of the actions required by LCO 3.0.3 is not required.

Each of the specified Mode change completion times (CTs) in the current LCO 3.0.3 (a, b, and c) are adjusted in the proposed LCO 3.0.3 to reflect the replacement of the existing 1-hour shutdown preparation time with the proposed increased time for initiating a shutdown of either 6 or 24 hours, as appropriate. The change in CTs does not change the amount of time allowed for the process of shutting down the plant. It reflects a change when the shutdown is required to begin based on the time of entry into LCO 3.0.3.

Table 1 shows current the LCO 3.0.3 compared to the proposed changes to LCO 3.0.3. Text deletions are shown in strikeout font, and additions are shown in bold italicized font.

Table 1
Comparison of Existing LCO 3.0.3 to the Proposed LCO 3.0.3

Current LCO 3.0.3	Proposed LCO 3.0.3
When an LCO is not met and the associated ACTIONS are not met, an associated ACTION is not provided, or if directed by the associated ACTIONS, the unit shall be placed in a MODE or other specified condition in which the LCO is not applicable. Action shall be initiated within 1 hour to place the unit, as applicable, in: a. MODE 2 within 7 hours; b. MODE 3 within 13 hours; and c. MODE 4 within 37 hours. Exceptions to this Specification are stated in the individual Specifications. Where corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, completion of the actions required by LCO 3.0.3 is not required. ...	When an LCO is not met and the associated ACTIONS are not met, an associated ACTION is not provided, or if directed by the associated ACTIONS, the unit shall be placed in a MODE or other specified condition in which the LCO is not applicable. <i>Within 6 hours perform a risk assessment addressing inoperable systems and components and implement appropriate risk management actions. If the risk assessment determines that continuing operation is acceptable, the appropriate risk management actions are implemented, and entry into LCO 3.0.3 was unplanned, then action</i> Action shall be initiated within <i>24 hours</i> 1 hour to place the unit, as applicable, in: a. MODE 2 within 7 hours; ab. MODE 3 within 36 13 hours; and bc. MODE 4 within 60 37 hours. <i>If the risk assessment determines that continuing operation is not acceptable, the risk assessment was not completed, the appropriate risk management actions were not implemented, or entry into LCO 3.0.3 was planned, then action shall be initiated to place the unit, as applicable, in:</i> <i>a. MODE 3 within 18 hours, and</i> <i>b. MODE 4 within 42 hours.</i> <i>All times are determined from entry into LCO 3.0.3.</i> Exceptions to this Specification are stated in the individual Specifications. If Where <i>Where</i> corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, <i>then</i> completion of the actions required by LCO 3.0.3 is not required. ...

Implicit entries into LCO 3.0.3 can occur when an LCO is not met, and there is no associated Action provided for that condition. There are also several places in the TS where entries into LCO 3.0.3 are explicitly required by the TS Actions. In some cases, the application of the revised LCO 3.0.3 would not be appropriate. Accordingly, the licensee in its LAR, proposes the following changes to replace these entries into LCO 3.0.3 with explicit requirements to immediately shutdown:

- TS 3.8.4, “DC [direct current] Sources – Operating,” would be modified to require shutdown when two Division 1 or Division 2 DC electrical power subsystems are inoperable.
- TS 3.8.7, “Distribution Systems – Operating,” would be modified to require a shutdown when two or more electrical power distribution subsystems are inoperable that result in a loss of function.

1.2.1 Additional Proposed TS Changes that were not in TSTF-585

In addition to the changes proposed consistent with the traveler discussed in section 1.2, the licensee proposed the variations below.

1.2.1.1 *Editorial Variations*

The licensee noted that Monticello TSs have different numbering and nomenclature than standard technical specifications (STSs).

Additionally, the licensee proposed to correct the “LCO Applicability” headers for TS subsection 3.0. Currently, there is a double-space between the words at the top of Monticello TS pages 3.0-1 through 3.0-3.

1.2.1.2 *Other Variations*

In its supplement dated June 22, 2026, the licensee proposed changes to address plant-specific variations by replacing explicit entries into LCO 3.0.3 with requirements to shut down the plant for the following:

- TS 3.6.4.3, “Standby Gas Treatment (SGT) System,” would be modified to require shutdown when two SGT subsystems are inoperable in Modes 1, 2, or 3.
- TS 3.7.4, “Control Room Emergency Filtration (CREF) System,” would be modified to require shutdown when two CREF subsystems are inoperable in Modes 1, 2, or 3 for reasons other than an inoperable control room envelope boundary.

2.0 REGULATORY EVALUATION

2.1 Applicable Regulatory Requirements and Guidance

The regulation under Title 10 of the *Code of Federal Regulations* (10 CFR) 50.36(b) requires that:

Each license authorizing operation of a ... utilization facility ... will include technical specifications. The technical specifications will be derived from the analyses and evaluation included in the safety analysis report, and amendments thereto, submitted pursuant to [10 CFR] 50.34 ["Contents of applications; technical information"]. The Commission may include such additional technical specifications as the Commission finds appropriate.

The categories of items required to be in the TS are listed in 10 CFR 50.36(c). The regulation at 10 CFR 50.36(c)(2) requires that TS include LCOs. Per 10 CFR 50.36(c)(2)(i), LCOs "are the lowest functional capability or performance levels of equipment required for safe operation of the facility." The regulation also requires that when an LCO of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the TS until the condition can be met.

LCO 3.0.3 implements 10 CFR 50.36(c)(2)(i) in that it requires a plant to shut down when an LCO is not met and: 1) there are no remedial actions provided in the TS, or 2) the remedial actions provided in the TS are not completed in the time allowed by the TS.

Paragraph (a)(4) of 10 CFR 50.65, "Requirements for monitoring the effectiveness of maintenance at nuclear power plants," (the Maintenance Rule) requires, in part, that:

[b]efore performing maintenance activities (including but not limited to surveillance, post-maintenance testing, and corrective and preventive maintenance), the licensee shall assess and manage the increase in risk that may result from the proposed maintenance activities. The scope of the assessment may be limited to structures, systems, and components that a risk-informed evaluation process has shown to be significant to public health and safety.

NUMARC 93-01, "Industry Guideline for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," (ML18120A069) Section 11.0, "Assessment of Risk Resulting from Maintenance Activities," provides a methodology that is an acceptable approach to manage and assess risk associated with maintenance activities. This methodology has been endorsed by the NRC in Regulatory Guide (RG) 1.160, Revision 4, "Monitoring the Effectiveness of Maintenance at Nuclear Power Plants" (ML18220B281). TSTF-585 revised STS LCO 3.0.3 to require the assessment and management of risk using this NRC-endorsed methodology.

The NRC staff's guidance for the review of TS is in NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR [Light-Water Reactor] Edition" (SRP), Chapter 16, Section 16.0, "Technical Specifications," March 2010 (ML100351425). The NRC staff's review includes consideration of whether the proposed changes are consistent with the NUREG-1433, "Standard Technical Specifications, General Electric Plants, BWR/4," Volume 1, "Specifications," and Volume 2, "Bases," Revision 5, September 2021 (ML21272A357 and ML21272A358, respectively).

3.0 TECHNICAL EVALUATION

3.1 Proposed TS Changes to Adopt TSTF-585

The requirements and guidance the NRC staff used to determine the acceptability of the licensee's proposed changes are listed in section 2.1 of this SE. The NRC staff compared the

licensee's proposed TS changes, summarized in section 1.2 of this SE, against the changes approved in TSTF-585. In accordance with SRP Chapter 16.0, the NRC staff determined that the STS changes approved in TSTF-585 are applicable to Monticello TSs because Monticello is a General Electric boiling water reactor type 3 (BWR/3) design and the NRC staff approved the TSTF-585 changes for that plant design. The NRC staff confirmed that there are no plant-specific design features that would affect the applicability of the proposed TSTF-585 changes to Monticello.

TSTF-585 proposed a restructured STS LCO 3.0.3 by replacing the 1-hour shutdown preparation time with a requirement to assess the risk and implement appropriate RMAs within 6 hours. Additionally, TSTF-585 revised or added some TS Required Actions to require a plant shutdown instead of entry into STS LCO 3.0.3, based on whether the TS directs entry into STS LCO 3.0.3 or could result from an implicit entry into STS LCO 3.0.3. These changes were made in cases where immediate shutdown was more appropriate than the delayed shutdown allowed by the revised STS LCO 3.0.3. The NRC staff finds that the licensee's proposed changes to Monticello TSs in section 1.2 of this SE are consistent with those found acceptable in TSTF-585.

The licensee also confirmed that they will use NUMARC 93-01, "Industry Guideline for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," Section 11.0, "Assessment of Risk Resulting from Maintenance Activities," as their methodology to assess and manage risk when entering LCO 3.0.3. This methodology was endorsed by the NRC in RG 1.160, Revision 4, "Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," and was found acceptable by the staff in the SE of TSTF-585 for assessing and managing risk when entering LCO 3.0.3.

In the NRC SE of TSTF-585, the NRC staff concluded that the proposed changes were acceptable because the proposed STS LCO 3.0.3 provides appropriate remedial actions in lieu of requiring an immediate shutdown. The proposed STS LCO 3.0.3 ensures that appropriate remedial actions are provided for the requested CT, of up to 24 hours, and the appropriate time for remedial actions are provided based on the risk assessment result. If the required systems are not restored to operable status within the specified CT, a shutdown is required. Therefore, the NRC staff found that the proposed changes to STS LCO 3.0.3 were acceptable because they continued to meet the requirements of 10 CFR 50.36(c)(2)(i) as discussed in section 3.0 of the NRC staff's SE of TSTF-585.

For these same reasons, the NRC staff concludes that the corresponding proposed changes to the Monticello TSs in section 1.2 of this SE continue to meet the requirements of 10 CFR 50.36(c)(2)(i).

3.2 Proposed TS Changes to Adopt TSTF-597

The regulatory framework the NRC staff used to determine the acceptability of the licensee's proposed changes consisted of the requirements and guidance listed in section 2.1 of this SE. The NRC staff compared the licensee's proposed TS changes summarized in section 1.1 of this SE against the changes approved in TSTF-597. In accordance with SRP Chapter 16.0, the NRC staff determined that the STS changes approved in TSTF-597 are applicable to Monticello TSs because Monticello is a BWR/3 and the NRC staff approved the TSTF-597 changes for BWR designs. The NRC staff confirmed that there are no plant-specific design features that would affect the applicability of the proposed TSTF-597 changes to Monticello.

TSTF-597 revised STS LCO 3.0.3 to remove the Mode 2 requirement. The NRC staff finds that the licensee's proposed changes to Monticello TSs summarized in section 1.1 of this SE are consistent with those found acceptable in TSTF-597.

In the NRC SE of TSTF-597, the NRC staff evaluated the BWR modes of operation, typical startup and shutdown operating procedures, and any potential impacts to safety that the removal of the Mode 2 requirement in STS LCO 3.0.3 would have. Additionally, the NRC staff evaluated other LCOs to verify the proposed change would not cause conflicts, including LCOs only applicable in Mode 1. As a result, the NRC staff concluded that the proposed changes to STS LCO 3.0.3 were acceptable because the requirement to enter Mode 2 in STS LCO 3.0.3 does not align with typical BWR shutdown procedures, and places unnecessary time pressure on licensees without a commensurate gain in safety. Therefore, the NRC staff found that the proposed changes to STS LCO 3.0.3 were acceptable because they would continue to meet 10 CFR 50.36(c)(2)(i) since the STS LCO 3.0.3 requirements would align with all other LCOs on the times to reach Mode 3 and Mode 4 and in not requiring entry into Mode 2 while shutting down. For these same reasons, the NRC staff concludes that the corresponding proposed changes to the Monticello TSs summarized in section 1.1 of this SE are acceptable and continue to meet the requirements of 10 CFR 50.36(c)(2)(i).

3.2 Additional Proposed Changes

In addition to the changes proposed consistent with TSTF-597 and TSTF-585 discussed in sections 1.1 and 1.2 of this SE, respectively, the licensee proposed the variations below.

3.2.1 Editorial Variations

The LAR noted that the Monticello TSs have different numbering and nomenclature than STS. The NRC staff finds that the different TS numbering and nomenclature changes proposed in the LAR are acceptable because they do not substantively alter TS requirements. Further, the proposed format changes for the TS 3.0 headers would correct the space between "LCO" and "Applicability" to a single space. The NRC staff finds this variation acceptable because it is editorial and non-technical in nature.

3.2.2 Other Variations

Section 1.2.1.2 of this SE shows the technical changes proposed in the LAR that were not included in TSTF-585. The NRC staff reviewed each of the proposed variations and determined that each would add an immediate shutdown requirement in lieu of entry into LCO 3.0.3. These proposed variations are more conservative than entry into the proposed LCO 3.0.3 and are essentially equivalent to their current requirements (without the 1-hour to prepare for shutdown). Therefore, they are acceptable.

3.3 TS Change Consistency

The NRC staff reviewed the proposed TS changes for technical clarity and consistency with the existing requirements for customary terminology and formatting. The NRC staff finds that the proposed changes are consistent with chapter 16.0 of the SRP and are therefore acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations on, the Minnesota State official was notified of the proposed issuance of the amendment on date August 5, 2026. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

These amendments relate to authorizations under, or changes to requirements in 10 CFR Part 50 with respect to installation or use of a facility component. The NRC staff has determined that any ground disturbance is limited to previously disturbed areas. Additionally, the NRC staff has determined that the action involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, no significant increase in individual or cumulative public or occupational radiation exposure, and no significant increase in the potential for or consequences from radiological accidents. Finally, the NRC staff has determined that a categorical exclusion applies and that special circumstances under 10 CFR 51.22, "Categorical exclusions," are not present that would preclude reliance on the categorical exclusion. Accordingly, this action meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(d)(8). Pursuant to 10 CFR 51.22, no environmental impact statement or environmental assessment need be prepared in connection with the action.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date of Issuance: September 28, 2026

SUBJECT: MONTICELLO NUCLEAR GENERATING PLANT, UNIT 1 - ISSUANCE OF AMENDMENT NO. 218 RE: ADOPTION OF TSTF-585, "REVISE LCO 3.0.3 TO REQUIRE MANAGING RISK" AND TSTF-597, "ELIMINATE LCO 3.0.3 MODE 2 REQUIREMENTS" (EPID L-2026-LLA-0096) DATED SEPTEMBER 28, 2026

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