



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**
WASHINGTON, D.C. 20555-0001

September 28, 2026

Mr. Werner Paulhardt
Site Vice President
Northern States Power Company - Minnesota
Prairie Island Nuclear Generating Plant
1717 Wakonade Drive East
Welch, MN 55089

**SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 -
ISSUANCE OF AMENDMENT NOS. 252 AND 240 RE: ADOPTION OF
TSTF-585 "REVISE LCO 3.0.3 TO REQUIRE MANAGING RISK"
(EPID L-2026-LLA-0128)**

Dear Mr. Paulhardt:

The U.S. Nuclear Regulatory Commission (NRC, the Commission) has issued the enclosed Amendment No. 252 to Renewed Facility Operating License No. DPR-42 and Amendment No. 240 to Renewed Facility Operating License No. DPR-60 for the Prairie Island Nuclear Generating Plant, Units 1 and 2, respectively. The amendments consist of changes to the technical specifications (TSs) in response to your application dated June 23, 2026.

The amendments adopt TS Task Force (TSTF) Traveler 585 "Revise LCO [Limiting Condition for Operation] 3.0.3 to Require Managing Risk."

A copy of the related safety evaluation is also enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Robert Kuntz, Senior Project Manager
Operating Reactor Licensing Branch 3
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Docket Nos. 50-282 and 50-306

Enclosures:

1. Amendment No. 252 to DPR-42
2. Amendment No. 240 to DPR-60
3. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-282

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 252
Renewed License No. DPR-42

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (NSPM, the licensee), dated June 23, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-42 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 252, are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 12 months.

FOR THE NUCLEAR REGULATORY COMMISSION

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Date: 2026.09.28
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Ilka Berrios, Chief
Operating Reactor Licensing Branch 3
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and Technical
Specifications

Date of Issuance: September 28, 2026



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

NORTHERN STATES POWER COMPANY

DOCKET NO. 50-306

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 240
Renewed License No. DPR-60

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Northern States Power Company (NSPM, the licensee), dated June 23, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-60 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 240, are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 12 months.

FOR THE NUCLEAR REGULATORY COMMISSION

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Date: 2026.09.28
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Ilka Berrios, Chief
Operating Reactor Licensing Branch 3
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and Technical
Specifications

Date of Issuance: September 28, 2026

ATTACHMENT TO LICENSE AMENDMENT NOS. 252 AND 240

RENEWED FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

DOCKET NOS. 50-282 AND 50-306

Replace the following pages of the Renewed Facility Operating License Nos. DPR-42 and DPR-60 with the attached revised pages. The changed areas are identified by a marginal line.

Renewed Facility Operating License No. DPR-42

REMOVE

Page 3

INSERT

Page 3

Renewed Facility Operating License No. DPR-60

REMOVE

Page 3

INSERT

Page 3

Technical Specifications

Replace the following pages of the Appendix A Technical Specifications with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

REMOVE

3.0-1
3.0-2
3.0-3
3.0-4
3.7.2-1
3.8.4-3
3.8.9-2
3.8.9-3

INSERT

3.0-1
3.0-2
3.0-3
3.0-4
3.7.2-1
3.8.4-3
3.8.9-2
3.8.9-3

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, NSPM to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility;
 - (6) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to transfer byproduct materials from other job sites owned by NSPM for the purpose of volume reduction and decontamination.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

NSPM is authorized to operate the facility at steady state reactor core power levels not in excess of 1677 megawatts thermal.
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 252, are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.
 - (3) Physical Protection

NSPM shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains

- (3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, NSPM to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (4) Pursuant to the Act and 10 CFR Parts 30, 40, and 70, NSPM to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument and equipment calibration or associated with radioactive apparatus or components;
 - (5) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility;
 - (6) Pursuant to the Act and 10 CFR Parts 30 and 70, NSPM to transfer byproduct materials from other job sites owned by NSPM for the purposes of volume reduction and decontamination.
- C. This renewed operating license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:
- (1) Maximum Power Level

NSPM is authorized to operate the facility at steady state reactor core power levels not in excess of 1677 megawatts thermal.
 - (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 240, are hereby incorporated in the renewed operating license. NSPM shall operate the facility in accordance with the Technical Specifications.
 - (3) Physical Protection

NSPM shall fully implement and maintain in effect all provisions of the Commission-approved physical security, guard training and qualification, and safeguards contingency plans including amendments made pursuant to provisions of the Miscellaneous Amendments and Search Requirements revisions to 10 CFR 73.55 (51 FR 27817 and 27822) and to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The combined set of plans, which contains

3.0 LIMITING CONDITION FOR OPERATION (LCO) APPLICABILITY

LCO 3.0.1	LCOs shall be met during the MODES or other specified conditions in the Applicability, except as provided in LCO 3.0.2 and 3.0.7.
LCO 3.0.2	Upon discovery of a failure to meet an LCO, the Required Actions of the associated Conditions shall be met, except as provided in LCO 3.0.5, LCO 3.0.6 and LCO 3.0.8. If the LCO is met or is no longer applicable prior to expiration of the specified Completion Time(s), completion of the Required Action(s) is not required unless otherwise stated.
LCO 3.0.3	When an LCO is not met and the associated ACTIONS are not met, an associated ACTION is not provided, or if directed by the associated ACTIONS, the unit shall be placed in a MODE or other specified condition in which the LCO is not applicable. Within 6 hours perform a risk assessment addressing inoperable systems and components and implement appropriate risk management actions. If the risk assessment determines that continuing operation is acceptable, the appropriate risk management actions are implemented, and entry into LCO 3.0.3 was unplanned, then action shall be initiated within 24 hours to place the unit, as applicable, in: - MODE 3 within 30 hours; - MODE 4 within 36 hours; and - MODE 5 within 60 hours. If the risk assessment determines that continuing operation is not acceptable, the risk assessment was not completed, the appropriate risk management actions were not implemented, or entry into LCO 3.0.3 was planned, then action shall be initiated to place the unit, as applicable, in:

3.0 LCO APPLICABILITY (continued)

LCO 3.0.3 (continued)

- a. MODE 3 within 12 hours,
- b. MODE 4 within 18 hours, and
- c. MODE 5 within 42 hours.

All times are determined from entry into LCO 3.0.3.

Exceptions to this Specification are stated in the individual Specifications.

If corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, then completion of the actions required by LCO 3.0.3 is not required.

LCO 3.0.3 is only applicable in MODES 1, 2, 3, and 4.

LCO 3.0.4

When an LCO is not met, entry into a MODE or other specified condition in the Applicability shall only be made:

- a. When the associated ACTIONS to be entered permit continued operation in the MODE or other specified condition in the Applicability for an unlimited period of time;
 - b. After performance of a risk assessment addressing inoperable systems and components, consideration of the results, determination of the acceptability of entering the MODE or other specified condition in the Applicability, and establishment of risk management actions, if appropriate (exceptions to this Specification are stated in the individual Specifications); or
 - c. When an allowance is stated in the individual value, parameter, or other Specification.
-

3.0 LCO APPLICABILITY (continued)

LCO 3.0.4
(continued) This Specification shall not prevent changes in MODES or other specified conditions in the Applicability that are required to comply with ACTIONS or that are part of a shutdown of the unit.

LCO 3.0.5 Equipment removed from service or declared inoperable to comply with ACTIONS may be returned to service under administrative control solely to perform testing required to demonstrate its OPERABILITY or the OPERABILITY of other equipment. This is an exception to LCO 3.0.2 for the system returned to service under administrative control to perform the testing required to demonstrate OPERABILITY.

LCO 3.0.6 When a supported system LCO is not met solely due to a support system LCO not being met, the Conditions and Required Actions associated with this supported system are not required to be entered. Only the support system LCO ACTIONS are required to be entered.

This is an exception to LCO 3.0.2 for the supported system. In this event, an evaluation shall be performed in accordance with Specification 5.5.13, “Safety Function Determination Program (SFDP).” If a loss of safety function is determined to exist by this program, the appropriate Conditions and Required Actions of the LCO in which the loss of safety function exists are required to be entered.

When a support system's Required Action directs a supported system to be declared inoperable or directs entry into Conditions and Required Actions for a supported system, the applicable Conditions and Required Actions shall be entered in accordance with LCO 3.0.2.

3.0 LCO APPLICABILITY

LCO 3.0.7 Test Exception LCOs 3.1.8 and 3.4.18 allow specified Technical Specification (TS) requirements to be changed to permit performance of special tests and operations. Unless otherwise specified, all other TS requirements remain unchanged. Compliance with Test Exception LCOs is optional. When a Test Exception LCO is desired to be met but is not met, the ACTIONS of the Test Exception LCO shall be met. When a Test Exception LCO is not desired to be met, entry into a MODE or other specified condition in the Applicability shall be made in accordance with the other applicable Specifications.

LCO 3.0.8 When a Technical Specification supported system LCO is not met solely due to the inoperability of a non-Technical Specification support system listed below, the Technical Specification supported system LCO is considered to be met unless the associated delay time of the non-Technical Specification support system has expired. This is an exception to LCO 3.0.2 for the Technical Specification. Upon expiration of the non-Technical Specification support system delay time, the Technical Specification supported system shall be declared inoperable and the applicable Conditions and Required Actions for the Technical Specifications supported system shall be entered in accordance with LCO 3.0.2.

<u>Non-Technical Specification Support System</u>	<u>Delay Time</u>
Snubbers	72 hours

LCO 3.0.9 Unless specifically noted, all the information provided in the LCO including the associated ACTION requirements shall apply to each unit individually. Whenever certain portions of a specification refer to systems, components, operating parameters, setpoints, etc., which are different for each unit, this will be identified in parentheses or notes or in the Applicability section as appropriate.

3.7 PLANT SYSTEMS

3.7.2 Main Steam Isolation Valves (MSIVs)

LCO 3.7.2 Two MSIVs shall be OPERABLE.

APPLICABILITY: MODE 1,
MODES 2 and 3 except when both MSIVs are closed.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One MSIV inoperable in MODE 1.	A.1 Restore MSIV to OPERABLE status.	8 hours <u>OR</u> In accordance with the Risk Informed Completion Time Program
B. More than one MSIV inoperable in MODE 1. <u>OR</u> Required Action and associated Completion Time of Condition A not met.	B.1 Be in MODE 2.	6 hours

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. One DC electrical power subsystem inoperable for reasons other than Condition A or B.	C.1 Restore DC electrical power subsystem to OPERABLE status.	2 hours <u>OR</u> In accordance with the Risk Informed Completion Time Program
D. Required Action and Associated Completion Time of Condition A, B, or C not met.	D.1 Be in MODE 3. <u>AND</u> D.2 Be in MODE 5.	6 hours 36 hours
E. Two DC electrical power subsystems inoperable.	E.1 Be in MODE 3. <u>AND</u> E.2 Be in MODE 5.	6 hours 36 hours

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. One or more safeguards DC electrical power distribution subsystems inoperable.	B.1 Restore safeguards DC electrical power distribution subsystems to OPERABLE status.	2 hours <u>OR</u> In accordance with the Risk Informed Completion Time Program
C. One Reactor Protection Instrument AC panel inoperable.	C.1 Restore Reactor Protection Instrument AC panel to OPERABLE status.	2 hours <u>OR</u> In accordance with the Risk Informed Completion Time Program
D. Required Action and associated Completion Time of Condition A, B, or C not met.	D.1 Be in MODE 3. <u>AND</u> D.2 Be in MODE 5.	6 hours 36 hours

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
E. Two trains with inoperable distribution subsystems that result in a loss of safety function. <u>OR</u> Two or more Reactor Protection Instrument AC panels inoperable.	E.1 Be in MODE 3.	6 hours
	<u>AND</u> E.2 Be in MODE 5.	36 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.8.9.1 Verify correct breaker and switch alignments and voltage to safeguards AC, DC, and Reactor Protection Instrument AC electrical power distribution subsystems.	In accordance with the Surveillance Frequency Control Program



UNITED STATES
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SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 252 AND 240

PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2

RENEWED FACILITY OPERATING LICENSE NOS. DPR-42 AND DPR-60

DOCKET NOS. 50-282 AND 50-306

1.0 INTRODUCTION

By letter dated June 23, 2026 (Agencywide Documents Access and Management System Accession No. ML26174A386), Northern States Power Company, a Minnesota corporation, doing business as Xcel Energy (the licensee) requested changes to the technical specifications (TSs) for Prairie Island Nuclear Generating Plant Units 1 and 2 (Prairie Island) by license amendment request (LAR, application). In its application, the licensee requested that the U.S. Nuclear Regulatory Commission (NRC, the Commission) process the proposed amendment under the Consolidated Line Item Improvement Process (CLIIP). The proposed changes would revise the TS based on Technical Specifications Task Force (TSTF) Traveler TSTF-585, Revision 5, "Revise LCO [Limiting Condition for Operation] 3.0.3 to Require Managing Risk," (ML25260A487) and the associated NRC staff safety evaluation (SE) of Traveler TSTF-585 (ML26005A205).

The proposed changes would revise the shutdown requirements in LCO 3.0.3 by requiring the assessment and management of plant risk whenever LCO 3.0.3 is entered. The licensee proposed that the risk assessment and implementation of any appropriate risk management actions must be completed within 6 hours. For three of the existing TS Required Actions, the proposed change also revises or adds TS Required Actions to require a plant shutdown instead of entry into LCO 3.0.3.

A notice of consideration of proposed issuance of amendments with proposed no significant hazards consideration determination was published in the *Federal Register* on July 21, 2026 (91 FR 45833), and there has been no public comment on such finding.

Description of the Current LCO 3.0.3

The current TS LCO 3.0.3 requires action to be initiated within 1-hour to place the unit, as applicable, in Mode 3 within 7 hours, Mode 4 within 13 hours, and Mode 5 within 37 hours when:

- 1) an LCO is not met and the associated actions are not met,
- 2) an associated action is not provided, or

3) if directed to enter LCO 3.0.3 by the associated actions.

LCO 3.0.3 is applicable in Modes 1, 2, 3, and 4.

1.1 Proposed TS Changes to Adopt TSTF-585

In accordance with the NRC staff-approved TSTF-585, the licensee proposed changes that restructure LCO 3.0.3. The 1-hour shutdown preparation time is proposed to be replaced with a requirement to assess the risk and implement appropriate risk management actions (RMAs) within 6 hours. In addition, if risk is assessed and managed, and is subsequently determined to be acceptable, operation may continue for a total of 24 hours after unplanned entry into LCO 3.0.3 (i.e., an additional 18 hours beyond the initial 6 hours for the risk assessment and implementation of RMAs). If the plant is still in LCO 3.0.3 at the end of the 24-hour period, the initiation of a shutdown is required. In addition, a shutdown is required to be initiated at the end of the 6-hour period “[i]f the risk assessment determines that continuing operation is not acceptable, the risk assessment was not completed, the appropriate risk management actions were not implemented, or entry into LCO 3.0.3 was planned.”

The licensee also proposes to change the following sentence in LCO 3.0.3:

Where corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, completion of the actions required by LCO 3.0.3 is not required.

to read:

If corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, then completion of the actions required by LCO 3.0.3 is not required.

Each of the specified Mode change completion times (CTs) in the current LCO 3.0.3 (a, b, and c) are adjusted in the proposed LCO 3.0.3 to reflect the replacement of the existing 1-hour shutdown preparation time with the proposed increased time for initiating a shutdown of either 6 or 24 hours, as appropriate. The change in CTs does not change the amount of time allowed for the process of shutting down the plant. It reflects a change when the shutdown is required to begin based on the time of entry into LCO 3.0.3.

Table 1 shows the current LCO 3.0.3 compared to the proposed changes to LCO 3.0.3. Text deletions are shown in strikeout font, and additions are shown in bold italicized font.

Table 1
Comparison of Existing LCO 3.0.3 to the Proposed LCO 3.0.3

Current LCO 3.0.3	Proposed LCO 3.0.3
When an LCO is not met and the associated ACTIONS are not met, an associated ACTION is not provided, or if directed by the associated ACTIONS, the unit shall be placed in a MODE or other specified condition in which the LCO is not applicable. Action shall be initiated within 1 hour to place the unit, as applicable, in: a. MODE 3 within 7 hours, b. MODE 4 within 13 hours, and c. MODE 5 within 37 hours. Exceptions to this Specification are stated in the individual Specifications. Where corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, completion of the actions required by LCO 3.0.3 is not required. ...	When an LCO is not met and the associated ACTIONS are not met, an associated ACTION is not provided, or if directed by the associated ACTIONS, the unit shall be placed in a MODE or other specified condition in which the LCO is not applicable. <i>Within 6 hours perform a risk assessment addressing inoperable systems and components and implement appropriate risk management actions. If the risk assessment determines that continuing operation is acceptable, the appropriate risk management actions are implemented, and entry into LCO 3.0.3 was unplanned, then action</i> Action shall be initiated within 24 hours 1 hour to place the unit, as applicable, in: a. MODE 3 within 307 hours; b. MODE 4 within 3643 hours; and c. MODE 5 within 6037 hours. <i>If the risk assessment determines that continuing operation is not acceptable, the risk assessment was not completed, the appropriate risk management actions were not implemented, or entry into LCO 3.0.3 was planned, then action shall be initiated to place the unit, as applicable, in:</i> <i>a. MODE 3 within 12 hours,</i> <i>b. MODE 4 within 18 hours, and</i> <i>c. MODE 5 within 42 hours.</i> <i>All times are determined from entry into LCO 3.0.3.</i> Exceptions to this Specification are stated in the individual Specifications. If Where corrective measures are completed that permit operation in accordance with the LCO or ACTIONS, <i>then</i> completion of the actions required by LCO 3.0.3 is not required. ...

Implicit entries into LCO 3.0.3 can occur when an LCO is not met, and there is no associated Action provided for that condition. There are also several places in the TS where entries into LCO 3.0.3 are explicitly required by the TS Actions. In some cases, the application of the revised LCO 3.0.3 would not be appropriate. Accordingly, TSTF-585 modified the standard TS (STS) to replace these implicit and explicit entries into LCO 3.0.3 with explicit requirements to immediately shutdown. The TSTF-585 changes that are applicable to Prairie Island are:

- TS 3.7.2, “Main Steam Isolation Valves (MSIVs)” would be modified to require placing the unit in Mode 2 when more than one MSIV is inoperable while in Mode 1.
- TS 3.8.4, “DC [direct current] Sources – Operating” would be modified to require shutdown if two DC electrical power subsystem were inoperable.
- TS 3.8.9, “Distribution Systems – Operating” would be modified to require a shutdown when two trains with inoperable distribution subsystems that result in a loss of safety function or when two or more Reactor Protection Instrument AC [alternate current] panels were inoperable.

2.0 REGULATORY EVALUATION

2.1 Applicable Regulatory Requirements and Guidance

The regulation under Title 10 of the *Code of Federal Regulations* (10 CFR) 50.36(b) requires that:

Each license authorizing operation of a ... utilization facility ... will include technical specifications. The technical specifications will be derived from the analyses and evaluation included in the safety analysis report, and amendments thereto, submitted pursuant to [10 CFR] 50.34 [“Contents of applications; technical information”]. The Commission may include such additional technical specifications as the Commission finds appropriate.

The categories of items required to be in the TS are listed in 10 CFR 50.36(c). The regulation at 10 CFR 50.36(c)(2) requires that TS include LCOs. Per 10 CFR 50.36(c)(2)(i), LCOs “are the lowest functional capability or performance levels of equipment required for safe operation of the facility.” The regulation also requires that when an LCO of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the TS until the condition can be met.

LCO 3.0.3 implements 10 CFR 50.36(c)(2)(i) in that it requires a plant to shut down when an LCO is not met and: 1) there are no remedial actions provided in the TS, or 2) the remedial actions provided in the TS are not completed in the time allowed by the TS.

Paragraph (a)(4) of 10 CFR 50.65, "Requirements for monitoring the effectiveness of maintenance at nuclear power plants," (the Maintenance Rule) requires, in part, that:

[b]efore performing maintenance activities (including but not limited to surveillance, post-maintenance testing, and corrective and preventive maintenance), the licensee shall assess and manage the increase in risk that may result from the proposed maintenance activities. The scope of the assessment may be limited to structures, systems, and components that a risk-informed evaluation process has shown to be significant to public health and safety.

NUMARC 93-01, "Industry Guideline for Monitoring the Effectiveness of Maintenance at Nuclear Power Plants," (ML18120A069) section 11.0, "Assessment of Risk Resulting from Maintenance Activities," provides a methodology that is an acceptable approach to manage and assess risk associated with maintenance activities. This methodology has been endorsed by the NRC in Regulatory Guide (RG) 1.160, Revision 4, "Monitoring the Effectiveness of Maintenance at Nuclear Power Plants" (ML18220B281). TSTF-585 revised STS LCO 3.0.3 to require the assessment and management of risk using this NRC-endorsed methodology.

The NRC staff's guidance for the review of TS is in NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR [Light-Water Reactor] Edition" (SRP), Chapter 16, Section 16.0, "Technical Specifications," March 2010 (ML100351425). The NRC staff's review includes consideration of whether the proposed changes are consistent with the NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Volume 1, "Specifications," and Volume 2, "Bases," Revision 5, September 2021 (ML21259A155 and ML21259A159, respectively).

3.0 TECHNICAL EVALUATION

3.1 Proposed TS Changes to Adopt TSTF-585

The requirements and guidance the NRC staff used to determine the acceptability of the licensee's proposed changes are listed in section 2.1 of this SE. The NRC staff compared the licensee's proposed TS changes, summarized in section 1.1 of this SE, against the changes approved in TSTF-585. In accordance with SRP Chapter 16.0, the NRC staff determined that the STS changes approved in TSTF-585 are applicable to Prairie Island TSs because it is a Westinghouse design and the NRC staff approved the TSTF-585 changes for that plant design. The NRC staff confirmed that there are no plant-specific design features that would affect the applicability of the proposed TSTF-585 changes to Prairie Island.

TSTF-585 proposed a restructured STS LCO 3.0.3 by replacing the 1-hour shutdown preparation time with a requirement to assess the risk and implement appropriate RMAs within 6 hours. Additionally, TSTF-585 revised or added some TS Required Actions to require a plant shutdown instead of entry into STS LCO 3.0.3, based on whether the TS directs entry into LCO 3.0.3 or could result from an implicit entry into STS LCO 3.0.3. These changes were made in cases where immediate shutdown was more appropriate than the delayed shutdown allowed by the revised LCO 3.0.3. The NRC staff finds that the licensee's proposed changes to Prairie Island TS in section 1.1 of this SE are consistent with those found acceptable in TSTF-585.

The licensee also confirmed that it will use NUMARC 93-01, section 11.0, "Assessment of Risk Resulting from Maintenance Activities," as its methodology to assess and manage risk when entering LCO 3.0.3. This methodology was endorsed by the NRC in RG 1.160, Revision 4, and was found acceptable by the staff in the SE of TSTF-585 for assessing and managing risk when entering LCO 3.0.3.

In the NRC SE of TSTF-585, the NRC staff concluded that the proposed changes were acceptable because the proposed STS LCO 3.0.3 provides appropriate remedial actions in lieu of requiring an immediate shutdown. The proposed STS LCO 3.0.3 ensures that appropriate remedial actions are provided for the requested CT, of up to 24 hours, and the appropriate time for remedial actions are provided based on the risk assessment result. If the required systems are not restored to operable status within the specified CT, a shutdown is required. Therefore, the NRC staff found that the proposed changes to STS LCO 3.0.3 were acceptable because they continued to meet the requirements of 10 CFR 50.36(c)(2)(i) as discussed in section 3.0 of the NRC staff's SE of TSTF-585.

For these same reasons, the NRC staff concludes that the corresponding proposed changes to the Prairie Island TS in section 1.1 of this SE continue to meet the requirements of 10 CFR 50.36(c)(2)(i).

3.2 TS Change Consistency

The NRC staff reviewed the proposed TS changes for technical clarity and consistency with the existing requirements for customary terminology and formatting. The NRC staff finds that the proposed changes are consistent with chapter 16.0 of the SRP and are therefore acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Minnesota State official was notified of the proposed issuance of the amendments on August 4, 2026. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments relate to authorizations under, or changes to requirements in 10 CFR Part 50 with respect to installation or use of a facility component. The NRC staff has determined that any ground disturbance is limited to previously disturbed areas. Additionally, the NRC staff has determined that the action involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, no significant increase in individual or cumulative public or occupational radiation exposure, and no significant increase in the potential for or consequences from radiological accidents. Finally, the NRC staff has determined that a categorical exclusion applies and that special circumstances under 10 CFR 51.22, "Categorical exclusions," are not present that would preclude reliance on the categorical exclusion. Accordingly, this action meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(d)(8). Pursuant to 10 CFR 51.22, no environmental impact statement or environmental assessment need be prepared in connection with the action.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by

operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: September 28, 2026

SUBJECT: PRAIRIE ISLAND NUCLEAR GENERATING PLANT, UNITS 1 AND 2 -
ISSUANCE OF AMENDMENT NOS. 252 AND 240 RE: ADOPTION OF
TSTF-585 "REVISE LCO 3.0.3 TO REQUIRE MANAGING RISK"
(EPID L-2026-LLA-0128) DATED SEPTEMBER 28, 2026

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