



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 13, 2026

Mr. Ken J. Peters
Executive Vice President and
Chief Nuclear Officer
Attention: Regulatory Affairs
Vistra Operations Company LLC
Comanche Peak Nuclear Power Plant
6322 N FM 56
P.O. Box 1002
Glen Rose, TX 76043

SUBJECT: COMANCHE PEAK NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2 -
ISSUANCE OF AMENDMENT NOS. 198 AND 198, RESPECTIVELY RE:
APPLICATION TO REVISE TECHNICAL SPECIFICATIONS TO REMOVE
EXPIRED NOTE AND TECHNICAL SPECIFICATION (EPID L-2026-LLA-0051)

Dear Mr. Peters:

The U.S. Nuclear Regulatory Commission (the Commission) has issued Amendment No. 198 to Renewed Facility Operating License No. NPF-87 and Amendment No. 198 to Renewed Facility Operating License No. NPF-89 for Comanche Peak Nuclear Power Plant (Comanche Peak) Unit Nos. 1 and 2, respectively. The amendments consist of changes to the technical specifications (TSs) in response to your application dated November 11, 2025 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML25315A015).

The amendments modify the Comanche Peak TSs by removing a note and TSs that have expired relevancy. Specifically, the changes remove TS 3.2.1A "Heat Flux Hot Channel ($F_Q(Z)$) (RAOC [Relaxed Axial Offset Control]-W(Z) Methodology)." The change also removes the unit specific note from TS 3.2.1B "Heat Flux Hot Channel ($F_Q(Z)$) (RAOC-T(Z) Methodology)" and renumbers TS 3.2.1B as TS 3.2.1.

A copy of the related Safety Evaluation is enclosed. Notice of Issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

William Orders, Senior Project Manager
Operating Reactor Licensing Branch 4
Division of Reactor Licensing Projects I
Office of Nuclear Reactor Regulation

Docket Nos. 50-445 and 50-446

Enclosures:

1. Amendment No. 198 to NPF-87
2. Amendment No. 198 to NPF-89
3. Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMANCHE PEAK POWER COMPANY LLC

AND VISTRA OPERATIONS COMPANY LLC

COMANCHE PEAK NUCLEAR POWER PLANT, UNIT NO. 1

DOCKET NO. 50-445

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 198
Renewed License No. NPF-87

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Vistra Operations Company LLC (Vistra OpCo) dated November 11, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-87 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 198 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this renewed license. Vistra OpCo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. The license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Hipólito González, Chief
Operating Reactor Licensing Branch 4
Division of Reactor Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and
Technical Specifications

Date of Issuance: August 13, 2026



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

COMANCHE PEAK POWER COMPANY LLC

AND VISTRA OPERATIONS COMPANY LLC

COMANCHE PEAK NUCLEAR POWER PLANT, UNIT NO. 2

DOCKET NO. 50-446

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 198
Renewed License No. NPF-89

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Vistra Operations Company LLC (Vistra OpCo) dated November 11, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-89 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 198 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this renewed license. Vistra OpCo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 60 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Hipólito González, Chief
Operating Reactor Licensing Branch 4
Division of Reactor Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and
Technical Specifications

Date of Issuance: August 13, 2026

ATTACHMENT TO LICENSE AMENDMENT NO. 198
TO RENEWED FACILITY OPERATING LICENSE NO. NPF-87
AND AMENDMENT NO. 198
TO RENEWED FACILITY OPERATING LICENSE NO. NPF-89
COMANCHE PEAK NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-445 AND 50-446

Replace the following pages of Renewed Facility Operating License Nos. NPF-87 and NPF-89, and the Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Renewed Facility Operating License No. NPF-87

REMOVE
3

INSERT
3

Renewed Facility Operating License No. NPF-89

REMOVE
3

INSERT
3

Technical Specifications

REMOVE

INSERT

-----	3.2-1
-----	3.2-2
-----	3.2-3
-----	3.2-4
-----	3.2-5
-----	3.2-6
3.2-1A	-----
3.2-2A	-----
3.2-3A	-----
3.2-4A	-----
3.2-5A	-----
3.2-6A	-----
3.2-1B	-----
3.2-2B	-----
3.2-3B	-----
3.2-4B	-----
3.2-5B	-----
3.2-6B	-----

- (3) Vistra OpCo, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time, special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, and described in the Final Safety Analysis Report, as supplemented and amended;
 - (4) Vistra OpCo, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use, at any time, any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (5) Vistra OpCo, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required, any byproduct, source, and special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (6) Vistra OpCo, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Vistra OpCo is authorized to operate the facility at reactor core power levels not in excess of 3458 megawatts thermal through Cycle 13 and 3612 megawatts thermal starting with Cycle 14 in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 198 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this renewed license. Vistra OpCo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Antitrust Conditions

DELETED

(4) License Transfer

The CP PowerCo Decommissioning Master Trust Agreement for the facility at the time the license transfers are effected and thereafter, is subject to the following:

- (2) CP PowerCo, pursuant to 10 CFR Part 50, to possess the facility at the designated location in Somervell County, Texas in accordance with the procedures and limitations set forth in this renewed license;
 - (3) Vistra OpCo, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time, special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, and described in the Final Safety Analysis Report, as supplemented and amended;
 - (4) Vistra OpCo, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use, at any time, any byproduct, source, and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
 - (5) Vistra OpCo, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required, any byproduct, source, and special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
 - (6) Vistra OpCo, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

Vistra OpCo is authorized to operate the facility at reactor core power levels not in excess of 3458 megawatts thermal through Cycle 11 and 3612 megawatts thermal starting with Cycle 12 in accordance with the conditions specified herein.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 198 and the Environmental Protection Plan contained in Appendix B, are hereby incorporated into this renewed license. Vistra OpCo shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Antitrust Conditions

DELETED

(4) License Transfer

3.2 POWER DISTRIBUTION LIMITS

3.2.1 Heat Flux Hot Channel (F_Q(Z)) (RAOC-T(Z) Methodology)

LCO 3.2.1 F_Q(Z) , as approximated by F_Q^C(Z) and F_Q^W(Z) , shall be within the limits specified in the COLR.

APPLICABILITY: MODE 1

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. -----NOTE----- Required Action A.4 shall be completed whenever this Condition is entered prior to increasing THERMAL POWER above the limit of Required Action A.1. SR 3.2.1.2 is not required to be performed if this Condition is entered prior to THERMAL POWER exceeding 75% RTP after a refueling. ----- F_Q^C(Z) not within limit.	A.1 Reduce THERMAL POWER ≥ 1% RTP for each 1% F_Q^C(Z) exceeds limit. <u>AND</u> A.2 Reduce Power Range Neutron Flux-- High trip setpoints ≥ 1% for each 1% that THERMAL POWER is limited below RATED THERMAL POWER by Required Action A.1. <u>AND</u>	15 minutes after each F_Q^C(Z) determination 72 hours after each F_Q^C(Z) determination

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. (continued)	A.3 Reduce Overpower N-16 trip setpoints $\geq 1\%$ for each 1% that THERMAL POWER is limited below RATED THERMAL POWER by Required Action A.1. <u>AND</u> A.4 Perform SR 3.2.1.1 and SR 3.2.1.2.	72 hours after each F_Q^C(Z) determination Prior to increasing THERMAL POWER above the limit of Required Action A.1

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. F _Q ^W (Z) not within limits.	B.1.1 Implement a RAOC operating space specified in the COLR that restores F _Q ^W (Z) to within its limits.	4 hours
	<u>AND</u>	
	B.1.2 Perform SR 3.2.1.1 and SR 3.2.1.2 if control rod motion is required to comply with the new operating space.	72 hours
	<u>OR</u>	
	B.2.1 -----NOTE----- Required Action B.2.4 shall be completed whenever Required Action B.2.1 is performed prior to increasing THERMAL POWER above the limit of Required Action B.2.1. ----- Limit THERMAL POWER to less than RATED THERMAL POWER and reduce AFD limits as specified in the COLR.	4 hours after each F _Q ^W (Z) determination
	<u>AND</u>	
	B.2.2 Reduce Power Range Neutron Flux-High trip setpoints ≥ 1% for each 1% that THERMAL POWER is limited below RATED THERMAL POWER by Required Action B.2.1.	72 hours after each F _Q ^W (Z) determination
	<u>AND</u>	

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. (continued)	B.2.3 Reduce Overpower N-16 trip setpoints $\geq 1\%$ for each 1% that THERMAL POWER is limited below RATED THERMAL POWER by Required Action B.2.1. <u>AND</u> B.2.4 Perform SR 3.2.1.1 and SR 3.2.1.2.	72 hours after each F_Q^W(Z) determination Prior to increasing THERMAL POWER above the limit of Required Action B.2.1.
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 2.	6 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.2.1.1	Verify $F_Q^C(Z)$ is within limit.	Once after each refueling prior to THERMAL POWER exceeding 75% RTP <u>AND</u> Once within 24 hours after achieving equilibrium conditions after exceeding, by $\geq 20\%$ RTP, the THERMAL POWER at which $F_Q^C(Z)$ was last verified <u>AND</u> In accordance with the Surveillance Frequency Control Program.

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.2.1.2	Verify $F_Q^W(Z)$ is within limit.	Once after each refueling within 24 hours after THERMAL POWER exceeds 75% RTP. <u>AND</u> Once within 24 hours after achieving equilibrium conditions after exceeding, by $\geq 20\%$ RTP, the THERMAL POWER at which $F_Q^W(Z)$ was last verified <u>AND</u> In accordance with the Surveillance Frequency Control Program.



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NO. 198 TO

RENEWED FACILITY OPERATING LICENSE NO. NPF-87

AND AMENDMENT NO. 198 TO

RENEWED FACILITY OPERATING LICENSE NO. NPF-89

COMANCHE PEAK POWER COMPANY LLC

AND VISTRA OPERATIONS COMPANY LLC

COMANCHE PEAK NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

DOCKET NOS. 50-445 AND 50-446

1.0 INTRODUCTION

By letter dated November 11, 2025, Vistra Operations Company LLC (the licensee) submitted a license amendment request (LAR) for changes to the Comanche Peak Nuclear Power Plant (Comanche Peak), Unit Nos. 1 and 2, Technical Specifications (TSs). The requested changes would modify the Comanche Peak TSs by removing a note and TSs that have expired relevancy. Specifically, the changes would remove, in its entirety, TS 3.2.1A "Heat Flux Hot Channel ($F_Q(Z)$) (RAOC [relaxed axial offset control]-W(Z) Methodology)." The changes also would remove the unit-specific note from TS 3.2.1B "Heat Flux Hot Channel ($F_Q(Z)$) (RAOC-T(Z) Methodology)" and renumber TS 3.2.1B as TS 3.2.1.

A notice of consideration of proposed issuance of amendments with proposed no significant hazards consideration determination was published in the *Federal Register* on June 9, 2026 (91 FR 34842) and there has been no public comment on such finding.

2.0 REGULATORY EVALUATION

The regulation in Title 10 of the *Code of Federal Regulations* (10 CFR) Section 50.36, "Technical specifications," provides the regulatory requirements for the content of the TS. It requires, in part, that a summary statement of the bases for such specifications shall be included by applicants for a license authorizing operation of a production or utilization facility. Specifically, 10 CFR 50.36(c) requires that TS include items in five specific categories related to station operation. These categories are (1) safety limits, limiting safety system settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements; (4) design features; and (5) administrative controls.

2.1 Description of Changes

Currently, Comanche Peak TSs describe two heat flux hot channel methodologies. In TS 3.2.1A, which is only applicable to Unit 1 Cycle 24, and TS 3.2.1B, which is not applicable to Unit 1 Cycle 24. The licensee proposed in the LAR to delete TS 3.2.1A in its entirety and renumber TS 3.2.1B as TS 3.2.1 with corresponding page numbering changes. The proposed changes also delete the Note in LCO 3.2.1B that prohibits its applicability to Unit 1 during Cycle 24.

2.2 Reason for the Changes

The licensee stated in the LAR that Comanche Peak, Unit 1 has begun Cycle 25; therefore, TS 3.2.1A is no longer required and TS 3.2.1B is currently applicable to both units. Additionally, the LCO 3.2.1B Note that prevents applicability during Unit 1 Cycle 24 has expired.

3.0 TECHNICAL EVALUATION

The NRC staff reviewed the proposed changes to delete TS 3.2.1A, renumber TS 3.2.1B as TS 3.2.1, and delete the Note from LCO 3.2.1B. The NRC approved the licensee's use of two versions of TS 3.2.1 for unit-wise implementation in License Amendment Nos. 184 and 184 (ML23313A073), dated December 6, 2023. The NRC approved use of (1) TS 3.2.1B for both units and (2) TS 3.2.1A to retain Unit 1's current requirements at the time to allow completion of Cycle 24 before implementing the requirements in TS 3.2.1B. The NRC staff notes that since Unit 1 has completed Cycle 24, TS 3.2.1A is now an expired requirement, inapplicable to either unit. Likewise, the LCO 3.2.1B Note, which states, "[t]his LCO is NOT applicable to Unit 1 Cycle 24," is also expired.

The NRC staff finds deletion of TS 3.2.1A and the Note in current LCO 3.2.1B acceptable because they are expired requirements that are no longer applicable. LCO 3.2.1B would be retained in the TSs for Units 1 and 2 and the requirements of 10 CFR 50.36 continue to be met. The associated proposed changes to renumber TS 3.2.1B to TS 3.2.1 are administrative in nature and are acceptable because they do not substantively alter TS requirements.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Texas State official was notified of the proposed issuance of the amendments on August 2, 2026. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

This action relates to authorizations under 10 CFR Part 50 with respect to installation or use of a facility component. The NRC staff has determined that any ground disturbance is limited to previously disturbed areas. Additionally, the NRC staff has determined that the action involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, no significant increase in individual or cumulative public or occupational radiation exposure, and no significant increase in the potential for or consequences from radiological accidents. Finally, the NRC staff has determined that a categorical exclusion applies and that special circumstances under 10 CFR 51.22, "Categorical exclusions," are not present that would preclude reliance on the categorical exclusion. Accordingly, this action meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(d)(8). Pursuant

to 10 CFR 51.22, no environmental impact statement or environmental assessment need be prepared for this action in connection with the action.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: August 13, 2026

SUBJECT: COMANCHE PEAK NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2 -
ISSUANCE OF AMENDMENT NOS. 198 AND 198, RESPECTIVELY RE:
APPLICATION TO REVISE TECHNICAL SPECIFICATIONS TO REMOVE
EXPIRED NOTE AND TECHNICAL SPECIFICATION (EPID L-2026-LLA-0051)
DATED AUGUST 13, 2026

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TSweat, NRR

ADAMS Accession Nos.:

Package: ML26217A018

Amendment: ML26217A021

eConcurrence Case: 20260805-01709