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**Information Request
July 22, 2026
Notification of Inspection and Request for Information
Callaway Plant
NRC Inspection Report 05000483/2026004**

INSERVICE INSPECTION DOCUMENT REQUEST

Inspection Dates: October 1, 2026, to October 12, 2026
Inspector: Jim Drake

A. Information Requested for the In-Office Preparation Week

The information requested should be sent to the Region IV office in hard copy or electronic format or via a secure document management service, in care of Jim Drake, by September 4, 2026, to facilitate the selection of specific items that will be reviewed during the inspection. The inspector will select specific items from the information requested below and then request from your staff additional documents needed during the onsite inspection. We ask that the specific items selected from the lists be available and ready for review on the first day of inspection. Please provide the requested documentation electronically if possible. If the requested documents are large and only hard copy formats are available, please inform the inspector, and provide subject documentation during the first day of the onsite inspection.

If you have any questions regarding this information request, please call the inspector as soon as possible.

From October 1, 2026, to October 12, 2026, a reactor inspector from the Region IV office will perform the baseline inservice inspection at Callaway Plant, using NRC Inspection Procedure 71111.08, "Inservice Inspection Activities." Experience has shown that this inspection is a resource intensive inspection both for the NRC inspector and your staff. The date of this inspection may change depending on the outage schedule

you provide. To minimize the impact on your onsite resources and to ensure a productive inspection, we have enclosed a request for documents needed for this inspection. The information identified on this request (Section A) is to be provided prior to the inspection to ensure that the inspector(s) is adequately prepared. The section identified as "Documents Upon Request" is intended to provide guidance to the type of information an inspector(s) will be requesting to complete the inspection following the initial request. It is important that these documents are up to date and complete to minimize the number of additional documents requested during the preparation and/or the onsite portions of the inspection (i.e., condition reports with attachments).

We have discussed the schedule for these inspection activities with your staff and understand that our regulatory contact for this inspection will be Erin Smith and Jeff Kosky of your licensing organization. The tentative inspection schedule is as follows:

Preparation week: September 21 to September 25, 2026

Onsite weeks: October 1 to October 12, 2026

Our inspection dates are subject to change based on your updated schedule of outage activities. If there are any questions about this inspection or the material requested, please contact Jim Drake at (817) 200-1558. (email: James.Drake@nrc.gov)

A.1 Inservice Inspection / Welding Programs and Schedule Information

1. A detailed schedule (including preliminary dates) of:
 - 1.1. Nondestructive examinations planned for ASME Code Class Components performed as part of your ASME Section XI, risk informed (if applicable), and augmented inservice inspection programs during the upcoming outage.
 - 1.2. Examinations planned for Alloy 82/182/600 components that are not included in the Section XI scope (If applicable)
 - 1.3. Welding activities that are scheduled to be completed during the upcoming outage (ASME Class 1, 2, or 3 structures, systems, or components)
 - 1.4. Examinations planned as part of your boric acid corrosion control program (Mode 3 walkdowns, bolted connection walkdowns, etc.)
2. ASME Section XI Code Relief Requests and associated NRC safety evaluations applicable to the examinations identified above.
 - 2.1. A list of ASME Code Cases currently being used to include the system and/or component the Code Case is being applied to.
3. A list of nondestructive examination reports which have identified recordable or rejectable indications on any ASME Code Class components since the beginning of the last refueling outage. This should include the previous Section XI pressure test(s) conducted during start up and any evaluations associated with the results of the pressure tests.

4. A list including a brief description (e.g., system, code class, weld category, nondestructive examination performed) associated with the repair/replacement activities of any ASME Code Class component since the beginning of the last outage and/or planned this refueling outage.
5. If reactor vessel weld examinations required by the ASME Code are scheduled to occur during the upcoming outage, provide a detailed description of the welds to be examined and the extent of the planned examination. Please also provide reference numbers for applicable procedures that will be used to conduct these examinations.
6. Any 10 CFR Part 21 reports applicable to structures, systems, or components within the scope of Section XI of the ASME Code that have been identified since the beginning of the last refueling outage.
7. A list of any temporary non-code repairs in service (e.g., pinhole leaks).
8. Most recent self-assessments for the inservice inspection, welding, and Alloy 600 programs.
9. Most current revision of the inservice inspection program manual and plan for the current interval.
10. Nondestructive examination (NDE) procedures for NDE that will be used during the outage.
11. Overarching site procedure for welding.

A.2 Reactor Pressure Vessel Head

1. Provide a detailed scope of the planned bare metal visual examinations (e.g., volume coverage, limitations, etc.) of the vessel upper head penetrations and/or any nonvisual nondestructive examination of the reactor vessel head including the examination procedures to be used.
 - 1.1. Provide the records recording the extent of inspection for each penetration nozzle including documents which resolved interference or masking issues that confirm that the extent of examination meets 10 CFR 50.55a(g)(6)(ii)(D).
 - 1.2. Provide records that demonstrate that a volumetric or surface leakage path examination assessment was performed.
2. Provide current calculations for EDY, and RIY as defined in Code Case N-729-1 that establishes the volumetric and visual inspection frequency for the reactor vessel head and J-groove welds.

A.3 Boric Acid Corrosion Control Program

1. Procedures that govern the scope, equipment and implementation of the inspections required to identify boric acid leakage and the procedures for boric acid leakage/corrosion evaluation.

2. Please provide a list of leaks (including code class of the components) that have been identified since the last refueling outage and associated corrective action documentation. If during the last cycle, the unit was shutdown, please provide documentation of containment walkdown inspections performed as part of the boric acid corrosion control program.

A.4 Steam Generator (SG) Tube Inspections (If being inspected, otherwise N/A)

1. A detailed schedule of:
 - SG tube inspection, data analyses, and repair activities for the upcoming outage (if occurring).
 - SG secondary side inspection activities for the upcoming outage (if occurring).

Note: Indicate where the primary, secondary, and resolution analyses are scheduled to take place.

2. SG history documentation given to vendors performing eddy current (ET) testing of the SGs during the upcoming outage.
3. Current SG condition monitoring and operational assessment report. Also include a copy of the following documents as they become available for the current cycle:
 - Degradation assessment
 - Condition monitoring assessment
4. Define the planned SG ET scope (e.g., 100 percent of unrepaired tubes with bobbin probe and 20 percent sample of hot leg expansion transition regions with rotating probe) and identify the scope expansion criteria, which will be applied.
5. Identify and quantify any SG tube leakage experienced during the previous operating cycle. Also provide documentation identifying which SG was leaking and corrective actions completed and planned for this condition.
6. SG eddy current data analyst guidelines and site validated eddy current technique specification sheets. Additionally, please provide a copy of EPRI Appendix H, "Examination Technique Specification Sheets," qualification records.
7. Provide the guidance to be followed if a loose part or foreign material is identified in the steam generators.
8. Provide history of the condition and issues pertaining to the secondary side of the steam generators (including items such as loose parts, fouling, top of tube sheet condition, crud removal amounts, etc.).

A.5 Additional Information Related to all Inservice Inspection Activities

1. A list with a brief description of inservice inspection, and boric acid corrosion control program related issues (e.g., Condition Reports) entered into your corrective action program since the beginning of the last refueling outage. For example, a list based upon database searches using key words related to piping such as: inservice inspection, ASME Code, Section XI, NDE, cracks, wear, thinning, leakage, rust, corrosion, boric acid, or errors in piping examinations.
2. Provide training (e.g. Scaffolding, Fall Protection, FME, Confined Space) if they are required for the activities described in A.1 through A.3.
3. Please provide names and phone numbers for the following program leads:
 - Inservice inspection (examination, planning)
 - Containment exams
 - Snubbers and support
 - Repair and replacement program
 - Licensing
 - Site welding engineer
 - Reactor pressure vessel head exams
 - Boric acid corrosion control program
 - Steam generator inspection activities (site lead and vendor contact)

B. Documents Upon Request

B.1 Inservice Inspection / Welding Programs and Schedule Information

1. Updated schedules for inservice inspection/nondestructive examination activities, including planned welding activities, and schedule showing contingency repair plans, if available.
2. For ASME Code Class welds selected by the inspector please provide copies of the following documentation (as applicable) for each subject weld:
 - Weld data sheet (traveler)
 - Weld configuration and system location
 - Applicable code edition and addenda for weldment
 - Applicable code edition and addenda for welding procedures
 - Applicable welding procedure specifications (WPS) used to fabricate the welds
 - Procedure qualification records (PQR)
 - Welder's performance qualification (WPQ) records
 - Nonconformance reports for the selected welds (If applicable)
 - Radiographs of the selected welds and access to equipment to allow viewing radiographs (if radiographic testing was performed)

- Preservice and inservice examination records for the selected welds
 - Readily accessible copies of nondestructive examination personnel qualifications records for reviewing
3. For the inservice inspection related corrective action issues selected by the inspector from Section A of this enclosure, provide copies of the corrective actions and supporting documentation
 4. For the nondestructive examination reports with relevant conditions on ASME Code Class components selected by the inspector from Section A above, provide copies of the examination records, examiner qualification records, and associated corrective action documents.
 5. For the nondestructive examinations selected by the inspector from Section A of this enclosure, provide copies of the nondestructive examination procedures used to perform the examinations (including calibration and flaw characterization/sizing procedures).
 6. For ultrasonic examination procedures qualified in accordance with ASME Code, Section XI, Appendix VIII, provide documentation supporting the procedure qualification (e.g. the EPRI performance demonstration qualification summary sheets). Also, include qualification documentation of the specific equipment to be used (e.g., ultrasonic unit, cables, and transducers including serial numbers) and nondestructive examination personnel qualification records.

B.2 Reactor Pressure Vessel Head

1. Nondestructive personnel qualification records for the examiners who will perform examinations of the reactor pressure vessel head replacement. (If applicable)
2. Drawings showing the following:
 - Reactor pressure vessel head and control rod drive mechanism nozzle configurations
 - Reactor pressure vessel head insulation configuration

Note: The drawings listed above should include fabrication drawings for the nozzle attachment welds as applicable.

B.3 Boric Acid Corrosion Control Program

1. Boric acid walkdown inspection results, an updated list of boric acid leaks identified so far this outage, associated corrective action documentation, and overall status of planned boric acid inspections.
2. List of boric acid evaluation and corrective action documents associated with the leakage.

B.4 Codes and Standards

1. Ready access to (i.e., copies provided to the inspector(s) for use during the inspection at the onsite inspection location, or room number and location where available):
 - Applicable editions of the ASME Code (Sections V, IX, and XI) for the inservice inspection program and the repair/replacement program.
2. Performance demonstration initiative (PDI) generic procedures with the latest applicable revisions that support site qualified ultrasonic examinations of piping welds and components (e.g., PDI-UT-1, PDI-UT-2, PDI-UT-3, PDI-UT-10, etc.).