UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200
ATLANTA, GEORGIA 30303-1200

July 15, 2026

Jonathan Huecker
Site Vice President
Catawba Nuclear Station
Duke Energy Carolinas, LLC
4800 Concord Road
York, SC 29745-9635

SUBJECT: CATAWBA NUCLEAR STATION – INTEGRATED INSPECTION REPORT 05000413/2026002 AND 05000414/2026002

Dear Jonathan Huecker:

On June 30, 2026, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Catawba Nuclear Station. On July 7, 2026, the NRC inspectors discussed the results of this inspection with you and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at http://www.nrc.gov/reading-rm/adams.html and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

Signed by Robert Williams on 07/15/26

Robert E. Williams, Jr., Chief
Reactor Projects Branch D
Division of Reactor Projects, Region 2

Docket Nos. 05000413 and 05000414
License Nos. NPF-35 and NPF-52

Enclosure:
As stated

cc w/ encl: GovDelivery Subscriber

U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Numbers:	05000413 and 05000414
License Numbers:	NPF-35 and NPF-52
Report Numbers:	05000413/2026002 and 05000414/2026002
Enterprise Identifier:	I-2026-002-0023
Licensee:	Duke Energy Carolinas, LLC
Facility:	Catawba Nuclear Station
Location:	York, South Carolina
Inspection Dates:	April 1, 2026 to June 30, 2026
Inspectors:	B. Bowker, Senior Reactor Inspector D. Neal, Resident Inspector A. Nielsen, Senior Health Physicist D. Restrepo, Health Physicist D. Rivard, Senior Resident Inspector A. Wang, Resident Inspector
Approved By:	Robert E. Williams, Jr., Chief Reactor Projects Branch D Division of Reactor Projects, Region 2

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Catawba Nuclear Station, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to https://www.nrc.gov/reactors/operating/oversight.html for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

PLANT STATUS

Unit 1 began the inspection period at rated thermal power (RTP). On April 18, 2026, the unit was shut down for planned to refuel outage C1R29. The unit started up to Mode 2 on May 13, 2026, and returned to RTP on May 16, 2026. The unit operated at or near RTP for the remainder of the inspection period.

Unit 2 began the inspection period in maintenance outage C2F26A to repair a steam generator tube leak. The unit restarted to Mode 2 on April 14, 2026, and returned to RTP on April 16, 2026. The unit operated at or near RTP for the remainder of the inspection period.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, “Light-Water Reactor Inspection Program - Operations Phase.” The inspectors performed activities described in IMC 2515, Appendix D, “Plant Status,” observed risk-significant activities, and completed onsite portions of IPs. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71111.04 - Equipment Alignment

Partial Walkdown (IP Section 03.01) (1 Sample)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

(1) Unit 1 alternate essential switchgear feed from Unit 2 utilizing spare transformers SATA and SATB on May 7, 2026

Complete Walkdown (IP Section 03.02) (1 Sample)

(1) The inspectors evaluated system configurations during a complete walkdown of the Unit 2 turbine driven auxiliary feedwater pump discharge header and distribution alignment on June 23 through June 29, 2026.

71111.05 - Fire Protection

Fire Area Walkdown and Inspection (IP Section 03.01) (4 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

(1) Fire Area RB1, reactor building elevation 552, lower containment on May 10, 2026

(2) Fire Area SSF, standby shutdown facility building 7748, on May 26, 2026
(3) Fire Areas 6 and 13, Unit 1 electrical penetration rooms on elevations 560 and 577, respectively, on May 29, 2026
(4) Select sections of Fire Area SRV, service building, elevation 568, Room S203, dead-battery room, on June 22, 2026

Fire Brigade Drill Performance (IP Section 03.02) (1 Sample)

(1) The inspectors evaluated the on-site fire brigade training and performance during announced fire drill Scenario No. 2026-2-C for a Unit 2 B main step-up transformer fire on May 29, 2026.

71111.08P - Inservice Inspection Activities (PWR)

The inspectors verified that the reactor coolant system boundary, reactor vessel internals, risk-significant piping system boundaries, and containment boundary were appropriately monitored for degradation and that repairs and replacements were appropriately fabricated, examined and accepted by reviewing the following Unit 1 activities conducted during refueling outage C1R29, from April 20 to April 27, 2026.

PWR Inservice Inspection Activities - Nondestructive Examination and Welding Activities (IP Section 03.01) (1 Sample)

The inspectors verified that the following nondestructive examination and welding activities were performed appropriately:

(1) Ultrasonic Examination
- 1NC40-3, 3-inch reactor coolant pipe-to-elbow, ASME Class 1
- 1NV201-1, 3-inch chemical and volume control (CVCS) valve-to-pipe, ASME Class 1
- 1NC82-1, 1.5-inch safety injection pipe-to-nozzle, ASME Class 1

Radiographic Examination
- 1-NV15-1, CVCS pipe-to-pipe weld, ASME Class 2 (includes review of associated welding activity)
- 1-NV17-1A, CVCS pipe-to-pipe weld, ASME Class 2 (includes review of associated welding activity)

Dye Penetrant Examination
- MI1523A-11, miscellaneous instrument pipe-to-valve weld, ASME Class 2 (includes review of associated welding activity)

Visual Examination
- Bare metal visual examination, 1RPV-BMI-NOZZLES, reactor vessel bottom-mounted instrumentation nozzles, ASME Class 1

Welding Activities
- Work Order (WO) 20618468, Weld Doc. No. 241086, GTAW (gas tungsten arc welding), welds 1NV15-1 and 1NV17-1A, valve-to-pipe replacement, ASME Class 2

- WO 20650025, Weld Doc. No. 241287, GTAW, weld MI1523A-11, pipe-to-valve, ASME Class 2

PWR Inservice Inspection Activities - Vessel Upper Head Penetration Inspection Activities (IP Section 03.02) (1 Sample)

The inspectors verified that the licensee conducted the following vessel upper head penetration inspections and addressed any identified defects appropriately:

(1) Bare metal visual examination, 1RPV-HEAD-SURFACE, reactor pressure vessel upper head, ASME Class 1

PWR Inservice Inspection Activities - Boric Acid Corrosion Control Inspection Activities (IP Section 03.03) (1 Sample)

The inspectors verified the licensee is managing the boric acid corrosion control program through a review of the following evaluations:

(1) Boric acid walkdown on April 21, 2026
Review of the following boric acid evaluation (by action request (AR)):
- AR 02562718
- AR 02583108
- AR 02586114

Review of the following corrective actions performed for evidence of boric acid leaks that were identified (by work request (WR) number):
- WR 20306791
- WR 20306795
- WR 20306864

71111.11Q - Licensed Operator Requalification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (2 Samples)

(1) The inspectors observed and evaluated licensed operator performance in the control room during the Unit 2 shutdown and cooldown associated with the B steam generator tube leak inspection and repair, from March 30 to April 3, 2026.
(2) The inspectors observed and evaluated licensed operator performance in the control room during shutdown of Unit 1 at the commencement of refueling outage C1R29 on April 18, 2026.

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management (IP Section 03.01) (2 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

(1) Unit 2 elevated risk plan for reduced inventory with high decay heat during steam generator nozzle dam installation, on April 3, 2026
(2) Unit 1 elevated risk plan for lowered inventory with high decay heat during reactor vessel head removal, on April 22, 2026

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (5 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

(1) Nuclear Condition Report (NCR) 02594746, decay heat removal isolation valve 1ND-32A motor-operator thrust exceeded structural limits, From April 29, 2026, through May 8, 2026.
(2) NCR 02590988, indications of tube leakage from Unit 2 steam generator B, from March 28, 2026, through April 21, 2026
(3) NCR 02595615, cold-leg safety injection nozzle overstress due to snubber lockup, from May 6, 2026, to May 9, 2026
(4) NCR 02595274, Unit 1 charging pump A casing drain isolation valve leakage and emergency core cooling systems total leakage, on May 4, 2026, and May 5, 2026
(5) NCR 02600269, WR 20309939/20309940 for inservice testing (IST) of the starting air dryer discharge check valves 1VG-5, -6, -7, and -8 for the Unit 1 emergency diesel generator A, on June 16, 2026

71111.20 - Refueling and Other Outage Activities

Refueling/Other Outage Sample (IP Section 03.01) (2 Samples)

(1) The inspectors evaluated forced outage C2F26A that began on March 30, 2026, to inspect for tube leakage on Unit 2 steam generator B, as documented in NCR 02590988.
(2) The inspectors evaluated Unit 1 refueling outage C1R29 activities from April 18, 2026, to May 13, 2026.

71111.24 - Testing and Maintenance of Equipment Important to Risk

The inspectors evaluated the following testing and maintenance activities to verify system operability and/or functionality:

Post-Maintenance Testing (PMT) (IP Section 03.01) (2 Samples)

(1) PT/1/A/4200/ 007 A, "Centrifugal Charging Pump 1A Test," for Unit 1 charging pump A rotating assembly replacement, under Engineering Change 425674 and WO 20650619, on May 1, 2026.
(2) WO 20740468, for Unit 1 replacement of recirculated cooling to auxiliary feedwater pump suction header vent (1RC-116), on May 6, 2026

Surveillance Testing (IP Section 03.01) (2 Samples)

(1) PT/1/A/4200/009, "Engineered Safety Features Actuation Periodic Test," performed

for Unit 1, train B, from April 18 to 20, 2026
(2) PT/1/A/4200/013 H, "NI/NV (safety injection and coolant charging) Check Valve Test," following repairs to Unit 1 charging pump A discharge check valve 1NC-270, under WO 20789558, on May 5, 2026

Containment Isolation Valve Testing (IP Section 03.01) (1 Sample)

(1) PT/2/A/4200/041 D, "Unit 1 Containment Hydrogen Sample and Purge Isolation Valve Leak Rate Test," on April 24, 2026

Ice Condenser Testing (IP Section 03.01) (1 Sample)

(1) PT/0/A/4200/086, "Ice Bed Analysis Periodic Test," conducted during Unit 1 refueling outage C1R29 and completed on May 4, 2026

Reactor Coolant System Leakage Detection Testing (IP Section 03.01) (1 Sample)

(1) NCR 02597001, Unit 1 unidentified leakage exceeded Action Level 3 limits, on May 19, 2026

RADIATION SAFETY

71124.01 - Radiological Hazard Assessment and Exposure Controls

Radiological Hazard Assessment (IP Section 03.01) (1 Sample)

(1) The inspectors evaluated how the licensee identifies the magnitude and extent of radiation levels, identifies the concentrations and quantities of radioactive materials, and assesses radiological hazards.

Instructions to Workers (IP Section 03.02) (1 Sample)

(1) The inspectors evaluated how the licensee instructs workers on plant-related radiological hazards and the radiation protection requirements intended to protect workers from those hazards.

Contamination and Radioactive Material Control (IP Section 03.03) (2 Samples)

The inspectors observed/evaluated the following licensee processes for monitoring and controlling contamination and radioactive material:

(1) Workers exiting the radiologically controlled area (RCA) during the Unit 1 refueling outage C1R29.
(2) Material being removed from the Unit 1 equipment hatch RCA.

Radiological Hazards Control and Work Coverage (IP Section 03.04) (3 Samples)

The inspectors evaluated the licensee's control of radiological hazards for the following radiological work:

(1) Unit 1 reactor head disassembly preparation work in refueling cavity.

(2) Unit 1 reactor head lift.
(3) Unit 1 thermal sleeve replacement reactor vessel head.

High Radiation Area and Very High Radiation Area Controls (IP Section 03.05) (2 Samples)

The inspectors evaluated licensee controls of the following high radiation areas (HRAs) and very high radiation areas (VHRAs):

(1) Unit 1 letdown heat exchanger room.
(2) Unit 2 valve gallery.

Radiation Worker Performance and Radiation Protection Technician Proficiency (IP Section 03.06) (1 Sample)

(1) The inspectors evaluated radiation worker and radiation protection technician performance as it pertains to radiation protection requirements.

71124.03 - In-Plant Airborne Radioactivity Control and Mitigation

Permanent Ventilation Systems (IP Section 03.01) (1 Sample)

The inspectors evaluated the configuration of the following permanently installed ventilation systems:

(1) Units 1 and 2 main control room ventilation system

Temporary Ventilation Systems (IP Section 03.02) (1 Sample)

The inspectors evaluated the configuration of the following temporary ventilation systems:

(1) High efficiency particulate air portable ventilation unit, ID 100, test 05-08-2025, for work in Unit 1 upper containment

Use of Respiratory Protection Devices (IP Section 03.03) (1 Sample)

(1) The inspectors evaluated the licensee’s use of respiratory protection devices.

Self-Contained Breathing Apparatus for Emergency Use (IP Section 03.04) (1 Sample)

(1) The inspectors evaluated the licensee’s use and maintenance of self-contained breathing apparatuses.

71124.04 - Occupational Dose Assessment

Source-Term Characterization (IP Section 03.01) (1 Sample)

(1) The inspectors evaluated licensee performance as it pertains to radioactive source-term characterization.

External Dosimetry (IP Section 03.02) (1 Sample)

(1) The inspectors evaluated how the licensee processes, stores, and uses external dosimetry.

Internal Dosimetry (IP Section 03.03) (1 Sample)

The inspectors evaluated the following internal dose assessments:

(1) For-cause internal dose investigation, September 15, 2025.

Special Dosimetric Situations (IP Section 03.04) (1 Sample)

The inspectors evaluated the following special dosimetric situations:

(1) Declared pregnant worker, February 2025.

71124.05 - Radiation Monitoring Instrumentation

Walkdowns and Observations (IP Section 03.01) (5 Samples)

The inspectors evaluated the following radiation detection instrumentation during plant walkdowns:

(1) Area radiation monitors in the auxiliary building.
(2) Continuous air monitors in the auxiliary building.
(3) Portable friskers used at the exit of the RCA.
(4) Personnel contamination monitors at the RCA exit.
(5) Portal monitors at the RCA exit.

Calibration and Testing Program (IP Section 03.02) (10 Samples)

The inspectors evaluated the calibration and testing of the following radiation detection instruments:

(1) 1EMF-12, control room radiation monitor, June 7, 2021.
(2) 1EMF-53A, containment high-range area radiation monitor, A-Train, October 15, 2024.
(3) 1EMF-53B, containment high-range area radiation monitor, B-Train, October 15, 2024.
(4) 2EMF-72, steam generator B Nitrogen-16 leakage monitor, January 9, 2025.
(5) Canberra iCAM, continuous air monitor, EnRad ID 13258, November 12, 2025.
(6) Canberra GEM-5, portal monitor, EnRad ID 13560, November 12, 2025.
(7) Mirion FASTSCAN, whole body counter, August 3, 2025.
(8) Ludlum Model 177, count-rate meter, EnRad ID 07187, May 27, 2025.
(9) Ludlum Model 3030P, alpha beta sample counter, EnRad ID 03584, January 20, 2026.
(10) Mirion HandFoot-Fibre XL (HFF XL), radiation contamination monitor, EnRad 12569, February 5, 2026.

Effluent Monitoring Calibration and Testing Program Sample (IP Section 03.03) (2 Samples)

The inspectors evaluated the calibration and maintenance of the following radioactive effluent monitoring and measurement instrumentation:

(1) Unit 1 plant vent stack radiation monitor, all channels (1EMF-35, -36, and -54)

(2) Liquid radwaste discharge monitor (0EMF-49)

OTHER ACTIVITIES – BASELINE

71152A - Annual Follow-up Problem Identification and Resolution

Annual Follow-up of Selected Issues (IP Section 03.03) (2 Samples)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issues:

(1) The inspectors reviewed the licensee's corrective action program to identify potential trends in steam generator main feedwater check valves' performance and maintenance issues that might be indicative of a more significant safety issue.
(2) The inspectors reviewed the licensee's corrective action program to identify potential trends in auxiliary feedwater check valve back-leakage issues that might be indicative or a more significant safety issue. The inspectors documented an Observation in this report under Inspection Results Section 71152A.

INSPECTION RESULTS

Observation: Check valve 2CA-53 Back-Leakage	71152A
The inspectors reviewed the licensee's corrective action program to identify potential trends in auxiliary feedwater (AFW) check valve back-leakage that might be indicative of a more significant safety issue. The AFW system is a risk-significant, safety-related system that provides a source of emergency feedwater to the steam generators to maintain level if the normal feedwater source is not available. This review was performed with consideration of the guidance in Generic Letter 88-03, "Steam Binding of Auxiliary Feedwater Pumps," dated February 17, 1988, which emphasizes monitoring AFW system conditions that may contribute to steam binding or degraded pump performance, including temperature indications that could reflect back-leakage through AFW check valves. Since 2022, Unit 2 check valve 2CA-53 has been an outlier with respect to AFW back-leakage. Standby temperatures upstream of comparable turbine-driven AFW pump discharge check valves have remained near 100 degrees Fahrenheit (°F), whereas standby temperatures upstream of 2CA-53 have increased from approximately 165°F in 2022 to consistently above 200°F since the most recent Unit 2 startup in April 2026. Furthermore, since 2024, standby temperatures have repeatedly reached or exceeded the 220°F high-temperature alarm setpoint. While the current back-leakage and resulting temperature conditions have not affected the operability of the system, as determined by the licensee, the inspectors noted the following regarding the licensee's tracking and resolution of the degraded condition. The licensee identified the condition and generated Work Request 20267633 on April 23, 2024. However, no corrective actions have been initiated (completed nor planned) to specifically address this back-leakage. A routine six-year valve disassembly, testing, and inspection was performed on September 16, 2025, (WO 20705034) as part of the inservice test program. Although no inspection deficiencies were identified, the back-leakage work request from 2024 was not included in the inspection scope to evaluate or resolve the degraded condition as part of the planned activity. As a result, despite the inspection,	

operational data continue to indicate elevated upstream standby temperatures consistent with back-leakage.

The site has a history of AFW check valve back-leakage and has addressed similar issues on other AFW trains through valve rebuilds and piping modifications that reoriented valves from horizontal to vertical piping to improve seating. Check valve 2CA-53 remains located on a horizontally oriented piping section. With the completion of corrective actions on the other AFW trains, 2CA-53 is currently the only train component exhibiting elevated upstream standby temperatures attributable to back-leakage.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified that no proprietary information was retained or documented in this report.

- On April 23, 2026, the inspectors presented the radiation protection (occupational safety) inspection results to Joe Smith, Nuclear Station Sciences Manager, and other members of the licensee staff.
- On April 23, 2026, the inspectors presented the inservice inspection (ISI) results to Rickey Koster, ISI Coordinator, and other members of the licensee staff.
- On June 11, 2026, the inspectors presented the radiation protection (radiation monitoring instrumentation) baseline inspection results to Joe Smith, Nuclear Station Sciences Manager, and other members of the licensee staff.
- On July 7, 2026, the inspectors presented the integrated inspection results to Jonathan Huecker, Site Vice President, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.04	Drawings	CN-2592-01.00	Flow diagram of Auxiliary feedwater system	
		CN-2592-01.01	Flow diagram of Auxiliary feedwater system	
71111.05	Fire Plans	CSD-CNS-PFP-AB-0560-001	Auxiliary Building Elevation 554 & 560 Pre-Fire Plan	
		CSD-CNS-PFP-AB-0577-001	Auxiliary Building Elevation 574 & 560 Pre-Fire Plan	
		CSD-CNS-PFP-PA-003	Protected Area South Pre-Fire Plan	
		CSD-CNS-PFP-RB1-0000-001	Unit 1 Reactor Building Elevation: ALL Pre-Fire Plan	
		CSD-CNS-PFP-SRV-0568-001	Service Building Elevation 568 & 574 Pre-Fire Plan	
71111.08P	Corrective Action Documents	NCR 02531825	Radial Support Key Indications (INR-C1R28-01A)	
	NDE Reports	UT-26-003	Ultrasonic Examination of 1NC40-3	4/25/2026
		UT-26-005	Ultrasonic Examination of 1NV201-1	4/24/2026
		UT-26-009	Ultrasonic Examination of 1NC82-1	4/26/2026
		VT-26-001	Visual Examination of Reactor Vessel Bottom-Mounted Instruments	4/20/2026
		VT-26-003	Visual Examination of Reactor Pressure Vessel Upper Head	4/26/2026
	Work Orders	Work Orders (by number)	20618468, 20650025	
71111.13	Drawings	DWG CN-2680-122-01	Reactor Coolant System Elevation View vs Instrument Percent Scale for NC Level Instrumentation	1
	Procedures	AD-WC-ALL-0420	Shutdown Risk Management	10
		AD-WC-CNS-0420	Catawba Nuclear Station Shutdown Risk Management	12
		OP/2/A/6150/006	Draining the Reactor Coolant System	102
71111.15	Drawings	CN-1609-01.00	Flow Diagram of Diesel Gen. Engine Starting Air System (VG)	
	Miscellaneous	CNC -1227.00-00-0106	Radiological Consequences of a DB LOCA at Catawba Nuclear Station	20

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		CNC-1223.21-00-0022	Post-Accident FWST In Leakage Sources	0
	Operability Evaluations	EC 426580	Evaluation of Negative Conductor Connection to Breaker, 1 EDE-F01 C, Loss of 1ETA Control Power	
	Procedures	PT/1/A/4200/007	VG/RN Valve Inservice Test	
71111.20	Corrective Action Documents	A/R 02593042	Notes, 50.59 Screen for C1 C30	
	Miscellaneous	AD-NF-ALL-0807	Reload Design Process	11
		CNEI-0400-416	Catawba 1 Cycle 29 Core Operating Limits Report	
		PT/0/A/4150/022	Total Core Reloading	69
71124.01	Procedures	AD-RP-ALL-2017	Access Controls to Very High Radiation Areas and Supplemental Access Controls for HRA and LHRA	10
	Radiation Surveys	CNS-M-20260105-5	Unit 1 Letdown Hx	1/5/2026
71124.03	Miscellaneous	Report # 260128043-00	Compressed Air/Gas Quality Testing	1/28/2026
71124.04	Calculations	AD-RP-ALL-2015 Alpha Radiation Characterization	2025 C2R27 Alpha Characterization	11/24/2025
	Miscellaneous	NVLAP LAB CODE 100505-0	Dosimetry Processor Accreditation March 2026 - March 2027	
71124.05	Corrective Action Documents Resulting from Inspection	Action Request (ARs)	02600252, 02599809, 02599420	
	Procedures	CSD-CP-CNS-0020	Catawba Nuclear Station Primary to Secondary Leak Rate Monitoring Program	4
		IP/2/B/3314/016 R	2EMF71, 2EMF72, 2EMF73 and 2EMF74 N-16 S/G Leakage Monitor Channel Calibration	35
	Work Orders	20640091 01	1EMF-54: Perform Channel Cal	6/12/2025
		20657215 01	2EMF-72: PERFORM CHANNEL CAL	2/6/2025
71152A	Corrective Action Documents	Nuclear Condition Report (NCR)	20292330, 2568687, 2590988, 2592578, 2198650, 2198666	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		PIP C-05-2303, C-96-120		
		WR 20267633		
	Drawings	CNM 1205.00-0160.001	Valve Items 2H-235, 2H-236 O/L	
	Engineering Changes	Engineering Change (EC)	426000	
	Miscellaneous	AD-EG-ALL-1452	Check Valve Component Program	
		AD-EG-ALL1103	Procurement Engineering Products	
		Bulletin 85-01	Steam Binding of Auxiliary Feedwater Pumps	
		Generic Letter 88-03	Issuance of Resolution of Generic Safety Issue 93, Steam Binding of Auxiliary Feedwater Pumps	
	Procedures	MP/0/A/7600/024	Pacific Pressure Seal Tilting Disc Check Valves Corrective Maintenance	22
	Work Orders	Work Order (WO)	20034175, 20604298, 20604303, 20604308, 20503700, 20408515, 20272229, 20705034	

Jonathan Huecker
SUBJECT: CATAWBA NUCLEAR STATION – INTEGRATED INSPECTION REPORT 05000413/2026002 AND 05000414/2026002 DATED: July 15, 2026
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