UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

August 18, 2026

Mr. Charles V. McFeaters
President and Chief Nuclear Officer
PSEG Nuclear LLC - N09
P.O. Box 236
Hancocks Bridge, NJ 08038

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENT NOS. 356 AND 338 REGARDING TECHNICAL SPECIFICATIONS FOR ALTERNATING CURRENT SOURCES (EPID L-2025-LLA-0169)

Dear Mr. McFeaters:

The U.S. Nuclear Regulatory Commission (the Commission) has issued the enclosed Amendment Nos. 356 and 338 to Renewed Facility Operating License Nos. DPR-70 and DPR-75 for the Salem Nuclear Generating Station, Unit Nos. 1 and 2, respectively. The amendments consist of changes to the operating licenses and technical specifications in response to PSEG Nuclear LLC's application dated October 16, 2025, as supplemented by letter dated May 14, 2026.

The amendments revise emergency diesel generator (EDG) starting and load-run test surveillance requirements and required actions when certain equipment is inoperable. Specifically, the amendments (1) revise the required actions for testing operable EDGs when offsite circuits are inoperable, (2) split the current surveillance requirement combining the EDG starting test and load-run test into two separate surveillance requirements, and (3) revise required actions for consistency with a previously approved extended allowed outage time for the EDGs.

A copy of the related safety evaluation is also enclosed. A notice of issuance will be included in the Commission's biweekly *Federal Register* notice.

Sincerely,

/RA/

Audrey Klett, Senior Project Manager
Operating Reactor Licensing and Data Branch 1
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Docket Nos. 50-272 and 50-311

Enclosures:
1. Amendment No. 356 to
   License No. DPR-70
2. Amendment No. 338 to
   License No. DPR-75
3. Safety Evaluation

cc: Listserv

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

CONSTELLATION ENERGY GENERATION, LLC

DOCKET NO. 50-272

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 356
Renewed License No. DPR-70

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Constellation Energy Generation, LLC (the licensees) October 16, 2025, as supplemented by letter dated May 14, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance that (i) the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to license conditions and technical specifications, as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-70 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 356, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

DAVID WRONA

Digitally signed by DAVID WRONA
Date: 2026.08.18 07:56:56 -04'00'

David Wrona, Chief
Operating Reactor Licensing and Data Branch 1
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed
Facility Operating License
and Technical Specifications

Date of Issuance: August 18, 2026

ATTACHMENT TO LICENSE AMENDMENT NO. 356

SALEM NUCLEAR GENERATING STATION, UNIT NO. 1

RENEWED FACILITY OPERATING LICENSE NO. DPR-70

DOCKET NO. 50-272

Replace the following page of Renewed Facility Operating License No. DPR-70 with the attached revised page as indicated. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove	Insert
Page 3	Page 3

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
3/4 8-2a	3/4 8-2a
3/4 8-2b	3/4 8-2b
3/4 8-3	3/4 8-3

(4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70 to receive, possess and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Sections 50.54 and 50.59 of Part 50, and Section 70.32 of Part 70; and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at a steady state reactor core power level not in excess of 3459 megawatts (one hundred percent of rated core power).

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 356, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications, and the Environmental Protection Plan.

(3) Deleted Per Amendment 22, 11-20-79

(4) Less than Four Loop Operation

PSEG Nuclear LLC shall not operate the reactor at power levels above P-7 (as defined in Table 3.3-1 of Specification 3.3.1.1 of Appendix A to this renewed license) with less than four (4) reactor coolant loops in operation until safety analyses for less than four loop operation have been submitted by the licensees and approval for less than four loop operation at power levels above P-7 has been granted by the Commission by Amendment of this renewed license.

(5) PSEG Nuclear LLC shall implement and maintain in effect all provisions of the approved fire protection program as described in the Updated Final Safety

ACTION (Continued)

c. With one independent A.C. circuit and one diesel generator of the above required A.C. electrical power sources inoperable:

1. Demonstrate the OPERABILITY of the remaining independent A.C. circuit by performing Surveillance Requirement 4.8.1.1.1.a within one hour and at least once per 8 hours thereafter; and

2. Within 4 hours, declare required systems or components supported by the inoperable diesel generator inoperable when a required redundant system or component is inoperable; and

3. Restore at least one of the inoperable sources to OPERABLE status within 12 hours; and

4. Restore the remaining inoperable A.C. power source (independent A.C. circuit or diesel generator) to OPERABLE status in accordance with the provisions of Section 3.8.1.1 ACTION Statement a or b, as appropriate, with the completion time of that ACTION Statement based on the time of initial loss of that remaining inoperable A.C. power source.

Otherwise,

5. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

d. With two of the above required independent A.C. circuits inoperable:

1. Deleted.

2. Within 12 hours, declare required systems or components supported by the inoperable offsite circuits inoperable when a required redundant system or component is inoperable, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours; and

3. Restore at least one of the inoperable independent A.C. circuits to OPERABLE status within 24 hours or in accordance with the Risk Informed Completion Time Program or be in at least HOT STANDBY within the next 6 hours; and

4. With only one of the independent A.C. circuits OPERABLE, restore the other independent A.C. circuit to OPERABLE status within 72 hours from time of initial loss or in accordance with the Risk Informed Completion Time Program or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

ACTION (Continued)

e. With two or more of the above required diesel generators inoperable:

1. Demonstrate the OPERABILITY of two independent A.C. circuits by performing Surveillance Requirement 4.8.1.1.1.a within one hour and at least once per 8 hours thereafter; and

2. Restore at least two of three diesel generators to OPERABLE status within 2 hours; and

3. Restore the remaining inoperable diesel generator to OPERABLE status in accordance with the provisions of Section 3.8.1.1 ACTION Statement b, with the completion time of that ACTION Statement based on the time of initial loss of the remaining inoperable diesel generator.

Otherwise,

4. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

f. With one of the above required fuel transfer pumps inoperable, either restore it to OPERABLE status within 72 hours or in accordance with the Risk Informed Completion Time Program or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

g. With one of the above required fuel storage tanks inoperable, either restore it to OPERABLE status within 72 hours or in accordance with the Risk Informed Completion Time Program or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

h. LCO 3.0.4.b is not applicable to DGs.

i. With both of the above required fuel transfer pumps inoperable, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

j. With both the above required fuel storage tanks inoperable, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.1.1.1 Two physically independent A.C. circuits between the offsite transmission network and the onsite Class 1E distribution system (vital bus system) shall be:

a. Determined OPERABLE in accordance with the Surveillance Frequency Control Program by verifying correct breaker alignments, power availability, and

b. Demonstrated OPERABLE in accordance with the Surveillance Frequency Control Program during shutdown by transferring (manually and automatically) vital bus supply from one 13/4 kv transformer to the other 13/4 kv transformer.

4.8.1.1.2 Each diesel generator shall be demonstrated OPERABLE:

a. In accordance with the Surveillance Frequency Control Program by:

1. Verifying the fuel level in its day tank.

2. Verifying the diesel generator starts from standby conditions* and achieves ≥ 3910 volts and ≥ 58.8 Hz in ≤ 13 seconds, and subsequently achieves steady state voltage of ≥ 3910 and ≤ 4400 volts and frequency of 60 ± 1.2 Hz.

3. Verifying the diesel generator is aligned to provide standby power to the associated vital bus.

4. Verifying the generator is synchronized with voltage maintained ≥ 3910 and ≤ 4580 volts, gradually loaded to 2340-2600 kw**, and operates at a load of 2340-2600 kw for greater than or equal to 60 minutes.

b. In accordance with the Surveillance Frequency Control Program and after each operation of the diesel where the period of operation was greater than or equal to one hour by checking for and removing accumulated water from the day tanks.

c. In accordance with the Surveillance Frequency Control Program by verifying the diesel generator starts from standby conditions* and achieves ≥ 3910 volts and ≥ 58.8 Hz in ≤ 13 seconds, and subsequently achieves steady state voltage of ≥ 3910 and ≤ 4400 volts and frequency of 60 ± 1.2 Hz.

The generator shall be synchronized to its emergency bus with voltage maintained ≥ 3910 and ≤ 4580 volts, loaded to 2340-2600** kw in less than or equal to 60 seconds, and operate at a load of 2340-2600 kw for at least 60 minutes.

This test, if it is performed so it coincides with the testing required by Surveillance Requirements 4.8.1.1.2.a.2 and 4.8.1.1.2.a.4 may also serve to concurrently meet those requirements.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

PSEG NUCLEAR LLC

CONSTELLATION ENERGY GENERATION, LLC

DOCKET NO. 50-311

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 338
Renewed License No. DPR-75

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment filed by PSEG Nuclear LLC, acting on behalf of itself and Constellation Energy Generation, LLC (the licensees) October 16, 2025, as supplemented by letter dated May 14, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in Title 10 of the *Code of Federal Regulations* (10 CFR), Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance that (i) the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to license conditions and technical specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-75 is hereby amended to read as follows:

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 338, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 90 days.

FOR THE NUCLEAR REGULATORY COMMISSION

DAVID WRONA

Digitally signed by DAVID WRONA
Date: 2026.08.18 07:57:34 -04'00'

David Wrona, Chief
Operating Reactor Licensing and Data Branch 1
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed
Facility Operating License
and Technical Specifications

Date of Issuance: August 18, 2026

ATTACHMENT TO LICENSE AMENDMENT NO. 338

SALEM NUCLEAR GENERATING STATION, UNIT NO. 2

RENEWED FACILITY OPERATING LICENSE NO. DPR-75

DOCKET NO. 50-311

Replace the following page of Renewed Facility Operating License No. DPR-75 with the attached revised page as indicated. The revised page is identified by amendment number and contains marginal lines indicating the areas of change.

Remove	Insert
Page 3	Page 3

Replace the following pages of the Appendix A, Technical Specifications, with the attached revised pages as indicated. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove	Insert
3/4 8-2a	3/4 8-2a
3/4 8-2b	3/4 8-2b
3/4 8-3	3/4 8-3

(3) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

(4) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use at any time any byproduct, source or special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration and as fission detectors in amounts as required;

(5) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(6) PSEG Nuclear LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

PSEG Nuclear LLC is authorized to operate the facility at steady state reactor core power levels not in excess of 3459 megawatts (thermal).

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 338, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the renewed license. PSEG Nuclear LLC shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

ACTION (Continued)

c. With one independent A.C. circuit and one diesel generator of the above required A.C. electrical power sources inoperable:

1. Demonstrate the OPERABILITY of the remaining independent A.C. circuit by performing Surveillance Requirement 4.8.1.1.1.a within one hour and at least once per 8 hours thereafter; and

2. Within 4 hours, declare required systems or components supported by the inoperable diesel generator inoperable when a required redundant system or component is inoperable; and

3. Restore at least one of the inoperable sources to OPERABLE status within 12 hours; and

4. Restore the remaining inoperable A.C. power source (independent A.C. circuit or diesel generator) to OPERABLE status in accordance with the provisions of Section 3.8.1.1 ACTION Statement a or b, as appropriate, with the completion time of that ACTION Statement based on the time of initial loss of that remaining inoperable A.C. power source.

Otherwise,

5. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

d. With two of the above required independent A.C. circuits inoperable:

1. Deleted.

2. Within 12 hours, declare required systems or components supported by the inoperable offsite circuits inoperable when a required redundant system or component is inoperable, or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours; and

3. Restore at least one of the inoperable independent A.C. circuits to OPERABLE status within 24 hours or in accordance with the Risk Informed Completion Time Program or be in at least HOT STANDBY within the next 6 hours; and

4. With only one of the independent A.C. circuits OPERABLE, restore the other independent A.C. circuit to OPERABLE status within 72 hours from time of initial loss or in accordance with the Risk Informed Completion Time Program or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

ACTION (Continued)

e. With two or more of the above required diesel generators inoperable:

1. Demonstrate the OPERABILITY of two independent A.C. circuits by performing Surveillance Requirement 4.8.1.1.1.a within one hour and at least once per 8 hours thereafter; and

2. Restore at least two of three diesel generators to OPERABLE status within 2 hours; and

3. Restore the remaining inoperable diesel generator to OPERABLE status in accordance with the provisions of Section 3.8.1.1 ACTION Statement b, with the completion time of that ACTION Statement based on the time of initial loss of the remaining inoperable diesel generator.

Otherwise,

4. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

f. With one of the above required fuel transfer pumps inoperable, either restore it to OPERABLE status within 72 hours or in accordance with the Risk Informed Completion Time Program or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

g. With one of the above required fuel storage tanks inoperable, either restore it to OPERABLE status within 72 hours or in accordance with the Risk Informed Completion Time Program or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

h. LCO 3.0.4.b is not applicable to DGs.

i. With both of the above required fuel transfer pumps inoperable, be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

j. With both of the above required fuel storage tanks inoperable be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN with the following 30 hours.

SURVEILLANCE REQUIREMENTS

4.8.1.1.1 Two physically independent A.C. circuits between the offsite transmission network and the onsite Class 1E distribution system (vital bus system) shall be:

a. Determined OPERABLE in accordance with the Surveillance Frequency Control Program by verifying correct breaker alignments, power availability, and

b. Demonstrated OPERABLE in accordance with the Surveillance Frequency Control Program during shutdown by transferring (manually and automatically) vital bus supply from one 13/4 kv transformer to the other 13/4 kv transformer.

4.8.1.1.2 Each diesel generator shall be demonstrated OPERABLE:

a. In accordance with the Surveillance Frequency Control Program by:

1. Verifying the fuel level in its day tank.

2. Verifying the diesel generator starts from standby conditions* and achieves ≥ 3910 volts and ≥ 58.8 Hz in ≤ 13 seconds, and subsequently achieves steady state voltage of ≥ 3910 and ≤ 4400 volts and frequency of 60 ± 1.2 Hz.

3. Verifying the diesel generator is aligned to provide standby power to the associated vital bus.

4. Verifying the generator is synchronized with voltage maintained ≥ 3910 and ≤ 4580 volts, gradually loaded to 2340-2600 kw**, and operates at a load of 2340-2600 kw for greater than or equal to 60 minutes.

b. In accordance with the Surveillance Frequency Control Program and after each operation of the diesel where the period of operation was greater than or equal to 1 hour by checking for and removing accumulated water from the day tanks.

c. In accordance with the Surveillance Frequency Control Program by verifying the diesel generator starts from standby conditions* and achieves ≥ 3910 volts and ≥ 58.8 Hz in ≤ 13 seconds, and subsequently achieves steady state voltage of ≥ 3910 and ≤ 4400 volts and frequency of 60 ± 1.2 Hz.

The generator shall be synchronized to its emergency bus with voltage maintained ≥ 3910 and ≤ 4580 volts, loaded to 2340-2600** kw in less than or equal to 60 seconds, and operate at a load of 2340-2600 kw for at least 60 minutes.

This test, if it is performed so it coincides with the testing required by Surveillance Requirements 4.8.1.1.2.a.2 and 4.8.1.1.2.a.4 may also serve to concurrently meet those requirements.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION FOR

AMENDMENT NO. 356 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-70

AMENDMENT NO. 338 TO RENEWED FACILITY OPERATING LICENSE NO. DPR-75

PSEG NUCLEAR LLC

CONSTELLATION ENERGY GENERATION, LLC

SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2

DOCKET NOS. 50-272 AND 50-311

1.0 INTRODUCTION

1.1 Application and Noticing Information

By letter LR-N25-0053 dated October 16, 2025 (Reference 1), as supplemented by letter LR-N26-0033 dated May 14, 2026 (Reference 2), PSEG Nuclear LLC (the licensee) submitted license amendment request (LAR) S22-06 to request changes to the renewed facility operating licenses and technical specifications (TSs) for the Salem Nuclear Generating Station (Salem), Unit Nos. 1 and 2.

A notice of consideration of proposed issuance of amendments with a proposed no significant hazards consideration (NSHC) determination was published in the *Federal Register* on February 17, 2026 (Reference 3), and there has been no public comment on the proposed NSHC determination. The supplement dated May 14, 2026, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the U.S. Nuclear Regulatory Commission (NRC or the Commission) staff's original proposed no significant hazards consideration determination.

1.2 Proposed Changes

The licensee proposed to (1) revise the required actions for testing operable emergency diesel generators (EDGs) when offsite circuits are inoperable, (2) split the current surveillance requirement (SR) combining the EDG starting test and load-run test into two separate SRs, and (3) revise required actions for consistency with a previously approved extended allowed outage time (AOT, also "completion time") for the EDGs.

The proposed revised marked-up TS pages provided in the LAR supplement superseded the proposed marked-up TS pages provided in the LAR. The licensee's proposed changes to TS 3/4.8.1.1, "A.C. [or AC, Alternating Current] Sources - Operating," are summarized as follows.

- The second paragraph of SR 4.8.1.1.2.a.2, which currently states, "Subsequently, verifying the generator is synchronized with voltage maintained ≥ 3910 and ≤ 4580 volts, gradually loaded to 2340-2600 kw [kilowatts]**, and operates at a load of 2340-2600 kw for greater than or equal to 60 minutes," would be deleted.

- A new SR 4.8.1.1.2.a.4 would be created and state, "Verifying the generator is synchronized with voltage maintained ≥ 3910 and ≤ 4580 volts, gradually loaded to 2340-2600 kw**, and operates at a load of 2340-2600 kw for greater than or equal to 60 minutes."

- The last sentence of SR 4.8.1.1.2.c would be revised to state, with additions shown in double underlined text, "This test, if it is performed so it coincides with the testing required by Surveillance Requirements 4.8.1.1.2.a.2 and 4.8.1.1.2.a.4, may also serve to concurrently meet those requirements."

- The intent of TS 3.8.1.1, Action b.3, would be modified. While the licensee did not propose changes to the text of TS 3.8.1.1, Action b.3, the effect of the proposed changes to current SR 4.8.1.1.2.a.2 (summarized above) would be to remove the requirement to perform the one-hour loaded-run test for the operable EDGs when one EDG is inoperable.

- The text in TS 3.8.1.1, Action c, would be replaced in its entirety to state:

  With one independent A.C. circuit and one diesel generator of the above required A.C. electrical power sources inoperable:

  1. Demonstrate the OPERABILITY of the remaining independent A.C. circuit by performing Surveillance Requirement 4.8.1.1.1.a within one hour and at least once per 8 hours thereafter; and

  2. Within 4 hours, declare required systems or components supported by the inoperable diesel generator inoperable when a required redundant system or component is inoperable; and

  3. Restore at least one of the inoperable sources to OPERABLE status within 12 hours; and

  4. Restore the remaining inoperable A.C. power source (independent A.C. circuit or diesel generator) to OPERABLE status in accordance with the provisions of Section 3.8.1.1 ACTION Statement a or b, as appropriate, with the completion time of that ACTION Statement based on the time of initial loss of that remaining inoperable A.C. power source.

Otherwise,

5. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

- The text in TS 3.8.1.1, Action d.1, would be deleted and replaced with “Deleted.”

- The text in TS 3.8.1.1, Action e, including footnote (2), would be deleted and replaced in its entirety to state:

With two or more of the above required diesel generators inoperable:

1. Demonstrate the OPERABILITY of two independent A.C. circuits by performing Surveillance Requirement 4.8.1.1.1.a within one hour and at least once per 8 hours thereafter; and

2. Restore at least two of three diesel generators to OPERABLE status within 2 hours; and

3. Restore the remaining inoperable diesel generator to OPERABLE status in accordance with the provisions of Section 3.8.1.1 ACTION Statement b, with the completion time of that ACTION Statement based on the time of initial loss of the remaining diesel generator.

Otherwise,

4. Be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

1.3 Background

In section 2.1 of the LAR enclosure, the licensee summarized its offsite power supply and onsite Class 1E distribution system. The licensee stated that offsite power is supplied from three physically independent, 500-kilovolt circuits designed and located to minimize the likelihood of simultaneous failure and that there are two physically independent circuits between the off-site transmission network and the on-site Class 1E distribution system. The licensee stated that any two of the EDGs and their associated vital buses can supply sufficient power for operation of the required safeguards equipment for a design basis loss-of-coolant accident coincident with a loss of offsite power. The licensee also stated that sufficient redundancy is provided in the safety features and their assignment to the vital buses so that failure to energize any one vital bus does not prevent operation of the required minimum safety equipment.

In section 2.3 of the LAR enclosure, the licensee indicated that the current TS requirements for AC sources: (1) place demands on plant operating personnel that can lessen the focus on the actions needed to restore the inoperable EDG or offsite circuit to operable status, (2) make the EDGs susceptible to perturbations in the offsite power system when the number of onsite power sources to the vital buses is already reduced, and (3) leave room for potential ambiguity with regard to the proper actions. The licensee indicated that its proposed changes would enhance clarity, align the intent of the TS with the standard TS in NUREG-1431, Revision 5, “Standard Technical Specifications, Westinghouse Plants” (Reference 4), reduce equipment degradation, and eliminate unnecessary burden on operating personnel.

2.0 REGULATORY EVALUATION

2.1 Requirements

Under Section 50.92(a) of Title 10 to the *Code of Federal Regulations* (10 CFR), determinations on whether to grant an applied-for license amendment are to be guided by the considerations that govern the issuance of initial licenses to the extent applicable and appropriate. Both the common standards for licenses in 10 CFR 50.40(a) (regarding, among other things, consideration of the operating procedures, the facility and equipment, the use of the facility, and other technical specifications, or the proposals) and those specifically for issuance of operating licenses in 10 CFR 50.57(a)(3), provide that there must be reasonable assurance that the activities at issue will not endanger the health and safety of the public, and that the applicant will comply with the Commission's regulations.

The regulation at 10 CFR 50.36, "Technical specifications," paragraph 50.36(b) requires, in part, that the operating license of a nuclear power facility include TS.

The regulation at 10 CFR 50.36(c)(2)(i) requires, in part, that TS include limiting conditions for operation (LCO), defined as the lowest functional capability or performance levels of equipment required for safe operation of the facility. When an LCO is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the TS until the condition can be met.

The remedial actions specified in the TS must provide the requisite reasonable assurance of public health and safety described in 10 CFR 50.40 and 50.57.

The regulation at 10 CFR 50.36(c)(3) requires that the TS include SRs, which are requirements relating to test, calibration, or inspection, to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the LCO will be met.

2.2 Mandated Licensing Basis Document Information

Salem was not licensed to the general design criteria (GDC) in 10 CFR Part 50, Appendix A, "General Design Criteria for Nuclear Power Plants" (Appendix A GDC). Salem was originally designed to comply with the Atomic Energy Commission (AEC) "Proposed General Design Criteria for Nuclear Power Plant Construction Permits" published in July 1967 (AEC proposed GDC). However, in section 4.1 of the LAR, the licensee assessed the applicable AEC proposed GDC relevant to this LAR, criteria 24, 39, and 48, as similar to the Appendix A GDC, criteria 17 and 18.

Salem's Updated Final Safety Analysis Report (UFSAR, Reference 5) documents Salem's conformance with the AEC proposed GDC, criteria 24, 39, and 48, in section 3.1.2, "Conformance with AEC Proposed General Design Criteria (July 1967)," as follows:

- Criterion 24, "Emergency Power for Protection Systems," states, "In the event of loss of all offsite power, sufficient alternate sources of power shall be provided to permit the required functioning of the protection systems."

- Criterion 39, "Emergency Power for Engineered safety Features," states:

> Alternate power systems shall be provided and designed with adequate independence, redundancy, capacity, and testability to permit the functioning

required of the engineered safety features. As a minimum, the onsite power system and the offsite power system shall each, independently, provide this capacity assuming a failure of a single active component in each power system.

- Criterion 48, "Testing of Operational Sequence of Emergency Core Cooling Systems," states, "A capability shall be provided to test under conditions as close to design as practical, the full operational sequence that would bring the Emergency Core Cooling Systems into action, including the transfer to alternate power source."

In comparison, the 10 CFR Part 50, Appendix A, GDC 17 and 18 are as follows:

- GDC 17, "Electric power systems," states, in part:

  An onsite electric power system and an offsite electric power system shall be provided to permit functioning of structures, systems, and components important to safety. The safety function for each system (assuming the other system is not functioning) shall be to provide sufficient capacity and capability to assure that (1) specified acceptable fuel design limits and design conditions of the reactor coolant pressure boundary are not exceeded as a result of anticipated operational occurrences and (2) the core is cooled and containment integrity and other vital functions are maintained in the event of postulated accidents.

  The onsite electric power supplies, including the batteries, and the onsite electric distribution system, shall have sufficient independence, redundancy, and testability to perform their safety functions assuming a single failure.

- GDC 18, "Inspection and testing of electric power systems," states:

  Electric power systems important to safety shall be designed to permit appropriate periodic inspection and testing of important areas and features, such as wiring, insulation, connections, and switchboards, to assess the continuity of the systems and the condition of their components. The systems shall be designed with a capability to test periodically (1) the operability and functional performance of the components of the systems, such as onsite power sources, relays, switches, and buses, and (2) the operability of the systems as a whole and, under conditions as close to design as practical, the full operation sequence that brings the systems into operation, including operation of applicable portions of the protection system, and the transfer of power among the nuclear power unit, the offsite power system, and the onsite power system.

2.3 NRC Guidance

- NUREG-1431, Revision 5, "Standard Technical Specifications, Westinghouse Plants," Volumes 1 and 2 (Reference 4)[1]

- Generic Letter (GL) 93-05, "Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing" (Reference 6)

[1] References to NUREG-1431 in this safety evaluation refer to revision 5 of this document unless otherwise stated.

3.0 TECHNICAL EVALUATION

As discussed in section 2.1, under 10 CFR 50.92(a), in determining whether an amendment to a license will be issued, the NRC staff is guided by the considerations that govern the issuance of initial licenses to the extent applicable and appropriate. The staff evaluated the request against the regulations, licensing basis, and guidance discussed in section 2. The staff reviewed the proposed changes to determine whether they meet the requirements of 10 CFR 50.36 and provide reasonable assurance that operation with the proposed changes will not endanger the health and safety of the public.

3.1 Change to TS SR 4.8.1.1.2.a.2 and new TS SR 4.8.1.1.2.a.4

Current SR 4.8.1.1.2.a.2 contains the EDG starting test and load-run test in a single SR. The licensee proposed splitting the current combined start and load-run requirement in SR 4.8.1.1.2.a.2 into two separate SRs to align with NUREG-1431. NUREG-1431 presents the standard TS requirements separately as SR 3.8.1.2 (start) and SR 3.8.1.3 (load-run). Similarly, the licensee proposed the Salem SR 4.8.1.1.2.a.2 to contain the starting test and new SR 4.8.1.1.2.a.4 to contain the load-run test.

The licensee proposed the change to SR 4.8.1.1.2.a.2 to accommodate the proposed changes affecting TS 3.8.1.1, Action b.3, which are further evaluated in section 3.3 of this safety evaluation. While no changes are being made to the text of TS 3.8.1.1, Action b.3, dividing the current SR 4.8.1.1.2.a.2 into two separate SRs would remove the requirement to perform the one-hour loaded-run test for the operable EDGs when one EDG is inoperable. In its LAR supplement, the licensee clarified that the proposed removal of the test was not administrative or editorial but, rather, was intended to categorize the division of current SR 4.8.1.1.2.a.2 into two parts and that section 2.3 of the LAR enclosure described the proposed justification for eliminating the test requirement.

Based on its review of the information provided by the licensee, the NRC staff finds that the proposed changes to SR 4.8.1.1.2.a do not adversely impact the availability and testing capability of the onsite and offsite power sources. The NRC staff also finds that the proposed changes are consistent with NUREG-1431, GDC 17, and GDC 18, and in conformance with criteria 24, 39, and 48, as described in the UFSAR. Therefore, the staff finds the proposed changes to SR 4.8.1.1.2.a are acceptable.

3.2 Change to SR 4.8.1.1.2.c

In section 2.4 of the LAR enclosure, the licensee proposed that an allowance in SR 4.8.1.1.2.c, which currently refers to SR 4.8.1.1.2.a.2, be revised to refer to both SR 4.8.1.1.2.a.2 and new SR 4.8.1.1.2.a.4. In its LAR supplement, the licensee clarified that this would be a conforming change resulting from splitting SR 4.8.1.1.2.a.2 into SR 4.8.1.1.2.a.2 and new SR 4.8.1.1.2.a.4. The licensee also indicated that SR 4.8.1.1.2.c allows performance of 4.8.1.1.2.a.2 to meet certain requirements and, since SR 4.8.1.1.2.a.2 is proposed to be split into two parts, both would have to be performed for the licensee to take credit.

Based on its review of the information provided by the licensee, the NRC staff determined that the proposed change to SR 4.8.1.1.2.c is a conforming change resulting from splitting SR 4.8.1.1.2.a.2 and does not adversely impact the availability and testing capability of the onsite and offsite power sources. The NRC staff also finds that the proposed change to SR 4.8.1.1.2.c is consistent with GDC 17, and GDC 18, and in conformance with criteria 24, 39,

and 48, as described in the UFSAR. Therefore, the staff finds the proposed change to SR 4.8.1.1.2.c is acceptable.

3.3 Change to Intent of TS 3.8.1.1, Action b.3

With one EDG inoperable in MODES 1, 2, 3, or 4, TS 3.8.1.1, Action b.3, requires the operable EDGs to be tested in accordance with SR 4.8.1.1.2.a.2 unless specified conditions are met. While the licensee did not propose changes to the text of TS 3.8.1.1, Action b.3, the NRC staff notes that the effect of dividing the current SR 4.8.1.1.2.a.2 into two separate SRs would be to remove the requirement to perform the one-hour loaded-run test for the operable EDGs when one EDG is inoperable.

In section 3.2 of the LAR enclosure, the licensee stated, “Eliminating the requirement in TS 3.8.1.1, Action b.3, to perform the one-hour load run test for the operable EDGs when one EDG is inoperable is consistent with NUREG-1431, TS 3.8.1, Required Action B.3.2.” The licensee also stated:

> Loading the operable EDGs requires that they be paralleled with an offsite power source, making them vulnerable to perturbations in the offsite power system and creates a potential for losing the operable EDGs at a time when the number of onsite power sources to the vital buses has already been reduced. The additional requirement for loading the remaining operable EDGs also presents an additional burden to operations personnel that are focused on actions to restore the EDG to operable status.

The NRC staff reviewed the proposed change to SR 4.8.1.1.2.a.2 to evaluate the impact of the change on TS 3.8.1.1, Action b.3. The NRC staff finds that removing the load-run test requirement for the operable EDGs when one EDG is inoperable is consistent with GL 93-05 regarding improving safety (e.g., reducing risk of the grid perturbation), minimizing equipment degradation (e.g., EDG wear and tear), and removing unnecessary burden on personnel resources (e.g., reducing the amount of testing that the TS require during power operation). The NRC staff also finds that the proposed change to SR 4.8.1.1.2.a does not adversely impact TS 3.8.1.1, Action b.3, requirement, and does not adversely impact the availability and testing capability of the onsite and offsite power sources. The NRC staff also finds that the proposed change to the intent of TS 3.8.1.1, Action b.3, is consistent with GDC 17, and GDC 18, and in conformance with criteria 24, 39, and 48, as described in the UFSAR. Therefore, the NRC staff finds that the proposed change to the intent of TS 3.8.1.1, Action b.3, is acceptable.

3.4 Change to TS 3.8.1.1, Action c

The licensee proposed to revise TS 3.8.1.1, Action c, for consistency with the extended EDG AOT approved in Amendment Nos. 346 and 327 (Reference 7) and to restructure the action for enhanced clarity and alignment with the intent of NUREG-1431. The licensee proposed to divide TS 3.8.1.1, Action c, into five steps, and to remove ambiguity as to the actions that must be taken if the specified requirements are not met.

With one offsite circuit and one EDG inoperable, TS 3.8.1.1, Action c, currently requires demonstrating the operability of the remaining operable EDGs by performing SR 4.8.1.1.2.a.2 within 8 hours. The NRC staff finds that removing the Salem TS 3.8.1.1, Action c, requirement to start and load the operable EDGs within 8 hours when one offsite circuit and one EDG are inoperable would align to the standard TS required actions for NUREG-1431, TS 3.8.1, Condition D. The licensee stated in the LAR that the current required action places demand on

plant operating personnel that can lessen the focus on the actions needed to restore the inoperable EDG or offsite circuit to operable status. The licensee also stated in its LAR that while operating in parallel with an offsite source, the operable EDGs are susceptible to perturbations in the offsite power system when the number of onsite power sources to the vital buses is already reduced.

In section 3.3 of the LAR enclosure, as supplemented, the licensee stated that if the inoperable offsite circuit or the inoperable EDG is restored to operable status before the unit is required to be in COLD SHUTDOWN, then the proposed TS 3.8.1.1, Action c.4, requires the other AC power source (offsite circuit or EDG) to be restored to operable status in accordance with the provisions of TS 3.8.1.1, Action a or b, as appropriate, with the time requirement of that action based on the time of initial loss of the remaining inoperable AC power source. The NRC staff notes that proposed TS 3.8.1.1, Action c.4, would refer to Actions a and b, and, therefore, potentially allow use of the extended AOT rather than preserving the current 72-hour AOT.

In section 3.3 of the LAR enclosure, as supplemented, the licensee stated that if the EDG remained inoperable after the offsite circuit was restored to operable status, then the provisions of TS 3.8.1.1, Action b, would be required to be completed, with the completion times based on the time the EDG became inoperable. The licensee also stated that the proposed TS 3.8.1.1, Action c.4, would remove an inconsistency regarding restoration of the inoperable EDG within 72 hours from time of initial loss if the extended 14-day AOT of TS 3.8.1.1, Action b.4.a.2, was entered.

In section 1.0 of the LAR enclosure, the licensee proposed that TS 3.8.1.1, Actions c and e, be revised for consistency with the extended EDG AOT approved in Amendment Nos. 346 and 327.[2] In its email dated April 7, 2026 (Reference 8), the NRC staff requested the licensee to confirm whether the intent of the proposed change was to revise TS 3.8.1.1, Actions c and e, to align with the changes made to TS 3.8.1.1, Action b.4, by Amendment Nos. 346 and 327 to eliminate the inconsistency regarding restoration of the inoperable EDG within 72 hours from time of initial loss if the extended 14-day AOT of TS 3.8.1.1, Action b.4.a.2, was entered. In its LAR supplement, the licensee confirmed that its intent was to align with the changes made in Amendment Nos. 346 and 327 to eliminate inconsistency.

On February 20, 2026, the NRC issued Amendment Nos. 352 and 334 (Reference 9), which changed several action statements in TS 3.8.1.1 for when LCO 3.8.1.1 is not met. The amendments also changed TS requirements that the licensee proposed to be changed per LAR S22-06 dated October 16, 2025. Therefore, in its LAR supplement, the licensee modified its proposed changes. Specifically, the licensee proposed that TS 3.8.1.1, Action b.2, be repeated as proposed TS 3.8.1.1, Action c.2, to ensure the four-hour requirement would not be exceeded prior to entry into Action b. When entering TS 3.8.1.1, Action b, from the proposed revised Action c, the possibility would exist for both the four-hour completion time for the declaration of inoperability and the six-hour completion time to be in HOT STANDBY in Action b.2 to be exceeded prior to being directed to enter TS 3.8.1.1, Action b. Therefore, in its LAR supplement, the licensee proposed to duplicate existing TS 3.8.1.1, Action b.2, as new TS 3.8.1.1, Action c.2, to eliminate this possibility. The licensee proposed to renumber the other parts of TS 3.8.1.1, Action c, accordingly.

---

[2] Section 3.7 of this safety evaluation describes the NRC staff's evaluation of the licensee's proposed changes to TS 3.8.1.1, Action e.

Based on its review of the information the licensee provided, the NRC staff finds the following:

- Removing the requirement of demonstrating the operability of the remaining operable EDGs by performing SR 4.8.1.1.2.a.2 is acceptable because the proposed change is consistent with the guidance in GL 93-05 regarding improving safety (e.g., reducing risk to EDGs from grid perturbations); minimizing equipment degradation (e.g., EDG wear and tear); and removing unnecessary burden on personnel resources (e.g., reducing the amount of testing that the TS require during power operation).

- Replacing the 72-hour AOT for restoring the remaining EDG to operable status with the reference to TS 3.8.1.1, Action b, is acceptable because this change removes the inconsistency resulting from changes made by Amendment Nos. 346 and 327.

- Restructuring TS 3.8.1.1, Action c, would align the actions with the intent of NUREG-1431.

The NRC staff also finds that the proposed change to TS 3.8.1.1, Action c, does not adversely impact the availability of the onsite and offsite power sources. The NRC staff finds that the proposed changes are consistent with GDC 17 and in conformance with criteria 24 and 39, as described in the UFSAR. Therefore, the staff finds the proposed changes to TS 3.8.1.1, Action c, acceptable.

3.5 Change to TS 3.8.1.1, Action d

With two offsite circuits inoperable, TS 3.8.1.1, Action d.1, requires the three operable EDGs to be tested in accordance with SR 4.8.1.1.2.a.2 unless specified conditions are met. In section 3.4 of the LAR enclosure, the licensee stated that eliminating the TS 3.8.1.1, Action d.1, requirement to start and load the operable EDGs within 8 hours when both offsite circuits are inoperable would be consistent with the required actions for NUREG-1431, TS 3.8.1, Condition C, and contribute to a reduction in unnecessary equipment challenges, burden on personnel resources, and potential degradation to the EDGs. The licensee also stated that periodic EDG testing required by SR 4.8.1.1.2 to demonstrate EDG operability provides a high degree of assurance that the EDGs are operable. In section 2.3 of the LAR enclosure, the licensee stated that in GL 93-05, the NRC staff recognized that removing the requirement to test EDGs with both offsite circuits inoperable can improve safety, decrease equipment degradation, and eliminate an unnecessary burden on personnel resources.

The NRC staff reviewed the proposed removal of TS 3.8.1.1, Action d.1. Based on the review of the information the licensee provided, the NRC staff determined that the proposed change is consistent with the guidance in GL 93-05 regarding improving safety (e.g., reducing risk to the EDGs from grid perturbations), minimizing equipment degradation (e.g., EDG wear and tear), and removing unnecessary burden on personnel resources (e.g., reducing the amount of testing that the TS require during power operation). The NRC staff also finds that the proposed change to TS 3.8.1.1, Action d, does not adversely impact the availability of the onsite and offsite power sources. The NRC staff finds that the proposed change is consistent with GDC 17 and in conformance with criteria 24, and 39, as described in the UFSAR. Therefore, the staff finds the proposed change to TS 3.8.1.1, Action d, acceptable.

3.6 Change to TS 3.8.1.1, Action e

In section 1.0 of the LAR enclosure, the licensee proposed that TS 3.8.1.1, Action e, be revised for consistency with the extended EDG AOT approved in Amendment Nos. 346 and 327 and restructured for enhanced clarity and alignment with the intent of NUREG-1431.

With two or more EDGs inoperable, TS 3.8.1.1, Action e, requires restoration of all three EDGs in 72 hours or HOT STANDBY within 6 hours and COLD SHUTDOWN within the following 30 hours. The proposed changes to Action e are similar to the changes proposed to TS 3.8.1.1, Action c. Specifically, the licensee proposed that TS 3.8.1.1, Action e, be divided into four steps and remove ambiguity as to the actions that must be taken if the specified requirements are not met.

Similar to Action c, TS 3.8.1.1, Action e, would refer to Action b and, thus, allow use of the extended AOT rather than preserving the current 72-hour AOT. Specifically, in section 3.3 of the LAR enclosure, the licensee stated that the proposed TS 3.8.1.1, Action e.3, would remove an inconsistency regarding restoration of the inoperable EDG within 72 hours from time of initial loss if the extended 14-day AOT of TS 3.8.1.1, Action b.4.a.2, was entered.

Amendment Nos. 352 and 334 (Reference 9) modified TS 3.8.1.1, Action e, to add a risk-informed completion time (RICT) and footnote (2), which stated that the RICT Program cannot be entered when a loss of function occurs. In its LAR supplement dated May 14, 2026, the licensee proposed to delete footnote (2) because proposed TS 3.8.1.1, Action e, would not contain an explicit allowance to use the RICT Program. Rather, it would allow an implicit allowance to use the RICT program via TS 3.8.1.1, Actions a or b. The NRC staff finds the proposed deletion of footnote (2), which would no longer be applicable to TS 3.8.1.1, Action e, acceptable.

Based on the review of the information the licensee provided, the NRC staff finds that replacing the 72-hour AOT for restoring the remaining EDG to operable status with the reference to TS 3.8.1.1, Action b, is acceptable because this change removes the inconsistency that resulted from changes under Amendment Nos. 346 and 327 (Reference 7). The NRC staff finds that restructuring TS 3.8.1.1, Action e, would remove ambiguity as to the actions that must be taken if the specified requirements are not met, and would better align the action with the intent of similar requirements in NUREG-1431. The NRC staff also finds that the proposed changes to TS 3.8.1.1, Action e, do not adversely impact the availability of the onsite and offsite power sources. The NRC staff finds that the proposed changes are consistent with GDC 17 and in conformance with criteria 24 and 39, as described in the UFSAR. Therefore, the staff finds the proposed changes to TS 3.8.1.1, Action e, acceptable.

3.7 Technical Evaluation Conclusion

Based on the above evaluation, the NRC staff finds that the proposed changes would not adversely impact the availability and testing capability of the onsite and offsite power sources. The proposed changes would also reduce risk to EDGs from grid perturbations, remove unnecessary burden on plant operational personnel, and reduce potential mechanical degradation of the EDGs.

The NRC staff finds that the proposed changes are consistent with NUREG-1431, GDC-17, and GDC-18 and are supported, in part, by GL 93-05. The staff finds that the proposed changes are in conformance with criteria 24, 39, and 48, as described in the UFSAR. The staff concludes

that the proposed changes comply with 10 CFR 50.36(c)(2)(i) and (c)(3) and, therefore, are acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the NRC staff notified the State of New Jersey official on June 4, 2026 (Reference 10), of the proposed issuance of the amendments. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

This action relates to authorizations under 10 CFR Part 50 with respect to installation or use of a facility component and changes surveillance requirements. The NRC staff has determined that any ground disturbance is limited to previously disturbed areas. Additionally, the NRC staff has determined that the action involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, no significant increase in individual or cumulative public or occupational radiation exposure, and no significant increase in the potential for or consequences from radiological accidents. Finally, the NRC staff has determined that a categorical exclusion applies and that special circumstances under 10 CFR 51.22, "Categorical exclusions," are not present that would preclude reliance on the categorical exclusion. Accordingly, this action meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(d)(1) and (d)(8). Pursuant to 10 CFR 51.22, no environmental impact statement or environmental assessment need be prepared in connection with the action.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner; (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations; and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

7.0 ABBREVIATIONS

AC (or A.C.)	alternating current
AEC	Atomic Energy Commission
AOT	allowed outage time
CFR	Code of Federal Regulations
EDG	emergency diesel generator
GDC	general design criteria
GL	generic letter
kw	kilowatt(s)
LAR	license amendment request
LCO	limiting condition for operation
NRC	U.S. Nuclear Regulatory Commission
NSHC	no significant hazards consideration
RICT	risk-informed completion time
SR	surveillance requirement
TS	technical specification(s)
UFSAR	Updated Final Safety Analysis Report

8.0 REFERENCES

1 Sharbaugh, D., PSEG Nuclear LLC, letter LR-N25-0053 to U.S. Nuclear Regulatory Commission, “License Amendment Request: A.C. Sources – Operating,” October 16, 2025, Agencywide Documents Access and Management System (ADAMS) Accession No. ML25290A331.

2 Sharbaugh, D., PSEG Nuclear LLC, letter LR-N26-0033 to U.S. Nuclear Regulatory Commission, “Supplement to License Amendment Request: A.C. Sources – Operating,” May 14, 2026, ML26134A408.

3 U.S. Nuclear Regulatory Commission, Monthly Notice, “Applications and Amendments to Facility Operating Licenses and Combined Licenses Involving No Significant Hazards Considerations Office of the Federal Register,” Volume 91, Page 7307 (91 FR 7307), February 17, 2026.

4 U.S. Nuclear Regulatory Commission, NUREG-1431, Revision 5, “Standard Technical Specifications, Westinghouse Plants,” Volume 1, “Specifications,” and Volume 2, “Bases,” September 2021, ML21259A155 and ML21259A159.

5 Jurek, S., PSEG Nuclear LLC, letter LR-N25-0078 to U.S. Nuclear Regulatory Commission, “Submittal of Salem Generating Station Updated Final Safety Analysis Report, Revision 35, Submittal of Salem Generating Station Technical Specification Bases Changes, Summary of Revised Regulatory Commitments for Salem, Summary of Changes to PSEG Nuclear LLC Quality Assurance Topical Report, , NO-AA-10, and 10 CFR 54.37(b) Review Results for Salem,” November 20, 2025, ML25324A037 (enclosure 1, ML25324A039).

6 U.S. Nuclear Regulatory Commission, Generic Letter 93-05, “Line-Item Technical Specifications Improvements to Reduce Surveillance Requirements for Testing,” September 27, 1993, ML031070342.

7 Kim, J., U.S. Nuclear Regulatory Commission, letter to Carr, E., PSEG Nuclear LLC, “Salem Nuclear Generating Station, Unit Nos. 1 and 2 – Issuance of Amendment Nos. 346 and 327 Re: Revise Technical Specifications to Extend Allowable Outage Time for Inoperable Emergency Diesel Generator (EPID L-2022-LLA-0095),” May 2, 2023, ML23081A466.

8 Klett, A., U.S. Nuclear Regulatory Commission, email to Cizin, B., PSEG Nuclear LLC, “RAI/RCI for Salem EDG TS LAR (EPID L-2025-LLA-0169),” April 7, 2026, ML26097A307.

9 Guzman, R., U.S. Nuclear Regulatory Commission, letter to McFeaters, C., PSEG Nuclear LLC, “Salem Nuclear Generating Station, Unit Nos. 1 and 2 – Issuance of Amendment Nos. 352 and 334 Re: Adoption of TSTF-505, ‘Provide Risk-Informed Extended Completion Times – RITSTF Initiative 4B’ and TSTF-591, ‘Revised Risk Informed Completion Time (RICT) Program’ (EPID L-2022-LLA-0095),” February 20, 2026, ML26023A048.

10 Klett, A., U.S. Nuclear Regulatory Commission, email to Pfaff, A., New Jersey Department of Environmental Protection, “NRC Consultation with the State of New Jersey re. Proposed NSHC for Salem Units 1 and 2 License Amendment Request S22-06,” June 4, 2026, ML26159A283.

DATE: August 18, 2026

SUBJECT: SALEM NUCLEAR GENERATING STATION, UNIT NOS. 1 AND 2 – ISSUANCE OF AMENDMENT NOS. 356 AND 338 REGARDING TECHNICAL SPECIFICATIONS FOR ALTERNATING CURRENT SOURCES (EPID L-2025-LLA-0169) DATED AUGUST 18, 2026

DISTRIBUTION:
PUBLIC
RidsNrrPMSalem Resource
RidsNrrLAKZeleznock Resource
RidsNrrDlp1Orlb1 Resource
RidsRgn1MailCenter Resource
RidsACRS_MailCTR Resource
RidsNrrDexEeeb Resource
RidsNrrDssStsb Resource
NKhan, NRR
KNguyen, NRR
KWest, NRR

ADAMS Accession Nos.:
Package: ML26181A017
Amendment: ML26181A019
e-Concurrence Case: 20260630-50004