UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 25, 2026

Mr. Joe Quinn
Vice President, Regulatory Support
Tennessee Valley Authority
1101 Market Street, LP 4A-C
Chattanooga, TN 37402-2801

SUBJECT: WATTS BAR NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENT NOS. 181 AND 86 REGARDING REBASELINE OF TECHNICAL SPECIFICATION SECTIONS 3.4 AND 3.9 (EPID L-2025-LLA-0138)

Dear Mr. Quinn:

The U.S. Nuclear Regulatory Commission (Commission) has issued the enclosed Amendment No. 181 to Facility Operating License No. NPF-90 and Amendment No. 86 to Facility Operating License No. NPF-96 for the Watts Bar Nuclear Plant (Watts Bar), Units 1 and 2, respectively. These amendments are in response to your application dated August 25, 2025, as supplemented by letter dated February 24, 2026.

The amendments revise various technical specifications (TS) within the Watts Bar, Units 1 and 2, Technical Specification Sections 1.4, 3.4, 3.7, and 3.9 by adopting several Technical Specifications Task Force (TSTF) Travelers.

A copy of our related safety evaluation is also enclosed. A notice of issuance will be included in the Commission's bi-weekly *Federal Register* notice.

Sincerely,

/RA/

Kimberly J. Green, Senior Project Manager
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Docket Nos. 50-390 and 50-391

Enclosures:
1. Amendment No. 181 to NPF-90
2. Amendment No. 86 to NPF-96
3. Safety Evaluation

cc: Listserv

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-390

WATTS BAR NUCLEAR PLANT, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 181
License No. NPF-90

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Tennessee Valley Authority (TVA, the licensee) dated August 25, 2025, as supplemented by letter dated February 24, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-90 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A as revised through Amendment No. 181 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 180 days.

FOR THE NUCLEAR REGULATORY COMMISSION

MICHAEL MAHONEY

Digitally signed by MICHAEL MAHONEY
Date: 2026.06.25 14:24:49 -04'00'

Michael Mahoney, Acting Chief
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Operating License
and Technical Specifications

Date of Issuance: June 25, 2026

ATTACHMENT TO AMENDMENT NO. 181

WATTS BAR NUCLEAR PLANT, UNIT 1

FACILITY OPERATING LICENSE NO. NPF-90

DOCKET NO. 50-390

Replace page 3 of Facility Operating License No. NPF-90 with the attached revised page 3. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain a marginal line indicating the area of change.

Remove Page	Insert Page	Remove Page	Insert Page
1.1-5	1.1-5	3.4-20	3.4-20
1.4-1	1.4-1	3.4-22	3.4-22
1.4-2	1.4-2	3.4-23	3.4-23
--	1.4-5	3.4-24	3.4-24
--	1.4-6	--	3.4-24a
--	1.4-7	3.4-25	3.4-25
3.4-3	3.4-3	3.4-27	3.4-27
3.4-4	3.4-4	3.4-29	3.4-29
3.4-8	3.4-8	3.4-36	3.4-36
3.4-9	3.4-9	3.4-37	3.4-37
3.4-10	3.4-10	3.7-9	3.7-9
3.4-11	3.4-11	3.9-1	3.9-1
3.4-12	3.4-12	3.9-4	3.9-4
3.4-13	3.4-13	3.9-8	3.9-8
3.4-14	3.4-14	3.9-9	3.9-9
3.4-15	3.4-15	3.9-10	3.9-10
--	3.4-15a	3.9-11	3.9-11
3.4-16	3.4-16	--	3.9-11a
3.4-17	3.4-17	3.9-12	3.9-12
3.4-18	3.4-18		

(4) TVA, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required, any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis, instrument calibration, or other activity associated with radioactive apparatus or components; and

(5) TVA, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified or incorporated below.

(1) Maximum Power Level

TVA is authorized to operate the facility at reactor core power levels not in excess of 3459 megawatts thermal.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 181 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Safety Parameter Display System (SPDS) (Section 18.2 of SER Supplements 5 and 15)

Prior to startup following the first refueling outage, TVA shall accomplish the necessary activities, provide acceptable responses, and implement all proposed corrective actions related to having the Watts Bar Unit 1 SPDS operational.

(4) Vehicle Bomb Control Program (Section 13.6.9 of SSER 20)

During the period of the exemption granted in paragraph 2.D.(3) of this license, in implementing the power ascension phase of the approved initial test program, TVA shall not exceed 50% power until the requirements of 10 CFR 73.55(c)(7) and (8) are fully implemented. TVA shall submit a letter under oath or affirmation when the requirements of 73.55(c)(7) and (8) have been fully implemented.

1.1 Definitions (continued)

PHYSICS TESTS (continued)	a. Described in Chapter 14, Initial Test Program of the FSAR; b. Authorized under the provisions of 10 CFR 50.59; or c. Otherwise approved by the Nuclear Regulatory Commission.
PRESSURE AND TEMPERATURE LIMITS REPORT	The PTLR is the unit specific document that provides the RCS pressure and temperature limits for heatup, cooldown, low temperature operation, criticality, and hydrostatic testing as well as heatup and cooldown rates and the cold overpressure mitigation system for the current reactor vessel fluence period. These pressure and temperature limits shall be determined for each fluence period in accordance with Specification 5.9.6. Plant operation within these operating limits is addressed in LCO 3.4.3, "RCS Pressure and Temperature (P/T) Limits," and LCO 3.4.12, "Cold Overpressure Mitigation System (COMS)."
QUADRANT POWER TILT RATIO (QPTR)	QPTR shall be the ratio of the maximum upper excore detector calibrated output to the average of the upper excore detector calibrated outputs, or the ratio of the maximum lower excore detector calibrated output to the average of the lower excore detector calibrated outputs, whichever is greater.
RATED THERMAL POWER (RTP)	RTP shall be a total reactor core heat transfer rate to the reactor coolant of 3459 MWt.
REACTOR TRIP SYSTEM (RTS) RESPONSE TIME	The RTS RESPONSE TIME shall be that time interval from when the monitored parameter exceeds its RTS trip setpoint at the channel sensor until loss of stationary gripper coil voltage. The response time may be measured by means of any series of sequential, overlapping, or total steps so that the entire response time is measured. In lieu of measurement, response time may be verified for selected components provided that the components and the methodology for verification have been previously reviewed and approved by the NRC (including transmitters in the Online Monitoring Program), or the components have been evaluated in accordance with an NRC approved methodology.
SHUTDOWN MARGIN (SDM)	SDM shall be the instantaneous amount of reactivity by which the reactor is subcritical or would be subcritical from its present condition assuming:

(continued)

1.0 USE AND APPLICATION

1.4 Frequency

PURPOSE

The purpose of this section is to define the proper use and application of Frequency requirements.

DESCRIPTION

Each Surveillance Requirement (SR) has a specified Frequency in which the Surveillance must be met in order to meet the associated LCO. An understanding of the correct application of the specified Frequency is necessary for compliance with the SR.

The "specified Frequency" is referred to throughout this section and each of the Specifications of Section 3.0, Surveillance Requirement (SR) Applicability. The "specified Frequency" consists of the requirements of the Frequency column of each SR as well as certain Notes in the Surveillance column that modify performance requirements.

Sometimes special situations dictate when the requirements of a Surveillance are to be met. They are "otherwise stated" conditions allowed by SR 3.0.1. They may be stated as clarifying Notes in the Surveillance, as part of the Surveillance or both.

Situations where a Surveillance could be required (i.e., its Frequency could expire), but where it is not possible or not desired that it be performed until sometime after the associated LCO is within its Applicability, represent potential SR 3.0.4 conflicts. To avoid these conflicts, the SR (i.e., the Surveillance or the Frequency) is stated such that it is only "required" when it can be and should be performed. With an SR satisfied, SR 3.0.4 imposes no restriction.

The use of "met" or "performed" in these instances conveys specific meanings. A Surveillance is "met" only when the acceptance criteria are satisfied. Known failure of the requirements of a Surveillance, even without a Surveillance specifically being "performed," constitutes a Surveillance not "met." "Performance" refers only to the requirement to specifically determine the ability to meet the acceptance criteria.

Some Surveillances contain notes that modify the Frequency of performance or the conditions during which the acceptance criteria must be satisfied. For these Surveillances, the MODE-entry restrictions of SR 3.0.4 may not apply. Such a Surveillance is not required to be performed prior to entering a MODE or other specified condition in the Applicability of the associated LCO if any of the following three conditions are satisfied:

(continued)

1.4 Frequency

DESCRIPTION (continued)

a. The Surveillance is not required to be met in the MODE or other specified condition to be entered, or

b. The Surveillance is required to be met in the MODE or other specified condition to be entered, but has been performed within the specified Frequency (i.e., it is current) and is known not to be failed, or

c. The Surveillance is required to be met, but not performed, in the MODE or other specified condition to be entered, and is known not to be failed.

Examples 1.4-3, 1.4-4, 1.4-5, and 1.4-6 discuss these special situations.

EXAMPLES

The following examples illustrate the various ways that Frequencies are specified. In these examples, the Applicability of the LCO (LCO not shown) is MODES 1, 2, and 3.

EXAMPLE 1.4-1

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
Perform CHANNEL CHECK.	12 hours

Example 1.4-1 contains the type of SR most often encountered in the Technical Specifications (TS). The Frequency specifies an interval (12 hours) during which the associated Surveillance must be performed at least one time. Performance of the Surveillance initiates the subsequent interval. Although the Frequency is stated as 12 hours, an extension of the time interval to 1.25 times the interval specified in the Frequency is allowed by SR 3.0.2 for operational flexibility. The measurement of this interval continues at all times, even when the SR is not required to be met per SR 3.0.1 (such as when the equipment is inoperable, a variable is outside specified limits, or the unit is outside the Applicability of the LCO). If the interval specified by SR 3.0.2 is exceeded while the unit is in a MODE or other specified condition in the Applicability of the LCO, and the performance of the Surveillance is not otherwise modified (refer to Example 1.4-3), then SR 3.0.3 becomes applicable.

If the interval as specified by SR 3.0.2 is exceeded while the unit is not in a MODE or other specified condition in the Applicability of the LCO for which performance of the SR is required, the Surveillance must be performed within the Frequency requirements of SR 3.0.2 prior to entry into the MODE or other specified condition. Failure to do so would result in a violation of SR 3.0.4.

(continued)

EXAMPLES
(continued)

EXAMPLE 1.4-4

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
----------NOTE---------- Only required to be met in MODE 1. ----------	
Verify leakage rates are within limits.	24 hours

Example 1.4-4 specifies that the requirements of this Surveillance do not have to be met until the unit is in MODE 1. The interval measurement for the Frequency of this Surveillance continues at all times, as described in Example 1.4-1. However, the Note constitutes an "otherwise stated" exception to the Applicability of this Surveillance. Therefore, if the Surveillance were not performed within the 24 hour interval (plus the extension allowed by SR 3.0.2), but the unit was not in MODE 1, there would be no failure of the SR nor failure to meet the LCO. Therefore, no violation of SR 3.0.4 occurs when changing MODES, even with the 24 hour Frequency exceeded, provided the MODE change was not made into MODE 1. Prior to entering MODE 1 (assuming again that the 24 hour Frequency were not met), SR 3.0.4 would require satisfying the SR.

(continued)

EXAMPLES
(continued)

EXAMPLE 1.4-5

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
----------NOTE---------- Only required to be performed in MODE 1. ----------	
Perform complete cycle of the valve.	7 days

The interval continues, whether or not the unit operation is in MODE 1, 2, or 3 (the assumed Applicability of the associated LCO) between performances.

As the Note modifies the required performance of the Surveillance, the Note is construed to be part of the "specified Frequency." Should the 7 day interval be exceeded while operation is not in MODE 1, this Note allows entry into and operation in MODES 2 and 3 to perform the Surveillance. The Surveillance is still considered to be performed within the "specified Frequency" if completed prior to entering MODE 1. Therefore, if the Surveillance were not performed within the 7 day (plus the extension allowed by SR 3.0.2) interval, but operation was not in MODE 1, it would not constitute a failure of the SR or failure to meet the LCO. Also, no violation of SR 3.0.4 occurs when changing MODES, even with the 7 day Frequency not met, provided operation does not result in entry into MODE 1.

Once the unit reaches MODE 1, the requirement for the Surveillance to be performed within its specified Frequency applies and would require that the Surveillance had been performed. If the Surveillance were not performed prior to entering MODE 1, there would then be a failure to perform a Surveillance within the specified Frequency, and the provisions of SR 3.0.3 would apply.

(continued)

EXAMPLES
(continued)

EXAMPLE 1.4-6

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
----------NOTE---------- Not required to be met in MODE 3. ----------	
Verify parameter is within limits.	24 hours

Example 1.4-6 specifies that the requirements of this Surveillance do not have to be met while the unit is in MODE 3 (the assumed Applicability of the associated LCO is MODES 1, 2, and 3). The interval measurement for the Frequency of this Surveillance continues at all times, as described in Example 1.4-1. However, the Note constitutes an "otherwise stated" exception to the Applicability of this Surveillance. Therefore, if the Surveillance were not performed within the 24 hour interval (plus the extension allowed by SR 3.0.2), and the unit was in MODE 3, there would be no failure of the SR nor failure to meet the LCO. Therefore, no violation of SR 3.0.4 occurs when changing MODES to enter MODE 3, even with the 24 hour Frequency exceeded, provided the MODE change does not result in entry into MODE 2. Prior to entering MODE 2 (assuming again that the 24 hour Frequency was not met), SR 3.0.4 would require satisfying the SR.

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.2 RCS Minimum Temperature for Criticality

LCO 3.4.2 Each RCS loop average temperature (T_{avg}) shall be $\geq$ 551°F.

APPLICABILITY: MODE 1,
MODE 2 with $k_{eff} \geq 1.0$.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. T_{avg} in one or more RCS loops not within limit.	A.1 Be in MODE 2 with $k_{eff} < 1.0$.	30 minutes

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
SR 3.4.2.1 Verify RCS T_{avg} in each loop $\geq$ 551°F.	In accordance with the Surveillance Frequency Control Program

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3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.5 RCS Loops - MODE 3

LCO 3.4.5 Two RCS loops shall be OPERABLE, and either:

a. Two RCS loops shall be in operation when the Rod Control System is capable of rod withdrawal; or

b. One RCS loop shall be in operation when the Rod Control System is not capable of rod withdrawal.

----------NOTE----------
All reactor coolant pumps may be removed from operation for ≤ 1 hour per 8 hour period provided:

a. No operations are permitted that would cause introduction of coolant into the RCS with boron concentration less than required to meet the SDM of LCO 3.1.1; and

b. Core outlet temperature is maintained at least 10°F below saturation temperature.

APPLICABILITY: MODE 3.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One required RCS loop inoperable.	A.1 Restore required RCS loop to OPERABLE status.	72 hours
B. Required Action and associated Completion Time of Condition A not met.	B.1 Be in MODE 4.	12 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. One required RCS loop not in operation with Rod Control System capable of rod withdrawal.	C.1 Restore required RCS loop to operation. OR C.2 Place the Rod Control System in a condition incapable of rod withdrawal.	1 hour 1 hour
D. Two required RCS loops inoperable. OR Required RCS loop(s) not in operation.	D.1 Place the Rod Control System in a condition incapable of rod withdrawal. AND D.2 Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet SDM of LCO 3.1.1. AND D.3 Initiate action to restore one RCS loop to OPERABLE status and operation.	Immediately Immediately Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.5.1	Verify required RCS loops are in operation.	In accordance with the Surveillance Frequency Control Program
SR 3.4.5.2	Verify steam generator secondary side water levels are greater than or equal to 32% narrow range for required RCS loops.	In accordance with the Surveillance Frequency Control Program
SR 3.4.5.3	----------NOTE---------- Not required to be performed until 24 hours after a required pump is not in operation. ---------- Verify correct breaker alignment and indicated power are available to each required pump.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.6 RCS Loops - MODE 4

LCO 3.4.6 Two loops shall be OPERABLE, and consist of either:

a. Any combination of RCS loops and residual heat removal (RHR) loops, and one loop shall be in operation, when the rod control system is not capable of rod withdrawal; or

b. Two RCS loops, and both loops shall be in operation, when the rod control system is capable of rod withdrawal.

----------NOTES----------

1. No RCP shall be started with any RCS cold leg temperature less than or equal to the COMS arming temperature specified in the PTLR unless the secondary side water temperature of each steam generator (SG) is ≤ 50°F above each of the RCS cold leg temperatures.

2. For the initial 7 hours after entry into MODE 3 from MODE 1 or MODE 2, two loops shall consist of:

   a. Two RCS loops with one loop in operation when the rod control system is not capable of rod withdrawal; or

   b. Two RCS loops with both loops in operation when the rod control system is capable of rod withdrawal.

3. Average reactor coolant temperature shall be maintained > 200ºF for the initial 7 hours after entry into MODE 3 from MODE 1 or MODE 2.

APPLICABILITY: MODE 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One required loop inoperable. AND Less than 7 hours since entry into MODE 3 from MODE 1 or MODE 2.	A.1 Initiate action to restore a second required RCS or RHR loop to OPERABLE status. AND A.2 ----------NOTE---------- Only required if RHR loop is OPERABLE. ---------- Be in MODE 5.	Immediately 24 hours
B. One required RCS loop not in operation with Rod Control System capable of rod withdrawal.	B.1 Restore required RCS loop to operation. OR B.2 Place the Rod Control System in a condition incapable of rod withdrawal.	1 hour 1 hour
C. Two required loops inoperable. OR Required loop not in operation.	C.1 Place the Rod Control System in a condition incapable of rod withdrawal. AND C.2 Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet SDM of LCO 3.1.1. AND C.3 Initiate action to restore one required loop to OPERABLE status and operation.	Immediately Immediately Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.6.1	Verify two RCS loop are in operation when the rod control system is capable of rod withdrawal.	In accordance with the Surveillance Frequency Control Program
SR 3.4.6.2	Verify required RHR or RCS loop is in operation when the rod control system is not capable of rod withdrawal.	In accordance with the Surveillance Frequency Control Program
SR 3.4.6.3	Verify SG secondary side water levels are greater than or equal to 32% narrow range for required RCS loops.	In accordance with the Surveillance Frequency Control Program
SR 3.4.6.4	----------NOTE---------- Not required to be performed until 24 hours after a required pump is not in operation. ---------- Verify correct breaker alignment and indicated power are available to each required pump.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.7 RCS Loops - MODE 5, Loops Filled

LCO 3.4.7 One residual heat removal (RHR) loop shall be OPERABLE and in operation, and either:

a. One additional RHR loop shall be OPERABLE; or

b. The secondary side water level of at least two steam generators (SGs) shall be greater than or equal to 32% narrow range.

----------NOTES----------

1. One required RHR loop may be inoperable for up to 2 hours for surveillance testing provided that the other RHR loop is OPERABLE and in operation.

2. No reactor coolant pump shall be started with one or more RCS cold leg temperatures less than or equal to the COMS arming temperature specified in the PTLR unless the secondary side water temperature of each SG is ≤ 50°F above each of the RCS cold leg temperatures.

3. All RHR loops may be removed from operation during planned heatup to MODE 4 when at least one RCS loop is in operation.

APPLICABILITY: MODE 5 with RCS loops filled.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One required RHR loop inoperable. AND One RHR loop OPERABLE.	A.1 Initiate action to restore a second RHR loop to OPERABLE status. OR A.2 Initiate action to restore required SGs secondary side water levels to within limits.	Immediately Immediately

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
B. One or more required SGs with secondary side water level not within limit. AND One RHR loop OPERABLE.	B.1	Initiate action to restore one RHR loop to OPERABLE status. AND	Immediately
	B.2	Initiate action to restore required SGs secondary side water level to within limit.	Immediately
C. No required RHR loops OPERABLE. OR Required RHR loop not in operation.	C.1	Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet SDM of LCO 3.1.1. AND	Immediately
	C.2	Initiate action to restore one RHR loop to OPERABLE status and operation.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.7.1	Verify required RHR loop is in operation.	In accordance with the Surveillance Frequency Control Program

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.4.7.2	Verify SG secondary side water level is greater than or equal to 32% narrow range in required SGs.	In accordance with the Surveillance Frequency Control Program
SR 3.4.7.3	----------NOTE---------- Not required to be performed until 24 hours after a required pump is not in operation. ---------- Verify correct breaker alignment and indicated power are available to each required RHR pump.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.8 RCS Loops - MODE 5, Loops Not Filled

LCO 3.4.8 Two residual heat removal (RHR) loops shall be OPERABLE and one RHR loop shall be in operation.

----------NOTES----------

1. All RHR pumps may be removed from operation for ≤ 15 minutes when switching from one loop to another provided:

   a. The core outlet temperature is maintained > 10°F below saturation temperature.

   b. No operations are permitted that would cause introduction of coolant into the RCS with boron concentration less than required to meet the SDM of LCO 3.1.1; and

   c. No draining operations to further reduce the RCS water volume are permitted.

2. One RHR loop may be inoperable for ≤ 2 hours for surveillance testing provided that the other RHR loop is OPERABLE and in operation.

APPLICABILITY: MODE 5 with RCS loops not filled.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One required RHR loop inoperable.	A.1 Initiate action to restore RHR loop to OPERABLE status.	Immediately

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. No required RHR loop OPERABLE. OR Required RHR loop not in operation.	B.1 Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet SDM of LCO 3.1.1. AND B.2 Initiate action to restore one RHR loop to OPERABLE status and operation.	Immediately Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.8.1	Verify required RHR loop is in operation.	In accordance with the Surveillance Frequency Control Program
SR 3.4.8.2	----------NOTE---------- Not required to be performed until 24 hours after a required pump is not in operation. ---------- Verify correct breaker alignment and indicated power are available to each required RHR pump.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.9 Pressurizer

LCO 3.4.9 The pressurizer shall be OPERABLE with:

a. Pressurizer water level ≤ 92%; and

b. Two groups of pressurizer heaters OPERABLE with the capacity of each group ≥ 150 kW.

APPLICABILITY: MODES 1, 2, and 3.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Pressurizer water level not within limit.	A.1 Be in MODE 3.	6 hours
	AND	
	A.2 Fully insert all rods.	6 hours
	AND	
	A.3 Place Rod Control System in a condition incapable of rod withdrawal.	6 hours
	AND	
	A.4 Be in MODE 4.	12 hours
B. One required group of pressurizer heaters inoperable.	B.1 Restore required group of pressurizer heaters to OPERABLE status.	72 hours
C. Required Action and associated Completion Time of Condition B not met.	C.1 Be in MODE 3.	6 hours
	AND	
	C.2 Be in MODE 4.	12 hours

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.10 Pressurizer Safety Valves

LCO 3.4.10 Three pressurizer safety valves shall be OPERABLE with lift settings ≥ 2410 psig and ≤ 2560 psig.

APPLICABILITY: MODES 1, 2, and 3.
MODE 4 with all RCS cold leg temperatures greater than the COMS arming temperature specified in the PTLR.

----------NOTE----------
The lift settings are not required to be within the LCO limits during MODE 3 and MODE 4 with all RCS cold leg temperatures greater than the COMS arming temperature specified in the PTLR for the purpose of setting the pressurizer safety valves under ambient (hot) conditions. This exception is allowed for 54 hours following entry into MODE 3 provided a preliminary cold setting was made prior to heatup.

ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	One pressurizer safety valve inoperable.	A.1	Restore valve to OPERABLE status.	15 minutes
B.	Required Action and associated Completion Time not met. OR Two or more pressurizer safety valves inoperable.	B.1 AND B.2	Be in MODE 3. Be in MODE 4 with any RCS cold leg temperature less than or equal to the COMS arming temperature specified in the PTLR.	6 hours 12 hours

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.11 Pressurizer Power Operated Relief Valves (PORVs)

LCO 3.4.11 Each PORV and associated block valve shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTIONS

---NOTE---
Separate Condition entry is allowed for each PORV and each block valve.

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	One or more PORVs inoperable and capable of being manually cycled.	A.1	Close and maintain power to associated block valve.	1 hour
B.	One PORV inoperable and not capable of being manually cycled.	B.1	Close associated block valve.	1 hour
		AND		
		B.2	Remove power from associated block valve.	1 hour
		AND		
		B.3	Restore PORV to OPERABLE status.	72 hours OR In accordance with the RISK Informed Completion Time Program

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. One block valve inoperable.	----------NOTE---------- Required Actions C.1 and C.2 do not apply when block valve is inoperable solely as a result of complying with Required Actions B.2 or E.2. ----------	
	C.1 Place associated PORV in manual control.	1 hour
	AND	
	C.2 Restore block valve to OPERABLE status.	72 hours OR In accordance with the Risk Informed Completion Time Program
D. Required Action and associated Completion Time of Condition A, B, or C not met.	D.1 Be in MODE 3.	6 hours
	AND	
	D.2 Be in MODE 4.	12 hours

(continued)

ACTIONS (continued)

CONDITION		REQUIRED ACTION		COMPLETION TIME
E.	Two PORVs inoperable and not capable of being manually cycled.	E.1	Close associated block valves.	1 hour
		AND		
		E.2	Remove power from associated block valves.	1 hour
		AND		
		E.3	Be in MODE 3.	6 hours
		AND		
		E.4	Be in MODE 4.	12 hours
F.	Two block valves inoperable.	F.1	----------NOTE---------- Required Action F.1 does not apply when block valve is inoperable solely as a result of complying with Required Actions B.2 or E.2. ----------	
			Restore one block valve to OPERABLE status.	2 hours
G.	Required Action and associated Completion Time of Condition F not met.	G.1	Be in MODE 3.	6 hours
		AND		
		G.2	Be in MODE 4.	12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.11.1	----------NOTES---------- 1. Not required to be performed with block valve closed in accordance with the Required Actions of this LCO. 2. Only required to be performed in Modes 1 and 2. ---------- Perform a complete cycle of each block valve.	In accordance with the Surveillance Frequency Control Program
SR 3.4.11.2	----------NOTE---------- Only required to be performed in Modes 1 and 2. ---------- Perform a complete cycle of each PORV.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.12 Cold Overpressure Mitigation System (COMS)

LCO 3.4.12 A COMS System shall be OPERABLE with a maximum of one charging pump and no safety injection pump capable of injecting into the RCS and the accumulators isolated and one of the following pressure relief capabilities:

a. Two power operated relief valves (PORVs) with lift settings within the limits specified in the PTLR, or

b. One PORV with a lift setting within the limits specified in the PTLR and the RHR suction relief valve with a setpoint ≥ 436.5 psig and ≤ 463.5 psig, or

c. The RCS depressurized and an RCS vent capable of relieving > 475 gpm water flow.

----------NOTES----------

1. Two charging pumps may be made capable of injecting for less than or equal to one hour for pump swap operations.

2. Accumulator may be unisolated when accumulator pressure is less than the maximum RCS Pressure for the existing RCS cold leg temperature allowed by the P/T limit curves provided in the PTLR.

3. One safety injection pump and one charging pump may be capable of injecting into the RCS for the purpose of testing in MODE 5 or MODE 6 when the reactor vessel head is on, provided the pressurizer manway cover is removed to provide a vent path for adequate pressure relief.

APPLICABILITY: MODE 4 with any RCS cold leg temperature ≤ COMS arming temperature specified in the PTLR,
MODE 5,
MODE 6 when the reactor vessel head is on.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
D. Required Action and associated Completion Time of Condition C not met.	D.1 Increase RCS cold leg temperature to greater than the COMS arming temperature specified in the PTLR. OR D.2 Depressurize affected accumulator to less than the maximum RCS pressure for existing cold leg temperature allowed in the PTLR.	12 hours 12 hours
E. One required RCS relief valve inoperable in MODE 4.	E.1 Restore required RCS relief valve to OPERABLE status.	7 days
F. One required RCS relief valve inoperable in MODE 5 or 6.	F.1 Restore required RCS relief valve to OPERABLE status.	24 hours
G. Two required RCS relief valves inoperable. OR Required Action and associated Completion Time of Condition A, B, D, E, or F not met. OR COMS inoperable for any reason other than Condition A, B, C, D, E, or F.	G.1 Depressurize RCS and establish RCS vent.	12 hours

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.4.12.4	Verify required RCS vent open.	In accordance with the Surveillance Frequency Control Program
SR 3.4.12.5	Verify PORV block valve is open for each required PORV.	In accordance with the Surveillance Frequency Control Program
SR 3.4.12.6	Verify both RHR suction isolation valves are locked open with operator power removed for the required RHR suction relief valve.	In accordance with the Surveillance Frequency Control Program
SR 3.4.12.7	----------NOTE---------- Not required to be performed until 12 hours after decreasing RCS cold leg temperature to less than or equal to the COMS arming temperature specified in the PTLR. ---------- Perform a COT on each required PORV, excluding actuation.	In accordance with the Surveillance Frequency Control Program
SR 3.4.12.8	Perform CHANNEL CALIBRATION for each required PORV actuation channel.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.15 RCS Leakage Detection Instrumentation

LCO 3.4.15 The following RCS leakage detection instrumentation shall be OPERABLE:

a. One containment pocket sump level monitor; and

b. One lower containment atmosphere particulate radioactivity monitor.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Required containment pocket sump level monitor inoperable.	A.1 ----------NOTE---------- Not required until 12 hours after establishment of steady state operation. ----------	
	Perform SR 3.4.13.1.	Once per 24 hours
	AND	
	A.2 Restore required containment pocket sump level monitor to OPERABLE status.	30 days

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. Required containment atmosphere particulate radioactivity monitor inoperable.	B.1.1 Analyze grab samples of the containment atmosphere. OR B.1.2 ----------NOTE---------- Not required until 12 hours after establishment of steady state operation. ---------- Perform SR 3.4.13.1. AND B.2 Restore required containment atmosphere particulate radioactivity monitor to OPERABLE status.	Once per 24 hours Once per 24 hours 30 days
C. Required Action and associated Completion Time not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 5.	6 hours 36 hours
D. All required monitors inoperable.	D.1 Enter LCO 3.0.3.	Immediately

3.7 PLANT SYSTEMS

3.7.4 Atmospheric Dump Valves (ADVs)

LCO 3.7.4 Four ADV lines shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3,
MODE 4 when steam generator is relied upon for heat removal.

ACTIONS

CONDITION		REQUIRED ACTION		COMPLETION TIME
A.	One required ADV line inoperable.	A.1	Restore required ADV line to OPERABLE status.	7 days OR In accordance with the Risk Informed Completion Time Program
B.	One train (two ADV lines) inoperable due to one train of ACAS inoperable.	B.1	Restore ADV lines to OPERABLE status.	72 hours
C.	Two or more required ADV lines inoperable for reasons other than Condition B.	C.1	Restore all but one ADV line to OPERABLE status.	24 hours
D.	Required Action and associated Completion Time not met.	D.1	Be in MODE 3.	6 hours
		AND		
		D.2	Be in MODE 4 without reliance upon steam generator for heat removal.	24 hours

3.9 REFUELING OPERATIONS

3.9.1 Boron Concentration

LCO 3.9.1 Boron concentrations of the Reactor Coolant System, the refueling canal, and the refueling cavity shall be maintained within the limit specified in the COLR.

APPLICABILITY: MODE 6.

----------NOTE----------
Only applicable to the refueling canal and refueling cavity when connected to the RCS.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Boron concentration not within limit.	A.1 Suspend CORE ALTERATIONS. AND A.2 Suspend positive reactivity additions. AND A.3 Initiate action to restore boron concentration to within limit.	Immediately Immediately Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.9.1.1	Verify boron concentration is within the limit specified in COLR.	In accordance with the Surveillance Frequency Control Program

3.9 REFUELING OPERATIONS

3.9.3 Nuclear Instrumentation

LCO 3.9.3 Two source range neutron flux monitors shall be OPERABLE.

APPLICABILITY: MODE 6.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One required source range neutron flux monitor inoperable.	A.1 Suspend CORE ALTERATIONS. AND	Immediately
	A.2 Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet the boron concentration of LCO 3.9.1.	Immediately
B. Two required source range neutron flux monitors inoperable.	B.1 Initiate action to restore one source range neutron flux monitor to OPERABLE status. AND	Immediately
	B.2 Perform SR 3.9.1.1.	Once per 12 hours

3.9 REFUELING OPERATIONS

3.9.5 Residual Heat Removal (RHR) and Coolant Circulation - High Water Level

LCO 3.9.5 One RHR loop shall be OPERABLE and in operation.

---NOTE---
The required RHR loop may be removed from operation for ≤ 1 hour per 8 hour period, provided no operations are permitted that would cause introduction of coolant into the Reactor Coolant System with boron concentration less than that required to meet the minimum required boron concentration of LCO 3.9.1.

APPLICABILITY: MODE 6 with the water level ≥ 23 ft above the top of reactor vessel flange.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. RHR loop requirements not met.	A.1	Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet the boron concentration of LCO 3.9.1.	Immediately
	AND		
	A.2	Suspend loading irradiated fuel assemblies in the core.	Immediately
	AND		
	A.3	Initiate action to satisfy RHR loop requirements.	Immediately
	AND		
			(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. (continued)	A.4 Close equipment hatch and secure with four bolts. AND	4 hours
	A.5 Close one door in each air lock. AND	4 hours
	A.6.1 Close each penetration providing direct access from the containment atmosphere to the outside atmosphere with a manual or automatic isolation valve, blind flange, or equivalent. OR	4 hours
	A.6.2 Verify each penetration is capable of being closed by an OPERABLE automatic Containment Ventilation isolation valve.	4 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.9.5.1	Verify one RHR loop is in operation and circulating reactor coolant at a flow rate of $\geq$ 2000 gpm.	In accordance with the Surveillance Frequency Control Program

3.9 REFUELING OPERATIONS

3.9.6 Residual Heat Removal (RHR) and Coolant Circulation - Low Water Level

LCO 3.9.6 Two RHR loops shall be OPERABLE, and one RHR loop shall be in operation.

----------NOTES----------

1. All RHR pumps may be removed from operation for ≤ 15 minutes when switching from one train to another provided:
   a. The core outlet temperature is maintained > 10 degrees F below saturation temperature;
   b. No operations are permitted that would cause introduction of coolant into the Reactor Coolant System (RCS) with boron concentration less than that required to meet the minimum required boron concentration of LCO 3.9.1; and
   c. No draining operations to further reduce RCS water volume are permitted.
2. One required RHR loop may be inoperable for up to 2 hours for surveillance testing, provided that the other RHR loop is OPERABLE and in operation.

APPLICABILITY: MODE 6 with the water level < 23 ft above the top of reactor vessel flange.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Less than the required number of RHR loops OPERABLE.	A.1 Initiate action to restore required RHR loops to OPERABLE status. OR A.2 Initiate action to establish ≥ 23 ft of water above the top of reactor vessel flange.	Immediately Immediately

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. No RHR loop in operation.	B.1 Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet the boron concentration of LCO 3.9.1.	Immediately
	AND	
	B.2 Initiate action to restore one RHR loop to operation.	Immediately
	AND	
	B.3 Close equipment hatch and secure with four bolts.	4 hours
	AND	
	B.4 Close one door in each air lock.	4 hours
	AND	
	B.5.1 Close each penetration providing direct access from the containment atmosphere to the outside atmosphere with a manual or automatic isolation valve, blind flange, or equivalent.	4 hours
	OR	
	B.5.2 Verify each penetration is capable of being closed by an OPERABLE automatic Containment Ventilation isolation valve.	4 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.9.6.1	Verify one RHR loop is in operation and circulating reactor coolant at a flow rate of $\geq$ 2000 gpm.	In accordance with the Surveillance Frequency Control Program
SR 3.9.6.2	Verify correct breaker alignment and indicated power available to the required RHR pump that is not in operation.	In accordance with the Surveillance Frequency Control Program

3.9 REFUELING OPERATIONS

3.9.7 Refueling Cavity Water Level

LCO 3.9.7 Refueling cavity water level shall be maintained ≥ 23 ft above the top of reactor vessel flange.

APPLICABILITY: During movement of irradiated fuel assemblies within containment.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Refueling cavity water level not within limit.	A.1 Suspend movement of irradiated fuel assemblies within containment.	Immediately

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

TENNESSEE VALLEY AUTHORITY

DOCKET NO. 50-391

WATTS BAR NUCLEAR PLANT, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 86
License No. NPF-96

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Tennessee Valley Authority (TVA, the licensee) dated August 25, 2025, as supplemented by letter dated February 24, 2026, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-96 is hereby amended to read as follows:

   (2) Technical Specifications and Environmental Protection Plan

   The Technical Specifications contained in Appendix A as revised through Amendment No. 86 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 180 days.

FOR THE NUCLEAR REGULATORY COMMISSION

MICHAEL MAHONEY

Digitally signed by MICHAEL MAHONEY
Date: 2026.06.25 14:25:33 -04'00'

Michael Mahoney, Acting Chief
Operating Reactor Licensing Branch 4
Division of Licensing Projects I
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Operating License
and Technical Specifications

Date of Issuance: June 25, 2026

ATTACHMENT TO AMENDMENT NO. 86

WATTS BAR NUCLEAR PLANT, UNIT 2

FACILITY OPERATING LICENSE NO. NPF-96

DOCKET NO. 50-391

Replace page 3 of Facility Operating License No. NPF-96 with the attached revised page 3. The revised page is identified by amendment number and contains a marginal line indicating the area of change.

Replace the following pages of the Appendix A, Technical Specifications, with the attached pages. The revised pages are identified by amendment number and contain a marginal line indicating the area of change.

Remove Page	Insert Page	Remove Page	Insert Page
1.1-5	1.1-5	3.4-15	3.4-15
1.4-1	1.4-1	3.4-16	3.4-16
1.4-2	1.4-2	3.4-17	3.4-17
1.4-3	1.4-3	3.4-20	3.4-20
1.4-4	1.4-4	3.4-21	3.4-21
--	1.4-5	3.4-22	3.4-22
--	1.4-6	--	3.4-22a
--	1.4-7	3.4-23	3.4-23
--	1.4-8	3.4-25	3.4-25
3.4-3	3.4-3	3.4-27	3.4-27
3.4-7	3.4-7	3.4-33	3.4-33
3.4-8	3.4-8	3.4-34	3.4-34
--	3.4-8a	3.7-8	3.7-8
3.4-9	3.4-9	3.9-1	3.9-1
--	3.4-9a	3.9-3	3.9-3
3.4-10	3.4-10	3.9-6	3.9-6
3.4-11	3.4-11	3.9-7	3.9-7
3.4-12	3.4-12	3.9-8	3.9-8
3.4-13	3.4-13	3.9-9	3.9-9
--	3.4-13a	--	3.9-9a
3.4-14	3.4-14	3.9-10	3.9-10

C. The license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act, and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified or incorporated below.

(1) Maximum Power Level

TVA is authorized to operate the facility at reactor core power levels not in excess of 3459 megawatts thermal.

(2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A as revised through Amendment No. 86 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. TVA shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) TVA shall implement permanent modifications to prevent overtopping of the embankments of the Fort Loudon Dam due to the Probable Maximum Flood by June 30, 2018.

(4) FULL SPECTRUM LOCA Methodology shall be implemented when the WBN Unit 2 steam generators are replaced with steam generators equivalent to the existing steam generators at WBN Unit 1.

(5) By December 31, 2019, the licensee shall report to the NRC that the actions to resolve the issues identified in Bulletin 2012-01, "Design Vulnerability in Electrical Power System," have been implemented.

(6) The licensee shall maintain in effect the provisions of the physical security plan, security personnel training and qualification plan, and safeguards contingency plan, and all amendments made pursuant to the authority of 10 CFR 50.90 and 50.54(p).

(7) TVA shall fully implement and maintain in effect all provisions of the Commission approved cyber security plan (CSP), including changes made pursuant to the authority of 10 CFR 50.90 and 10 CFR 50.54(p). The TVA approved CSP was discussed in NUREG-0847, Supplement 28, as amended by changes approved in License Amendment No. 7.

(8) TVA shall implement and maintain in effect all provisions of the approved fire protection program as described in the Fire Protection Report for the facility, as described in NUREG-0847, Supplement 29, subject to the following provision:

1.1 Definitions (continued)

OPERABLE-OPERABILITY	A system, subsystem, train, component, or device shall be OPERABLE or have OPERABILITY when it is capable of performing its specified safety function(s) and when all necessary attendant instrumentation, controls, normal or emergency electrical power, cooling and seal water, lubrication, and other auxiliary equipment that are required for the system, subsystem, train, component, or device to perform its specified safety function(s) are also capable of performing their related support function(s).
PDMS	The Power Distribution Monitoring System (PDMS) is a real-time three dimensional core monitoring system. The system utilizes existing core instrumentation data and an on-line neutronics code to provide surveillance of core thermal limits.
PHYSICS TESTS	PHYSICS TESTS shall be those tests performed to measure the fundamental nuclear characteristics of the reactor core and related instrumentation. These tests are: a. Described in Chapter 14, Initial Test Program of the FSAR; b. Authorized under the provisions of 10 CFR 50.59; or c. Otherwise approved by the Nuclear Regulatory Commission.
PRESSURE AND TEMPERATURE LIMITS REPORT (PTLR)	The PTLR is the unit specific document that provides the RCS pressure and temperature limits for heatup, cooldown, low temperature operation, criticality, and hydrostatic testing as well as heatup and cooldown rates and the cold overpressure mitigation system for the current reactor vessel fluence period. These pressure and temperature limits shall be determined for each fluence period in accordance with Specification 5.9.6. Plant operation within these operating limits is addressed in LCO 3.4.3, "RCS Pressure and Temperature (P/T) Limits," and LCO 3.4.12, "Cold Overpressure Mitigation System (COMS)."

1.0 USE AND APPLICATION

1.4 Frequency

PURPOSE

The purpose of this section is to define the proper use and application of Frequency requirements.

DESCRIPTION

Each Surveillance Requirement (SR) has a specified Frequency in which the Surveillance must be met in order to meet the associated LCO. An understanding of the correct application of the specified Frequency is necessary for compliance with the SR.

The "specified Frequency" is referred to throughout this section and each of the Specifications of Section 3.0, Surveillance Requirement (SR) Applicability. The "specified Frequency" consists of the requirements of the Frequency column of each SR as well as certain Notes in the Surveillance column that modify performance requirements.

Sometimes special situations dictate when the requirements of a Surveillance are to be met. They are "otherwise stated" conditions allowed by SR 3.0.1. They may be stated as clarifying Notes in the Surveillance, as part of the Surveillance or both.

Situations where a Surveillance could be required (i.e., its Frequency could expire), but where it is not possible or not desired that it be performed until sometime after the associated LCO is within its Applicability, represent potential SR 3.0.4 conflicts. To avoid these conflicts, the SR (i.e., the Surveillance or the Frequency) is stated such that it is only "required" when it can be and should be performed. With an SR satisfied, SR 3.0.4 imposes no restriction.

The use of "met" or "performed" in these instances conveys specific meanings. A Surveillance is "met" only when the acceptance criteria are satisfied. Known failure of the requirements of a Surveillance, even without a Surveillance specifically being "performed," constitutes a Surveillance not "met." "Performance" refers only to the requirement to specifically determine the ability to meet the acceptance criteria.

Some Surveillances contain notes that modify the Frequency of performance or the conditions during which the acceptance criteria must be satisfied. For these Surveillances, the MODE-entry restrictions of SR 3.0.4 may not apply. Such a Surveillance is not required to be performed prior to entering a MODE or other specified condition in the Applicability of the associated LCO if any of the following three

(continued)

1.4 Frequency

DESCRIPTION (continued)

conditions are satisfied:

a. The Surveillance is not required to be met in the MODE or other specified condition to be entered, or

b. The Surveillance is required to be met in the MODE or other specified condition to be entered, but has been performed within the specified Frequency (i.e., it is current) and is known not to be failed, or

c. The Surveillance is required to be met, but not performed, in the MODE or other specified condition to be entered, and is known not to be failed.

Examples 1.4-3, 1.4-4, 1.4-5, and 1.4-6 discuss these special situations.

EXAMPLES

The following examples illustrate the various ways that Frequencies are specified. In these examples, the Applicability of the LCO (LCO not shown) is MODES 1, 2, and 3.

EXAMPLE 1.4-1

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
Perform CHANNEL CHECK.	12 hours

Example 1.4-1 contains the type of SR most often encountered in the Technical Specifications (TS). The Frequency specifies an interval (12 hours) during which the associated Surveillance must be performed at least one time. Performance of the Surveillance initiates the subsequent interval. Although the Frequency is stated as 12 hours, an extension of the time interval to 1.25 times the interval specified in the Frequency is allowed by SR 3.0.2 for operational flexibility. The measurement of this interval continues at all times, even when the SR is not required to be met per SR 3.0.1 (such as when the equipment is inoperable, a variable is outside specified limits, or the unit is outside the Applicability of the LCO). If the interval specified by SR 3.0.2 is exceeded while the unit is in a MODE or other specified condition in the Applicability of the LCO, and the performance of the Surveillance is not otherwise modified (refer to Example 1.4-3), then SR 3.0.3 becomes applicable.

If the interval as specified by SR 3.0.2 is exceeded while the unit is not in a MODE or other specified condition in the Applicability of the LCO for which performance of the SR is required, the Surveillance must be performed within the Frequency requirements of SR 3.0.2 prior to entry into the MODE or other specified condition. Failure to do so would result in a violation of SR 3.0.4

EXAMPLES (continued)

EXAMPLE 1.4-2

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
Verify flow is within limits.	Once within 12 hours after ≥ 25% RTP AND 24 hours thereafter

Example 1.4-2 has two Frequencies. The first is a one time performance Frequency, and the second is of the type shown in Example 1.4-1. The logical connector "AND" indicates that both Frequency requirements must be met. Each time reactor power is increased from a power level < 25% RTP to ≥ 25% RTP, the Surveillance must be performed within 12 hours.

The use of "once" indicates a single performance will satisfy the specified Frequency (assuming no other Frequencies are connected by "AND"). This type of Frequency does not qualify for the extension allowed by SR 3.0.2. "Thereafter" indicates future performances must be established per SR 3.0.2, but only after a specified condition is first met (i.e., the "once" performance in this example). If reactor power decreases to < 25% RTP, the measurement of both intervals stops. New intervals start upon reactor power reaching 25% RTP.

(continued)

EXAMPLES (continued)

EXAMPLE 1.4-3

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
----------NOTE---------- Not required to be performed until 12 hours after ≥ 25% RTP. ----------	
Perform channel adjustment.	7 days

The interval continues, whether or not the unit operation is < 25% RTP between performances.

As the Note modifies the required performance of the Surveillance, it is construed to be part of the "specified Frequency." Should the 7 day interval be exceeded while operation is < 25% RTP, this Note allows 12 hours after power reaches ≥ 25% RTP to perform the Surveillance. The Surveillance is still considered to be performed within the "specified Frequency." Therefore, if the Surveillance was not performed within the 7 day interval (plus the extension allowed by SR 3.0.2), but operation was < 25% RTP, it would not constitute a failure of the SR or failure to meet the LCO. Also, no violation of SR 3.0.4 occurs when changing MODES, even with the 7 day Frequency not met, provided operation does not exceed 12 hours with power ≥ 25% RTP.

Once the unit reaches 25% RTP, 12 hours would be allowed for completing the Surveillance. If the Surveillance was not performed within this 12 hour interval, there would then be a failure to perform a Surveillance within the specified Frequency and the provisions of SR 3.0.3 would apply.

EXAMPLES (continued)

EXAMPLE 1.4-4

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
----------NOTE---------- Only required to be met in MODE 1. ----------	
Verify leakage rates are within limits.	24 hours

Example 1.4-4 specifies that the requirements of this Surveillance do not have to be met until the unit is in MODE 1. The interval measurement for the Frequency of this Surveillance continues at all times, as described in Example 1.4-1. However, the Note constitutes an "otherwise stated" exception to the Applicability of this Surveillance. Therefore, if the Surveillance were not performed within the 24 hour interval (plus the extension allowed by SR 3.0.2), but the unit was not in MODE 1, there would be no failure of the SR nor failure to meet the LCO. Therefore, no violation of SR 3.0.4 occurs when changing MODES, even with the 24 hour Frequency exceeded, provided the MODE change was not made into MODE 1. Prior to entering MODE 1 (assuming again that the 24 hour Frequency were not met), SR 3.0.4 would require satisfying the SR.

EXAMPLES (continued)

EXAMPLE 1.4-5

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
----------NOTE---------- Only required to be performed in MODE 1. ----------	
Perform complete cycle of the valve.	7 days

The interval continues, whether or not the unit operation is in MODE 1, 2, or 3 (the assumed Applicability of the associated LCO) between performances.

As the Note modifies the required performance of the Surveillance, the Note is construed to be part of the "specified Frequency." Should the 7 day interval be exceeded while operation is not in MODE 1, this Note allows entry into and operation in MODES 2 and 3 to perform the Surveillance. The Surveillance is still considered to be performed within the "specified Frequency" if completed prior to entering MODE 1. Therefore, if the Surveillance were not performed within the 7 day (plus the extension allowed by SR 3.0.2) interval, but operation was not in MODE 1, it would not constitute a failure of the SR or failure to meet the LCO. Also, no violation of SR 3.0.4 occurs when changing MODES, even with the 7 day Frequency not met, provided operation does not result in entry into MODE 1.

Once the unit reaches MODE 1, the requirement for the Surveillance to be performed within its specified Frequency applies and would require that the Surveillance had been performed. If the Surveillance were not performed prior to entering MODE 1, there would then be a failure to perform a Surveillance within the specified Frequency, and the provisions of SR 3.0.3 would apply.

EXAMPLES (continued)

EXAMPLE 1.4-6

SURVEILLANCE REQUIREMENTS

SURVEILLANCE	FREQUENCY
----------NOTE---------- Not required to be met in MODE 3. ----------	
Verify parameter is within limits.	24 hours

Example 1.4-6 specifies that the requirements of this Surveillance do not have to be met while the unit is in MODE 3 (the assumed Applicability of the associated LCO is MODES 1, 2, and 3). The interval measurement for the Frequency of this Surveillance continues at all times, as described in Example 1.4-1. However, the Note constitutes an "otherwise stated" exception to the Applicability of this Surveillance. Therefore, if the Surveillance were not performed within the 24 hour interval (plus the extension allowed by SR 3.0.2), and the unit was in MODE 3, there would be no failure of the SR nor failure to meet the LCO. Therefore, no violation of SR 3.0.4 occurs when changing MODES to enter MODE 3, even with the 24 hour Frequency exceeded, provided the MODE change does not result in entry into MODE 2. Prior to entering MODE 2 (assuming again that the 24 hour Frequency was not met), SR 3.0.4 would require satisfying the SR.

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.2 RCS Minimum Temperature for Criticality

LCO 3.4.2 Each RCS loop average temperature (T_{avg}) shall be $\geq$ 551°F.

APPLICABILITY: MODE 1,
MODE 2 with $k_{eff} \geq 1.0$.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. T_{avg} in one or more RCS loops not within limit.	A.1 Be in MODE 2 with k_{eff} < 1.0.	30 minutes

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.2.1	Verify RCS T_{avg} in each loop $\geq$ 551°F.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.5 RCS Loops - MODE 3

LCO 3.4.5 Two RCS loops shall be OPERABLE, and either:

a. Two RCS loops shall be in operation when the Rod Control System is capable of rod withdrawal; or

b. One RCS loop shall be in operation when the Rod Control System is not capable of rod withdrawal.

----------NOTE----------
All reactor coolant pumps may be removed from operation for ≤ 1 hour per 8 hour period provided:

a. No operations are permitted that would cause introduction of coolant into the RCS with boron concentration less than required to meet the SDM of LCO 3.1.1; and

b. Core outlet temperature is maintained at least 10°F below saturation temperature.

APPLICABILITY: MODE 3.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. One required RCS loop inoperable.	A.1	Restore required RCS loop to OPERABLE status.	72 hours
B. Required Action and associated Completion Time of Condition A not met.	B.1	Be in MODE 4.	12 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
C. One required RCS loop not in operation with Rod Control System capable of rod withdrawal.	C.1	Restore required RCS loop to operation.	1 hour
	OR		
	C.2	Place the Rod Control System in a condition incapable of rod withdrawal.	1 hour
D. Two required RCS loops inoperable. OR Required RCS loop(s) not in operation.	D.1	Place the Rod Control System in a condition incapable of rod withdrawal.	Immediately
	AND		
	D.2	Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet SDM of LCO 3.1.1.	Immediately
	AND		
	D.3	Initiate action to restore one RCS loop to OPERABLE status and operation.	Immediately

SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY
SR 3.4.5.1	Verify required RCS loops are in operation	In accordance with the Surveillance Frequency Control Program
SR 3.4.5.2	Verify steam generator secondary side water levels are great than or equal to 32% narrow range for required RCS loops.	In accordance with the Surveillance Frequency Control Program
SR 3.4.5.3	----------NOTE---------- Not required to be performed until 24 hours after a required pump is not in operation. ---------- Verify correct breaker alignment and indicated power are available to each required pump.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.6 RCS Loops - MODE 4

LCO 3.4.6 Two loops shall be OPERABLE, and consist of either:

a. Any combination of RCS loops and residual heat removal (RHR) loops, and one loop shall be in operation, when the rod control system is not capable of rod withdrawal; or

b. Two RCS loops, and both loops shall be in operation, when the rod control system is capable of rod withdrawal.

----------NOTES----------

1. No RCP shall be started with any RCS cold leg temperature less than or equal to the COMS arming temperature specified in the PTLR unless the secondary side water temperature of each steam generator (SG) is $\leq$ 50°F above each of the RCS cold leg temperatures.

2. For the initial 7 hours after entry into MODE 3 from MODE 1 or MODE 2, two loops shall consist of:

   a. Two RCS loops with one loop in operation when the rod control system is not capable of rod withdrawal; or

   b. Two RCS loops with both loops in operation when the rod control system is capable of rod withdrawal.

3. Average reactor coolant temperature shall be maintained > 200ºF for the initial 7 hours after entry into MODE 3 from MODE 1 or MODE 2.

APPLICABILITY: MODE 4.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. One required loop inoperable AND Less than 7 hours since entry into MODE 3 from MODE 1 or MODE 2.	A.1	Initiate action to restore a second required RCS or RHR loop to OPERABLE status.	Immediately
	AND		
	A.2	----------NOTE---------- Only required if RHR loop is OPERABLE. ----------	
		Be in MODE 5.	24 hours
B. One required RCS loop not in operation with Rod Control System capable of rod withdrawal.	B.1	Restore required RCS loop to operation.	1 hour
	OR		
	B.2	Place the Rod Control System in a condition incapable of rod withdrawal.	1 hour

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
C. Two required loops inoperable. OR Required loop not in operation.	C.1	Place the Rod Control System in a condition incapable of rod withdrawal.	Immediately
	AND		
	C.2	Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet SDM of LCO 3.1.1.	Immediately
	AND		
	C.3	Initiate action to restore one required loop to OPERABLE status and operation.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.6.1	Verify two RCS loops are in operation when the rod control system is capable of rod withdrawal.	In accordance with the Surveillance Frequency Control Program
SR 3.4.6.2	Verify required RHR or RCS loop is in operation when the rod control system is not capable of rod withdrawal.	In accordance with the Surveillance Frequency Control Program
SR 3.4.6.3	Verify SG secondary side water levels are greater than or equal to 32% narrow range for required RCS loops.	In accordance with the Surveillance Frequency Control Program
SR 3.4.6.4	----------NOTE---------- Not required to be performed until 24 hours after a required pump is not in operation. ---------- Verify correct breaker alignment and indicated power are available to each required RHR pump.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.7 RCS Loops - MODE 5, Loops Filled

LCO 3.4.7 One residual heat removal (RHR) loop shall be OPERABLE and in operation, and either:

a. One additional RHR loop shall be OPERABLE; or

b. The secondary side water level of at least two steam generators (SGs) shall be greater than or equal to 32% narrow range.

----------NOTES----------

1. One required RHR loop may be inoperable for up to 2 hours for surveillance testing provided that the other RHR loop is OPERABLE and in operation.

2. No reactor coolant pump shall be started with one or more RCS cold leg temperatures less than or equal to the COMS arming temperature specified in the PTLR unless the secondary side water temperature of each SG is $\leq$ 50°F above each of the RCS cold leg temperatures.

3. All RHR loops may be removed from operation during planned heatup to MODE 4 when at least one RCS loop is in operation.

APPLICABILITY: MODE 5 with RCS loops filled.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. One required RHR loop inoperable. AND One RHR loop OPERABLE.	A.1	Initiate action to restore a second RHR loop to OPERABLE status. OR	Immediately
	A.2	Initiate action to restore required SGs secondary side water levels to within limits.	Immediately

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
B. One or more required SGs with secondary side water level not within limit. AND One RHR loop OPERABLE.	B.1	Initiate action to restore one RHR loop to OPERABLE status.	Immediately
	AND		
	B.2	Initiate action to restore required SGs secondary side water level to within limit.	Immediately
C. No required RHR loops OPERABLE. OR Required RHR loop not in operation.	C.1	Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet SDM of LCO 3.1.1.	Immediately
	AND		
	C.2	Initiate action to restore one RHR loop to OPERABLE status and operation.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.7.1	Verify required RHR loop is in operation.	In accordance with the Surveillance Frequency Control Program

(continued)

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.4.7.2	Verify SG secondary side water level is greater than or equal to 32% narrow range in required SGs.	In accordance with the Surveillance Frequency Control Program
SR 3.4.7.3	----------NOTE---------- Not required to be performed until 24 hours after a required pump is not in operation. ---------- Verify correct breaker alignment and indicated power are available to each required RHR pump.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.8 RCS Loops - MODE 5, Loops Not Filled

LCO 3.4.8 Two residual heat removal (RHR) loops shall be OPERABLE and one RHR loop shall be in operation.

----------NOTE----------

1. All RHR pumps may be removed from operation for ≤ 15 minutes when switching from one loop to another provided:

   a. The core outlet temperature is maintained > 10°F below saturation temperature.

   b. No operations are permitted that would cause introduction of coolant into the RCS with boron concentration less than required to meet the SDM of LCO 3.1.1; and

   c. No draining operations to further reduce the RCS water volume are permitted.

2. One RHR loop may be inoperable for ≤ 2 hours for surveillance testing provided that the other RHR loop is OPERABLE and in operation.

APPLICABILITY: MODE 5 with RCS loops not filled.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. One required RHR loop inoperable.	A.1 Initiate action to restore RHR loop to OPERABLE status.	Immediately

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
B. No required RHR loops OPERABLE. OR Required RHR loop not in operation.	B.1	Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet SDM of LCO 3.1.1.	Immediately
	AND		
	B.2	Initiate action to restore one RHR loop to OPERABLE status and operation.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.8.1	Verify required RHR loop is in operation.	In accordance with the Surveillance Frequency Control Program
SR 3.4.8.2	----------NOTE---------- Not required to be performed until 24 hours after a required pump is not in operation. ---------- Verify correct breaker alignment and indicated power are available to each required RHR pump.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.9 Pressurizer

LCO 3.4.9 The pressurizer shall be OPERABLE with:

a. Pressurizer water level ≤ 92%; and

b. Two groups of pressurizer heaters OPERABLE with the capacity of each group ≥ 150 kW

APPLICABILITY: MODES 1, 2, and 3.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. Pressurizer water level not within limit.	A.1	Be in MODE 3.	6 hours
	AND		
	A.2	Fully insert all rods.	6 hours
	AND		
	A.3	Place Rod Control System in a condition incapable of rod withdrawal.	6 hours
	AND		
	A.4	Be in MODE 4.	12 hours
B. One required group of pressurizer heaters inoperable.	B.1	Restore required group of pressurizer heaters to OPERABLE status.	72 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
C. Required Action and associated Completion Time of Condition B not met.	C.1 Be in MODE 3. AND C.2 Be in MODE 4.	6 hours 12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.9.1	Verify pressurizer water level is $\leq$ 92%.	In accordance with the Surveillance Frequency Control Program
SR 3.4.9.2	Verify capacity of each required group of pressurizer heaters is $\geq$ 150 kW.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.11 Pressurizer Power Operated Relief Valves (PORVs)

LCO 3.4.11 Each PORV and associated block valve shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3.

ACTIONS

----------NOTE----------
Separate Condition entry is allowed for each PORV and each block valve.

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. One or more PORVs inoperable and capable of being manually cycled.	A.1	Close and maintain power to associated block valve.	1 hour
B. One PORV inoperable and not capable of being manually cycled.	B.1	Close associated block valve.	1 hour
	AND		
	B.2	Remove power from associated block valve.	1 hour
	AND		
	B.3	Restore PORV to OPERABLE status.	72 hours OR In accordance with the Risk Informed Completion Time Program

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
C. One block valve inoperable.	----------NOTE---------- Required Actions C.1 and C.2 do not apply when block valve is inoperable solely as a result of complying with Required Actions B.2 or E.2. ----------		
	C.1	Place associated PORV in manual control.	1 hour
	AND		
	C.2	Restore block valve to OPERABLE status.	72 hours OR In accordance with the Risk Informed Completion Time Program
D. Required Action and associated Completion Time of Condition A, B, or C not met.	D.1	Be in MODE 3.	6 hours
	AND		
	D.2	Be in MODE 4.	12 hours
E. Two PORVs inoperable and not capable of being manually cycled.	E.1	Close associated block valves.	1 hour
	AND		
	E.2	Remove power from associated block valves.	1 hour
	AND		
	E.3	Be in MODE 3.	6 hours
	AND		
	E.4	Be in MODE 4.	12 hours

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
F. Two block valves inoperable.	F.1	----------NOTE---------- Required Action F.1 does not apply when block valve is inoperable solely as a result of complying with Required Actions B.2 or E.2 ---------- Restore one block valve to OPERABLE status.	2 hours
G. Required Action and associated Completion Time of Condition F not met.	G.1 AND G.2	Be in MODE 3. Be in MODE 4.	6 hours 12 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.11.1	----------NOTES---------- 1. Not required to be performed with block valve closed in accordance with the Required Actions of this LCO. 2. Only required to be performed in Modes 1 and 2. ---------- Perform a complete cycle of each block valve.	In accordance with the Surveillance Frequency Control Program
SR 3.4.11.2	----------NOTE---------- Only required to be performed in Modes 1 and 2. ---------- Perform a complete cycle of each PORV.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.12 Cold Overpressure Mitigation System (COMS)

LCO 3.4.12 A COMS System shall be OPERABLE with a maximum of one charging pump and no safety injection pump capable of injecting into the RCS and the accumulators isolated and one of the following pressure relief capabilities:

a. Two power operated relief valves (PORVs) with lift settings within the limits specified in the PTLR, or

b. One PORV with a lift setting within the limits specified in the PTLR and the RHR suction relief valve with a setpoint $\geq$ 436.5 psig and $\leq$ 463.5 psig, or

c. The RCS depressurized and an RCS vent capable of relieving > 475 gpm water flow.

----------NOTES----------

1. Two charging pumps may be made capable of injecting for less than or equal to one hour for pump swap operations.

2. Accumulator may be unisolated when accumulator pressure is less than the maximum RCS pressure for the existing RCS cold leg temperature allowed by the P/T limit curves provided in the PTLR.

3. One safety injection pump and one charging pump may be capable of injecting into the RCS for the purpose of testing in MODE 5 or MODE 6 when the reactor vessel head is on, provided the pressurizer manway cover is removed to provide a vent path for adequate pressure relief.

APPLICABILITY: MODE 4 with any RCS cold leg temperature $\leq$ COMS arming temperature specified in the PTLR,
MODE 5,
MODE 6 when the reactor vessel head is on.

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
E. One required RCS relief valve inoperable in MODE 4.	E.1 Restore required RCS relief valve to OPERABLE status.	7 days
F. One required RCS relief valve inoperable in MODE 5 or 6.	F.1 Restore required RCS relief valve to OPERABLE status.	24 hours
G. Two required RCS relief valves inoperable. OR Required Action and associated Completion Time of Condition A, B, D, E, or F not met. OR COMS inoperable for any reason other than Condition A, B, C, D, E, or F.	G.1 Depressurize RCS and establish RCS vent.	12 hours

SURVEILLANCE REQUIREMENTS (continued)

SURVEILLANCE		FREQUENCY
SR 3.4.12.4	Verify required RCS vent open.	In accordance with the Surveillance Frequency Control Program
SR 3.4.12.5	Verify PORV block valve is open for each required PORV.	In accordance with the Surveillance Frequency Control Program
SR 3.4.12.6	Verify both RHR suction isolation valves are locked open with operator power removed for the required RHR suction relief valve.	In accordance with the Surveillance Frequency Control Program
SR 3.4.12.7	----------NOTE---------- Not required to be performed until 12 hours after decreasing RCS cold leg temperature to less than or equal to the COMS arming temperature specified in the PTLR. ---------- Perform a COT on each required PORV, excluding actuation.	In accordance with the Surveillance Frequency Control Program
SR 3.4.12.8	Perform CHANNEL CALIBRATION for each required PORV actuation channel.	In accordance with the Surveillance Frequency Control Program

3.4 REACTOR COOLANT SYSTEM (RCS)

3.4.15 RCS Leakage Detection Instrumentation

LCO 3.4.15 The following RCS leakage detection instrumentation shall be OPERABLE:

a. One containment pocket sump level monitor; and

b. One lower containment atmosphere particulate radioactivity monitor.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. Required containment pocket sump level monitor inoperable.	A.1	----------NOTE---------- Not required until 12 hours after establishment of steady state operation. ---------- Perform SR 3.4.13.1.	Once per 24 hours
	AND		
	A.2	Restore required containment pocket sump level monitor to OPERABLE status.	30 days
B. Required containment atmosphere particulate radioactivity monitor inoperable.	B.1.1	Analyze grab samples of the containment atmosphere.	Once per 24 hours
		OR	
	B.1.2	----------NOTE---------- Not required until 12 hours after establishment of steady state operation. ---------- Perform SR 3.4.13.1.	Once per 24 hours
	AND		(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
B. (continued)	B.2	Restore required containment atmosphere particulate radioactivity monitor to OPERABLE status.	30 days
C. Required Action and associated Completion Time not met.	C.1 AND C.2	Be in MODE 3. Be in MODE 5.	6 hours 36 hours
D. All required monitors inoperable.	D.1	Enter LCO 3.0.3.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.4.15.1	Perform CHANNEL CHECK of the required containment atmosphere particulate radioactivity monitor.	In accordance with the Surveillance Frequency Control Program
SR 3.4.15.2	Perform COT of the required containment atmosphere particulate radioactivity monitor.	In accordance with the Surveillance Frequency Control Program
SR 3.4.15.3	Perform CHANNEL CALIBRATION of the required containment pocket sump level monitor.	In accordance with the Surveillance Frequency Control Program
SR 3.4.15.4	Perform CHANNEL CALIBRATION of the required containment atmosphere particulate radioactivity monitor.	In accordance with the Surveillance Frequency Control Program

3.7 PLANT SYSTEMS

3.7.4 Atmospheric Dump Valves (ADVs)

LCO 3.7.4 Four ADV lines shall be OPERABLE.

APPLICABILITY: MODES 1, 2, and 3,
MODE 4 when steam generator is relied upon for heat removal.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. One required ADV line inoperable.	A.1	Restore required ADV line to OPERABLE status.	7 days OR In accordance with the Risk Informed Completion Time Program
B. One train (two ADV lines) inoperable due to one train of ACAS inoperable.	B.1	Restore ADV lines to OPERABLE status.	72 hours
C. Two or more required ADV lines inoperable for reasons other than Condition B.	C.1	Restore all but one ADV line to OPERABLE status.	24 hours
D. Required Action and associated Completion Time not met.	D.1 AND D.2	Be in MODE 3. Be in MODE 4 without reliance upon steam generator for heat removal.	6 hours 24 hours

3.9 REFUELING OPERATIONS

3.9.1 Boron Concentration

LCO 3.9.1 Boron concentrations of the Reactor Coolant System, the refueling canal, and the refueling cavity shall be maintained within the limit specified in the COLR.

APPLICABILITY: MODE 6.

----------NOTE----------
Only applicable to the refueling canal and refueling cavity when connected to the RCS.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. Boron concentration not within limit.	A.1	Suspend CORE ALTERATIONS.	Immediately
	AND		
	A.2	Suspend positive reactivity additions.	Immediately
	AND		
	A.3	Initiate action to restore boron concentration to within limit.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.9.1.1	Verify boron concentration is within the limit specified in COLR.	In accordance with the Surveillance Frequency Control Program

3.9 REFUELING OPERATIONS

3.9.3 Nuclear Instrumentation

LCO 3.9.3 Two source range neutron flux monitors shall be OPERABLE.

APPLICABILITY: MODE 6.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. One required source range neutron flux monitor inoperable.	A.1	Suspend CORE ALTERATIONS.	Immediately
	AND		
	A.2	Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet the boron concentration of LCO 3.9.1.	Immediately
B. Two required source range neutron flux monitors inoperable.	B.1	Initiate action to restore one source range neutron flux monitor to OPERABLE status.	Immediately
	AND		
	B.2	Perform SR 3.9.1.1.	Once per 12 hours

3.9 REFUELING OPERATIONS

3.9.5 Residual Heat Removal (RHR) and Coolant Circulation - High Water Level

LCO 3.9.5 One RHR loop shall be OPERABLE and in operation.

---NOTE---
The required RHR loop may be removed from operation for ≤ 1 hour per 8 hour period, provided no operations are permitted that would cause introduction of coolant into the Reactor Coolant System with boron concentration less than that required to meet the minimum required boron concentration of LCO 3.9.1.

APPLICABILITY: MODE 6 with the water level ≥ 23 ft above the top of reactor vessel flange.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. RHR loop requirements not met.	A.1	Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet the boron concentration of LCO 3.9.1.	Immediately
	AND		
	A.2	Suspend loading irradiated fuel assemblies in the core.	Immediately
	AND		
	A.3	Initiate action to satisfy RHR loop requirements.	Immediately
	AND		

(continued)

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. (continued)	A.4	Close equipment hatch and secure with four bolts.	4 hours
	AND		
	A.5	Close one door in each air lock.	4 hours
	AND		
	A.6.1	Close each penetration providing direct access from the containment atmosphere to the outside atmosphere with a manual or automatic isolation valve, blind flange, or equivalent.	4 hours
	OR		
	A.6.2	Verify each penetration is capable of being closed by an OPERABLE automatic Containment Ventilation isolation valve.	4 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.9.5.1	Verify one RHR loop is in operation and circulating reactor coolant at a flow rate of $\geq$ 2000 gpm.	In accordance with the Surveillance Frequency Control Program

3.9 REFUELING OPERATIONS

3.9.6 Residual Heat Removal (RHR) and Coolant Circulation - Low Water Level

LCO 3.9.6 Two RHR loops shall be OPERABLE, and one RHR loop shall be in operation.

----------NOTES----------

1. All RHR pumps may be removed from operation for ≤ 15 minutes when switching from one train to another provided:
   a. The core outlet temperature is maintained > 10 degrees F below saturation temperature;
   b. No operations are permitted that would cause introduction of coolant into the Reactor Coolant System (RCS) with boron concentration less than that required to meet the minimum required boron concentration of LCO 3.9.1; and
   c. No draining operations to further reduce RCS water volume are permitted.
2. One required RHR loop may be inoperable for up to 2 hours for surveillance testing, provided that the other RHR loop is OPERABLE and in operation.

APPLICABILITY: MODE 6 with the water level < 23 ft above the top of reactor vessel flange.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. Less than the required number of RHR loops OPERABLE.	A.1	Initiate action to restore required RHR loops to OPERABLE status.	Immediately
	OR		
	A.2	Initiate action to establish ≥ 23 ft of water above the top of reactor vessel flange.	Immediately

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION		COMPLETION TIME
B. No RHR loop in operation.	B.1	Suspend operations that would cause introduction of coolant into the RCS with boron concentration less than required to meet the boron concentration of LCO 3.9.1.	Immediately
	AND		
	B.2	Initiate action to restore one RHR loop to operation.	Immediately
	AND		
	B.3	Close equipment hatch and secure with four bolts.	4 hours
	AND		
	B.4	Close one door in each air lock.	4 hours
	AND		

(continued)

ACTIONS (continued)

CONDITION	REQUIRED ACTION	COMPLETION TIME
B. (continued)	B.5.1 Close each penetration providing direct access from the containment atmosphere to the outside atmosphere with a manual or automatic isolation valve, blind flange, or equivalent. OR	4 hours
	B.5.2 Verify each penetration is capable of being closed by an OPERABLE automatic Containment Ventilation isolation valve.	4 hours

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.9.6.1	Verify one RHR loop is in operation and circulating reactor coolant at a flow rate of ≥ 2000 gpm.	In accordance with the Surveillance Frequency Control Program
SR 3.9.6.2	Verify correct breaker alignment and indicated power available to the required RHR pump that is not in operation.	In accordance with the Surveillance Frequency Control Program

3.9 REFUELING OPERATIONS

3.9.7 Refueling Cavity Water Level

LCO 3.9.7 Refueling cavity water level shall be maintained ≥ 23 ft above the top of reactor vessel flange.

APPLICABILITY: During movement of irradiated fuel assemblies within containment.

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. Refueling cavity water level not within limit.	A.1 Suspend movement of irradiated fuel assemblies within containment.	Immediately

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.9.7.1	Verify refueling cavity water level is ≥ 23 ft above the top of reactor vessel flange.	In accordance with the Surveillance Frequency Control Program

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO AMENDMENT NOS. 181 AND 86

TO FACILITY OPERATING LICENSE NOS. NPF-90 AND NPF-96

TENNESSEE VALLEY AUTHORITY

WATTS BAR NUCLEAR PLANT, UNITS 1 AND 2

DOCKET NOS. 50-390 AND 50-391

1.0 INTRODUCTION

By application dated August 25, 2025 (Agencywide Documents Access and Management System Accession No. ML25237A197), as supplemented by letter dated February 24, 2026 (ML26055A349), the Tennessee Valley Authority (TVA, the licensee) submitted a license amendment request (LAR) to the U.S. Nuclear Regulatory Commission (NRC, the Commission) for changes to the Watts Bar Nuclear Plant (WBN), Units 1 and 2, technical specifications (TSs). The requested changes would revise various WBN, Units 1 and 2, TSs in Sections 1.4, 3.4, 3.7, and 3.9 by adopting the following approved Technical Specifications Task Force (TSTF) travelers:

- TSTF-26, Revision 0 – "Revise the Action for Minimum Temperature for Criticality to match the Applicability"
- TSTF-27, Revision 3 – "Revise SR frequency for Minimum Temperature for Criticality"
- TSTF-87, Revision 2 – "Revise 'RTBs open' & 'CRDM de-energized' Actions to 'incapable of rod withdrawal'"
- TSTF-96, Revision 1 – "Delete the initial performance of the boron concentration measurement with no source range detectors"
- TSTF-116, Revision 2 – "RCS Inventory Balance SR: Steady State Clarification"
- TSTF-197, Revision 2 – "Require containment closure when shutdown cooling requirements are not met"
- TSTF-233, Rev 0 – "Relocate LTOP Arming Temperature to PTLR"
- TSTF-263, Revision 3 – "Correct usage of 'required' components and base ACTIONS on inoperable required equipment"
- TSTF-265, Revision 2 – "Clarify 3.4 'non-operating loop' SRs"
- TSTF-272, Revision 1 – "Refueling Boron Concentration Clarification"
- TSTF-280, Revision 1 – "Exempt SRs on LTOP equipment not used to satisfy the LCO"
- TSTF-284, Revision 3 – "Add 'Met vs. Perform' to Specification 1.4, Frequency"
- TSTF-286, Revision 2 – "Define 'Operations Involving Positive Reactivity Additions'"

- TSTF-349, Revision 1 – "Add Note to LCO 3.9.5 Allowing Shutdown Cooling Loops Removal from Operation"
- TSTF-352, Revision 1 – "Provide Consistent Completion Time to Reach MODE 4"
- TSTF-361, Revision 2 – "Allow standby SDC/RHR/DHR loop to inoperable to support testing"
- TSTF-438, Revision 0 – "Clarify Exception Notes to be Consistent with the Requirement Being Excepted"
- TSTF-20, Revision 0 – "Delete extraneous Action from Refueling Cavity Water Level"
- TSTF-247, Revision 0 – "Provide separate condition entry for each PORV and block valve"
- TSTF-309, Revision 2 – "Revise Pressurizer PORV Actions to not require cycling of block valve when closed for isolation"

A notice of consideration of proposed issuance of amendments with proposed no significant hazards consideration (NSHC) determinations was published in the *Federal Register* on November 25, 2025 (90 FR 53396). One comment was received; however, it was unrelated to this LAR. The supplement dated February 24, 2026, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the staff's original proposed NSHC determinations as published in this notice.

2.0 REGULATORY EVALUATION

2.1 Requested Changes

The proposed TS changes for each TSTF traveler are described in the relevant sections of this safety evaluation (SE).

2.2 Applicable Regulations and Guidance

As described in Title 10 of the *Code of Federal Regulations* (10 CFR) 50.92(a), in determining whether an amendment to a license will be issued to the applicant, the Commission will be guided by the considerations which govern the issuance of initial licenses to the extent applicable and appropriate. The general considerations that guide the Commission include, as stated in 10 CFR 50.40(a), how the TSs provide reasonable assurance that the health and safety of the public will not be endangered. Also, to issue an operating license, of which TSs are a part, the Commission must make the findings of 10 CFR 50.57, "Issuance of operating license," including the 10 CFR 50.57(a)(3)(i) finding that there is reasonable assurance that the activities authorized by the operating license can be conducted without endangering the health and safety of the public.

The regulation at 10 CFR 50.36(a)(1) requires each applicant for a license authorizing operation of a utilization facility to include in the application proposed TSs in accordance with the requirements of 10 CFR 50.36, "Technical specifications." The regulation at 10 CFR 50.36(a)(1) requires an applicant to submit, as part of the application, a "summary statement of the bases or reasons for such specifications, other than those covering administrative controls." However, per 10 CFR 50.36(a)(1), these TS bases "shall not become part of the technical specifications."

As required by 10 CFR 50.36(b), the TSs will be derived from the analyses and evaluation included in the safety analysis report, and amendments thereto, submitted pursuant to

10 CFR 50.34, “Contents of applications; technical information.” The Commission may include such additional TSs as the Commission finds appropriate.

The regulation at 10 CFR 50.36(c) requires, in part, that the TSs include items in the following categories: (1) safety limits (SLs), limiting safety systems settings, and limiting control settings; (2) limiting conditions for operation (LCOs); (3) surveillance requirements (SRs); (4) design features; and (5) administrative controls.

As required by 10 CFR 50.36(c)(2)(i), the TSs will include LCOs, which are the lowest functional capability or performance levels of equipment required for safe operation of the facility. Per 10 CFR 50.36(c)(2)(i), when an LCO of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the TSs until the LCO can be met.

The regulation at 10 CFR 50.36(c)(3), “Surveillance requirements,” requires TSs to include items in the category of SRs, which are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within SLs, and that the LCOs will be met.

The regulation at 10 CFR 50.36(c)(5), “Administrative controls,” requires administrative controls, which are the provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner. The regulation further requires each licensee to submit any reports to the Commission pursuant to approved TSs as specified in 10 CFR 50.4, “Written communications.”

The NRC staff’s guidance for the review of TSs is in chapter 16, “Technical Specifications,” of NUREG-0800, Revision 3, “Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants” (SRP), dated March 2010 (ML100351425). As described therein, as part of the regulatory standardization effort, the NRC staff has prepared Standard Technical Specifications (STS) for each of the light-water reactor nuclear designs. NUREG-1431[1] contains the STS for Westinghouse plants.

Section IV, “The Commission Policy,” of the Final Policy Statement on Technical Specifications Improvements for Nuclear Power Reactors (58 FR 39132), dated July 22, 1993, states, in part:

> The purpose of Technical Specifications is to impose those conditions or limitations upon reactor operation necessary to obviate the possibility of an abnormal situation or event giving rise to an immediate threat to the public health and safety by identifying those features that are of controlling importance to safety and establishing on them certain conditions of operation which cannot be changed without prior Commission approval….
>
> [T]he Commission will also entertain requests to adopt portions of the improved STS, even if the licensee does not adopt all STS improvements….
>
> The Commission encourages licensees to use the improved STS as the basis for plant-specific [TSs]….

[1] U.S. Nuclear Regulatory Commission, “Standard Technical Specifications, Westinghouse Plants,” NUREG-1431, Volume 1, “Specifications,” and Volume 2, “Bases,” Revision 5, September 2021 (ML21259A155 and ML21259A159, respectively).

> [I]t is the Commission intent that the wording and Bases of the improved STS be used … to the extent practicable.

Although the NRC-approved TSTF travelers listed in the LAR were based on changes to previous revisions of NUREG-1431, the NRC staff's review of this LAR includes consideration of whether the proposed changes are consistent with Revision 5 of NUREG-1431. Additionally, the NRC staff determined that the TS changes approved in the TSTF travelers in the LAR are applicable to the WBN TSs because the units are a Westinghouse design, and the NRC staff approved the various travelers for Westinghouse designs. Revision 5 of NUREG-1431 provides example TSs that meet the requirements in 10 CFR 50.36 for a standard Westinghouse plant design.

3.0 TECHNICAL EVALUATION

3.1 TSTF-26, Revision 0 – "Revise the Action for Minimum Temperature for Criticality to match the Applicability"

3.1.1 Description of TSTF

TSTF-26, Revision 0 (ML040350532), proposed changes to NUREG-1431, Revision 1, TS 3.4.2. The APPLICABILITY of TS 3.4.2 in NUREG-1431, Revision 1, is MODE 1 and MODE 2 with [effective multiplication factor] $k_{eff} \geq 1.0$. Required Action A.1 required the plant to "Be in MODE 3." TSTF-26-A, Revision 0, changed Required Action A.1 to "Be in MODE 2 with $k_{eff} < 1.0$."

This traveler preceded the current NRC practice of documenting approval in an SE. However, an NRC letter from Christopher Grimes to James Davis (Nuclear Energy Institute), dated September 27, 1996, documented approval (ML20129E506).

3.1.2 Proposed TS Changes

As described in LAR Section 2.1, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to change WBN, Units 1 and 2, TS 3.4.2 Required Action A.1 from "Be in MODE 3." to "Be in MODE 2 with $k_{eff} < 1.0$."

3.1.3 Staff Evaluation of Proposed TS Changes and Variation

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler. The NRC staff determined that the proposed changes are identical to the TS changes in the approved TSTF traveler and are consistent with the STS (NUREG-1431).

The NRC staff reviewed the non-technical variation described by the licensee in Section 2.1.2 of the LAR. The NRC staff determined that the variation is appropriate for WBN due to plant-specific differences.

The NRC staff finds that the proposed changes to Required Action A.1 are consistent with the STS and TSTF-26-A, Revision 0, and the pertinent portions of STS and TSTF-26 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.2 TSTF-27, Revision 3 – "Revise SR frequency for Minimum Temperature for Criticality"

3.2.1 Description of TSTF

TSTF-27, Revision 3 (ML040350548), proposed changes to NUREG-1431, Revision 1, SR 3.4.2.1 by deleting the NOTE above the frequency and revising the frequency to "12 hours." The frequency for SR 3.4.2.1 in NUREG-1431, Revision 1, was "30 minutes thereafter" and was preceded by a NOTE which stated: "Only required if [T_{avg} - T_{ref} deviation, low low T_{avg}] alarm not reset and any RCS [reactor coolant system] loop T_{avg}, <[547] °F [degrees Fahrenheit]."

The changes were justified as an improved frequency statement presentation that would allow operators to improve their focus on plant safety during plant start up. This traveler preceded the current NRC practice of documenting approval in an SE. However, a summary of an April 22, 1998, meeting (ML18284A132) stated that TSTF-27 was approved and the change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.2.2 Proposed TS Changes and Variations

As described in LAR Section 2.2, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-27 changes to WBN, Units 1 and 2, SR 3.4.2.1 with a variation. Since the current frequency of WBN, Units 1 and 2, SR 3.4.2.1 is "In accordance with the Surveillance Frequency Control Program," the licensee would not be changing the frequency to 12 hours. The licensee stated this was a technical variation.

3.2.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler.

Regarding the licensee-identified variation, the licensee stated:

> TVA is applying the Surveillance Frequency Control Program (SFCP) to the Frequency of SR 3.4.2.1, in lieu of the Frequency of 12 hours provided in the traveler. This SR is analogous to NUREG-1431 SR 3.4.2.1, which has an SFCP Frequency of either 12 hours or in accordance with the SFCP as derived from TSTF-425-A, "Relocate Surveillance Frequencies to Licensee Control – RITSTF [Risk-Informed Technical Specifications Task Force] Initiative 5b." TSTF-425 was adopted by WBN Units 1 and 2 with License Amendments 132 and 36, respectively (ML20028F733). The application of the SFCP to the proposed SR 3.4.2.1 Frequency is consistent with the SFCP inclusion criteria contained in the Proposed Safety Evaluation provided in the Federal Register Notice of Availability for TSTF-425.

The NRC staff reviewed the variation described by the licensee. The NRC staff determined that the variation is appropriate for WBN, Units 1 and 2, due to the licensing basis, which allows control of the SR frequency in the WBN SFCP.

The NRC staff finds that deletion of the NOTE above the SR 3.4.2.1 frequency is consistent with the STS and TSTF-27-A, Revision 3, and the pertinent portions of STS and TSTF-27 are

applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.3 TSTF-87, Revision 2 – "Revise 'RTBs open' & 'CRDM de-energized' Actions to 'incapable of rod withdrawal'"

3.3.1 Description of TSTF

TSTF-87, Revision 2 (ML040440223), modified TS 3.4.5 and TS 3.4.9 in NUREG-1431, Revision 1. TS 3.4.5, Required Actions C.2 and D.1 were changed from "De-energize all control rod drive mechanisms" to "Place the Rod Control System in a condition incapable of rod withdrawal." TS 3.4.9, Required Action A.1, was changed from "Be in MODE 3 with reactor trip breakers [RTBs] open" to "Place the Rod Control System in a condition incapable of rod withdrawal," and to require full insertion of all rods.

The changes were justified as necessary to eliminate undesirable secondary effects of opening the RTBs. By opening the RTBs, plant interlock P-4 is tripped, which results in a trip of the main turbine and will close the main and bypass feedwater lines if RCS T_{avg} is below the low setpoint in MODE 3. Forcing reliance on auxiliary feedwater in this condition is not the intent, nor is it desirable, over continued use of normal feedwater.

This traveler preceded the current NRC practice of documenting approval in an SE. However, the change was adopted into Revision 2 of NUREG-1431 and every subsequent revision (ML15239A098).

3.3.2 Proposed TS Changes and Variations

As described in LAR Section 2.3, and illustrated in Attachments 1, 2, 3 and 4, as supplemented, the licensee proposed to adopt the TSTF-87 changes to WBN, Units 1 and 2, with a variation. Along with changes to TS 3.4.5 and 3.4.9, the licensee proposed adopting TSTF-87 changes to TS 3.4.6 Required Actions C.2 and D.1. The licensee stated this was a technical variation.

3.3.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler.

Regarding the licensee-identified variation, the licensee stated:

> TVA is also applying TSTF-87-A, Revision 2 to WBN Units 1 and 2 TS 3.4.6, "RCS Loops - Mode 4," Required Action C.2 (revised as B.2), which currently states, "De-energize all control rod drive mechanisms (CRDMS)." This Required Action is being revised to state, "Place the Rod Control System in a condition incapable of rod withdrawal" consistent with TSTF-87-A, Revision 2. As noted in Section 2.8.2 of this enclosure regarding TSTF-263-A, current WBN Units 1 and 2 TS 3.4.6 Condition C (revised Condition B) is unique to WBN; however, the proposed revision to adopt TSTF-87-A would also apply to this Condition.

> TVA is also applying TSTF-87-A, Revision 2 to WBN Units 1 and 2 TS 3.4.6, "RCS Loops - Mode 4," Required Action D.1 (revised as C.1), which currently states, "De-energize all control rod drive mechanisms (CRDMs)." This Required Action is being revised to state, "Place the Rod Control System in a condition incapable of rod withdrawal," consistent with TSTF-87-A, Revision 2 and the above proposed change to Required Action C.2 (revised as B.2). The proposed revision to Required Action D.1 (revised as C.1) appropriately addresses the WBN Units 1 and 2 TS 3.4.6 Limiting Condition for Operation (LCO) 3.4.6, which states "Two RCS loops, and both loops shall be in operation, when the rod control system is capable of rod withdrawal." The current WBN Units 1 and 2 TS 3.4.6 LCO 3.4.6 was added to the WBN Unit 1 TS during the development of the initial WBN Unit 1 TS (ML073200377) and subsequently incorporated into the initial issuance of the WBN Unit 2 TS (ML15273A064).

The NRC staff reviewed the variation described by the licensee, as supplemented. The NRC staff determined that applying TSTF-87 changes to current TS 3.4.6 Required Actions C.2 and D.1, is acceptable because while TS 3.4.6 Required Actions C.2 and D.1 are unique to WBN, Units 1 and 2, the TSTF-87 justification applies to the Required Actions as well as those in common with NUREG-1431.

The NRC staff finds that the proposed changes to the Conditions and Required Actions are consistent with the STS and TSTF-87-A, Revision 2, and the pertinent portions of STS and TSTF-87 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, as supplemented, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.4 TSTF-96, Revision 1 – "Delete the initial performance of the boron concentration measurement with no source range detectors"

3.4.1 Description of TSTF

TSTF-96, Revision 1 (ML040490043), modified NUREG-1431, Revision 1, by changing the TS 3.9.3 Required Action B.2 Completion Time from "4 hours AND Once per 12 hours thereafter" to "Once per 12 hours."

The change was justified by the TSTF which stated:

> The accelerated performance of this SR is not warranted based on routine performance of this SR (every 72 hours), and knowledge of stable conditions prior to the loss of the source range monitor. Secondarily, RCS dilution events are recognizable through other means such as uncontrolled increases in pool water level.

This traveler preceded the current NRC practice of documenting approval in an SE. A summary for a September 24, 1998, meeting (ML20154D539) states that TSTF-96 was approved. The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.4.2 Proposed TS Changes and Variations

As described in LAR Section 2.4, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-96 changes to WBN, Units 1 and 2, TS 3.9.2 Required Action B.2

Completion Times. The licensee stated the TS number difference between NUREG-1431, Revision 1, and the WBN TSs was a result of a TS number change introduced by TSTF-23 between Revisions 1 and 2 of NUREG-1431.

3.4.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler.

The NRC staff reviewed the variations described by the licensee. The NRC staff determined that the variations are non-technical and appropriate for WBN.

The NRC staff finds that changes to the WBN, Units 1 and 2, TS 3.9.2 Required Action B.2 Completion Times are consistent with the STS and TSTF-96-A, Revision 1. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.5 TSTF-116, Revision 2 – "RCS Inventory Balance SR: Steady State Clarification"

3.5.1 Description of TSTF

TSTF-116 (ML040480059) modified the NOTEs for SR 3.4.13.1 and added NOTEs above TS 3.4.15 Required Actions A.1, B.1.2 and C.2. The modifications and additions were justified to address the fact that a RCS inventory balance cannot provide useful information unless the unit is at steady state conditions.

This traveler preceded the current NRC practice of documenting approval in an SE. A summary for a September 24, 1998, meeting (ML20154D539) states that TSTF-116 was approved. The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.5.2 Proposed TS Changes and Variations

As described in LAR Section 2.5, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to add NOTEs above TS 3.4.15 Required Actions A.1 and B.1.2 to WBN, Units 1 and 2. The licensee stated TSTF-116-related changes to SR 3.4.13.1 and TS 3.4.15 Required Action C.2 were not being made because SR 3.4.13.1 has the TSTF-116-related change and the TSTF-116-A change to TS 3.4.15, Condition C, does not apply to WBN, Units 1 and 2. The licensee also stated that the LCO statement for TS 3.4.15 in WBN, Unit 2, TSs would adopt the presentation of NUREG-1431 by adding an "a." and "b." in front of the phrases denoting the containment pocket sump level monitor and containment atmosphere particulate monitor, respectively.

3.5.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler.

The NRC staff reviewed the variations described by the licensee. The NRC staff determined that the variations are acceptable and appropriate for WBN.

The NRC staff finds that adding a NOTE above TS 3.4.15 Required Actions A.1 and B.1.2 is consistent with the STS and TSTF-116-A, Revision 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.6 TSTF-197, Revision 2 – "Require containment closure when shutdown cooling requirements are not met"

3.6.1 Description of TSTF

TSTF-197, Revision 2 (ML040560357), replaced single Required Actions for containment penetrations with a list of Required Actions for each type of containment penetration. The changes were justified as providing more detail, removing overly restrictive and/or vague language while preserving the requirement to establish containment closure.

This traveler preceded the current NRC practice of documenting approval in an SE. A letter dated July 26, 1999 (ML19067A141), states that TSTF-197 was approved. The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.6.2 Proposed TS Changes and Variations

As described in LAR Section 2.6, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed adopting the TSTF-197 changes to WBN, Units 1 and 2, TS 3.9.5 Required Action A.4 and TS 3.9.6 Required Action B.3. The licensee identified a variation from TSTF-197, TS 3.9.5 Required Action A.6.2 and TS 3.9.6 Required Action B.5.2 which refer to an "OPERABLE Containment Purge and Exhaust Isolation System." The licensee proposed to use the phrase "OPERABLE automatic Containment Ventilation isolation valve." The licensee stated that the variation is consistent with WBN design.

3.6.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler.

The NRC staff reviewed the variations described by the licensee. The NRC staff determined that the variations are acceptable and appropriate for WBN because the variation adapts TSTF-197 phrasing to the WBN design nomenclature.

The NRC staff finds that adopting the TSTF-197 changes to WBN, Units 1 and 2, TS 3.9.5 Required Action A.4 and TS 3.9.6 Required Action B.3 is consistent with the STS and TSTF-197-A, Revision 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.7 TSTF-233, Revision 0 – "Relocate LTOP Arming Temperature to PTLR"

3.7.1 Description of TSTF

TSTF-233, Revision 0 (ML040580472), changed TSs 3.4.6, 3.4.7, 3.4.10, 3.4.12, and 3.5.2 in NUREG-1431, Revision 1, by replacing the explicit listing of the temperature below which the Low Temperature Overpressure Protection (LTOP) system must be operable in various LCO statements, Applicability Statements, Required Actions, and SR Notes with a reference to the temperature as specified in the Pressure Temperature Limits Report (PTLR). The Definition of PTLR in TS Section 1.1 and the description of the PTLR in TS Section 5.6 were also modified to include the LTOP arming temperature.

The TSTF-233 changes to NUREG-1431 were justified as follows:

> The specific value for the limiting RCS cold leg temperature, below which the reactor vessel may suffer damage from a cold overpressure event, is reactor vessel plant specific and varies with vessel fluence. Use of a specific value, which will require periodic amendment is not consistent with the PTLR philosophy. Reference to the PTLR for other plant specific temperature (e.g., LCO 3.4.3, with the PTLR reference is used) is acceptable, and results in simplifying the revision process when the temperature value changes with reactor fluence. Periodic updates to the vessel limiting temperature can be accommodated without going through the license amendment process. The methodology used to determine the limiting temperature is controlled by TS and requires NRC approval for changes. Relocating this value to the PTLR is consistent with the Westinghouse PTLR methodology Topical Report, which is the only approved PTLR Topical Report.

This traveler preceded the current NRC practice of documenting approval in an SE. A letter dated July 16, 1998 (ML20236T184), states that TSTF-233 was approved. The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.7.2 Proposed TS Changes and Variations

As described in LAR Section 2.7, and illustrated in Attachments 1, 2, 3 and 4, as supplemented, the licensee proposed to revise WBN, Units 1 and 2, TSs 1.1, 3.4.6, 3.4.7, 3.4.10, and 3.4.12 with TSTF-233-related changes. The licensee proposed the non-adoption of TSTF-233-related changes to TS 3.5.2 and TS 5.9.6. The licensee listed the variations between TSTF-233 and the proposed changes to WBN, Units 1 and 2, TSs in Section 2.7.2 of the LAR.

3.7.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler. The staff confirmed the TSTF traveler is applicable to WBN, Units 1 and 2.

The NRC staff reviewed the variations described by the licensee in Section 2.7.2 of the LAR, as supplemented. The NRC staff determined that the variations are acceptable and appropriate for WBN because the variations adapt TSTF-233 phrasing to the WBN design nomenclature (i.e., WBN uses the term Cold Overpressure Mitigation System (COMS) instead of LTOP System)

and do not change the intent or result of the TSTF-233-related changes. The staff determined that non-adoption of TSTF-233 changes to TS 3.5.2 and TS 5.9.6 is acceptable because LCO NOTE 2 for TS 3.5.2 will still contain the explicit temperature limit and TS 5.9.6.a currently specifies a COMS arming temperature for the PTLR.

The NRC staff finds that adopting the TSTF-233 changes to WBN, Units 1 and 2, is consistent with the STS and TSTF-233, Revision 0, and the cited variations are acceptable. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, as supplemented, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.8 TSTF-263, Revision 3 – "Correct usage of 'required' components and base ACTIONS on inoperable required equipment"

3.8.1 Description of TSTF

TSTF-263, Revision 3 (ML993540010), clarified the Conditions descriptions and SRs in TSs 3.4.5, 3.4.6, 3.4.7, and 3.4.8 of all pressurized-water reactor (PWR) STS by adding the term "required" and adding new Conditions A and B to TS 3.4.7.

This traveler preceded the current NRC practice of documenting approval in an SE. TSTF-263 was approved in a letter dated December 21, 1999 (ML993630256). The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.8.2 Proposed TS Changes and Variations

As described in LAR Section 2.8, and illustrated in Attachments 1, 2, 3 and 4, as supplemented, the licensee proposed to adopt the TSTF-263 changes to WBN, Units 1 and 2, TSs 3.4.5, 3.4.6, 3.4.7, and 3.4.8 with four variations.

3.8.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler. In Section 2.8.2 of the LAR, as supplemented, the licensee described the variations between the changes to NUREG-1431 made by TSTF-263 and the changes made to WBN, Units 1 and 2, TSs.

The NRC staff reviewed the variations described by the licensee. The NRC staff determined that the variations are appropriate and acceptable for WBN due to plant-specific differences in the WBN TSs that exist due to WBN's licensing basis history.

The NRC staff finds that adopting the TSTF-263 changes to WBN, Units 1 and 2, is consistent with the STS and TSTF-263, Revision 3, and the cited variations are acceptable. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, as supplemented, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.9 TSTF-265, Revision 2 – "Clarify 3.4 'non-operating loop' SRs"

3.9.1 Description of TSTF

TSTF-265, Revision 2 (ML993540010), changed PWR STS SRs for RCS pumps that require the verification of the correct breaker alignment and indicated power available to the required pump that is not in operation by clarifying that the SRs are applicable to each required pump. For NUREG-1431, in particular, SRs 3.4.5.3, 3.4.6.3, 3.4.7.3, and 3.4.8.2, which had stated: "Verify correct breaker alignment and indicated power are available to the required pump that is not in operation," were changed to state: "Verify correct breaker alignment and indicated power are available to each required pump." A NOTE was added above each of the changed SRs, which stated, "Not required to be performed until 24 hours after a required pump is not in operation."

The TSTF-265 changes were justified because the changes clarified that the SRs were applicable to each required pump regardless of its operating status, since both pumps may be operating. The NOTE was justified because without it, when pumps are swapped, the SR is immediately not met on the pump taken out of operation. The NOTE avoids entering an ACTION or invoking SR 3.0.3 for a routine operational occurrence. The change was deemed acceptable because adequate assurance exists that the pump is aligned to the correct breaker with power available because, prior to being removed from operation, the applicable pump had been in operation. Allowing 24 hours to perform the breaker alignment verification was deemed acceptable because the pump was in operation, which demonstrated its OPERABILITY.

This traveler preceded the current NRC practice of documenting approval in an SE. A letter dated December 21, 1999 (ML993630256), states that TSTF-265 was approved. The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.9.2 Proposed TS Changes and Variations

As described in LAR Section 2.9, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-265 changes to the WBN, Units 1 and 2, TSs with one non-technical variation and one technical variation.

3.9.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler. The staff determined that the proposed changes are consistent with the TS changes in the approved TSTF traveler and are consistent with the STS (NUREG-1431).

The NRC staff reviewed the non-technical variation described by the licensee in Section 2.9.2 of the LAR. The NRC staff determined that the variation is appropriate for WBN due to plant-specific differences in SR numbering.

The NRC reviewed the technical variation described by the licensee in Section 2.9.2 of the LAR. The staff determined that the frequencies for the SRs being controlled in the WBN's SFCP, instead of being fixed at every 7 days, does not invalidate the justification for the TSTF-265 changes. The NRC staff determined that the licensee-identified variation is acceptable.

The NRC staff finds that the proposed changes to SRs 3.4.5.3, 3.4.6.4, 3.4.7.3, and 3.4.8.2 are consistent with the STS and TSTF-265, Revision 2, and that the pertinent portions of STS and TSTF-265 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.10 TSTF-272, Revision 1 – "Refueling Boron Concentration Clarification"

3.10.1 Description of TSTF

TSTF-272, Revision 1 (ML993540010), changed PWR STS by adding a NOTE below the APPLICABILITY statement for TS 3.9.1, "Boron Concentration." TS 3.9.1 limits the boron concentrations of the RCS, the refueling canal, and the refueling cavity during refueling to ensure that the reactor remains subcritical. The NOTE stated: "Only applicable to the refueling canal and refueling cavity when connected to the RCS."

The TSTF-272 changes were justified because when the refueling canal and refueling cavity are isolated from the RCS, no potential for dilution exists. In this condition it is not necessary to place a limit on the boron concentration of water in the refueling cavity and the refueling canal. The NOTE clarifies that the LCO only applies to the refueling canal and the refueling cavity when those volumes are connected to the RCS.

This traveler preceded the current NRC practice of documenting approval in an SE. TSTF-272 was approved in a letter dated December 21, 1999 (ML993630256). The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.10.2 Proposed TS Changes and Variations

As described in LAR Section 2.10, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-272 changes to the WBN, Units 1 and 2, TSs with no variations.

3.10.3 Staff Evaluation of Proposed TS Changes

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler. The staff determined that the proposed changes are consistent with the TS changes in the approved TSTF traveler and are consistent with the STS (NUREG-1431).

The NRC staff finds that the proposed changes to TS 3.9.1 are consistent with the STS and TSTF-272, Revision 1, and are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.11 TSTF-280, Revision 1 – "Exempt SRs on LTOP equipment not used to satisfy the LCO"

3.11.1 Description of TSTF

TSTF-280, Revision 1 (ML040611059), changed the PWR STS LCO 3.4.12 by changing the presentation of the list of options for required equipment in the LCO statement, adding the word

“required” in SR 3.4.12.4 to verify an RCS vent is open, and deleting the existing NOTE in the same SR.

TSTF-280 changes were justified because changing the presentation of the options for required equipment in the LCO statement was an editorial change and adding the word “required” to the SR made the existing SR NOTE redundant, justifying its deletion.

This traveler preceded the current NRC practice of documenting approval in an SE. TSTF-280 was approved in a letter dated July 26, 1999 (ML19067A141). The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.11.2 Proposed TS Changes and Variations

As described in LAR Section 2.11, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-280 changes to the WBN, Units 1 and 2, TSs with four non-technical variations and one technical variation.

3.11.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee’s submittal and comparing it to the approved TSTF traveler. The staff determined that the proposed changes are consistent with the TS changes in the approved TSTF traveler and are consistent with the STS (NUREG-1431).

The NRC staff reviewed the non-technical variations described by the licensee in Sections 2.11.2.1, 2.11.2.2, 2.11.2.4, and 2.11.2.5 of the LAR. The NRC staff determined that the variation in Section 2.11.2.1 is appropriate due to plant specific differences in system nomenclature. The NRC staff determined that the variation in Section 2.11.2.2 is appropriate due to plant specific design differences. The staff determined that the variation in Section 2.11.2.4, frequencies for the SR being controlled in the WBN’s SFCP instead of being fixed, does not invalidate the justification for the TSTF-280 changes and is appropriate. The NRC staff determined that the variation in Section 2.11.2.5, changing “NOTE” to “NOTES” in the WBN, Unit 2 LCO is an appropriate editorial correction needed to match the number of items in the NOTES.

The NRC staff reviewed the technical variations described by the licensee in Section 2.11.2.3 of the LAR and determined that keeping the text of the LCO and SR does not invalidate the justification for the TSTF-280 changes and is appropriate due to WBN’s licensing basis history.

The NRC staff finds that the proposed changes to LCO 3.4.12 and SR 3.4.12.4 are consistent with the STS and TSTF-280, Revision 1, and the pertinent portions of STS and TSTF-280 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.12 TSTF-284, Revision 3 – "Add 'Met vs. Perform' to Specification 1.4, Frequency"

3.12.1 Description of TSTF

TSTF-284, Revision 3, changed PWR STS by adding a discussion paragraph into Specification 1.4, and several new examples to facilitate the use and application of SR Notes that utilize the terms "met" and "perform." SRs were also modified as necessary to appropriately use "met" and "perform" exceptions.

TSTF-284 changes were justified because they alleviate misunderstanding and provide explicit direction for SR Notes with "met" or "performed" language. The changes also established consistency between boiling-water reactor (BWR) and PWR STS usage rules because the BWR STS usage rules included similar discussions and direction.

This traveler preceded the current NRC practice of documenting approval in an SE. TSTF-284 was approved in a letter dated February 16, 2000 (ML003684596). The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.12.2 Proposed TS Changes and Variations

As described in LAR Section 2.12, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-284 changes to the WBN, Units 1 and 2, TSs with four non-technical and two technical variations.

3.12.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler. The staff determined that the proposed changes are consistent with the TS changes in the approved TSTF traveler and are consistent with the STS (NUREG-1431).

The NRC staff reviewed the non-technical variations described by the licensee in Section 2.12.2 of the LAR. The NRC staff determined that the variation in WBN using "COMS" instead of "LTOP" is appropriate due to plant-specific differences in system nomenclature. The NRC staff determined that the variation in WBN of no longer having a NOTE corresponding to STS SR 3.4.12.5 is appropriate due to WBN's deletion of the NOTE as part of adoption of TSTF-280 (as discussed in section 3.11 of this safety evaluation). The NRC staff determined that the variation in the NOTE for WBN SR 3.4.12.7 is appropriate due to SR numbering differences between NUREG-1431 and the WBN TSs. The NRC staff determined that the variation in WBN of no longer having a TS corresponding to STS 3.9.4 is appropriate due to WBN's licensing basis.

The NRC staff reviewed the technical variations described by the licensee in Section 2.12.2 of the LAR. The staff determined that since WBN does not have a TS analogous to STS 3.1.11, WBN's non adoption of those changes is acceptable. The staff determined that the frequencies for SRs 3.4.11.1 and 3.4.11.2 being controlled in the WBN's SFCP, instead of being fixed 192 days and 18 months, respectively, does not invalidate the justification for the TSTF-284 changes. The staff determined that the licensee-identified variation is acceptable.

The NRC staff finds that the proposed changes to the WBN TSs are consistent with the STS and TSTF-284, Revision 3, and the pertinent portions of STS and TSTF-284 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.13 TSTF-286, Revision 2 – "Define 'Operations Involving Positive Reactivity Additions'"

3.13.1 Description of TSTF

TSTF-286, Revision 2 (ML20106F133), changed PWR STS by removing requirements that prevented "positive reactivity addition" or "reduction in boron concentration" and replacing them with more generic requirements that ensured Shutdown Margin (SDM) would be maintained during RCS inventory or temperature adjustments.

TSTF-286 changes were justified because the TS would continue to limit any potential for excess positive reactivity addition while allowing safe reactor operations.

This traveler preceded the current NRC practice of documenting approval in an SE. TSTF-286 was approved in a letter dated July 6, 2000 (ML003730788). The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.13.2 Proposed TS Changes and Variations

As described in LAR Section 2.13, and illustrated in Attachments 1, 2, 3 and 4, as supplemented, the licensee proposed to adopt the TSTF-286 changes to WBN, Units 1 and 2, TS Sections 3.4 and 3.9 with four non-technical variations.

3.13.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler. The staff determined that the proposed changes are consistent with the TS changes in the approved TSTF traveler and are consistent with the STS (NUREG-1431).

The NRC staff reviewed the non-technical variations described by the licensee in Section 2.13.2 of the LAR, as supplemented. The NRC staff determined that the variation in WBN, TS 3.4.6 LCO NOTES compared to the STS 3.4.6 LCO NOTES justifies non-adoption of the TSTF-286 changes to the WBN, TS 3.4.6 LCO NOTES. Likewise, the NRC staff determined that the variation in WBN, TS 3.4.7 LCO NOTES compared to the STS 3.4.7 LCO NOTES justifies non-adoption of the TSTF-286 changes to the WBN, TS 3.4.7 LCO NOTES. The NRC staff determined that since the WBN TSs do not contain a TS 3.4.18, non-adoption of those TSTF-286 changes is justified. Finally, the staff reviewed the non-technical changes described in the final paragraph of Section 2.13.2 and determined that the variations are acceptable editorial variations.

The NRC staff finds that the proposed changes to the WBN TSs are consistent with the STS and TSTF-286, Revision 2, and the pertinent portions of STS and TSTF-286 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, as supplemented, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.14 TSTF-349, Revision 1 – "Add Note to LCO 3.9.5 Allowing Shutdown Cooling Loops Removal from Operation"

3.14.1 Description of TSTF

TSTF-349, Revision 1 (ML003723263), changed PWR STS LCO 3.9.6 by adding a NOTE to the RHR [residual heat removal] and Coolant Circulation – Low Water Level LCO to allow deenergizing all RHR pumps for up to 15 minutes to accommodate switching from one train to another, subject to maintaining certain RCS conditions. Specifically, for NUREG-1431, the NOTE would allow all RHR pumps to be deenergized if core outlet temperature is maintained more than 10 °F below saturation temperature, no operations that would cause a reduction of the RCS boron concentration occur and no operations to reduce RCS water volume occur.

TSTF-349 changes were justified because the TS would continue to limit the potential for uncontrolled heat up, the allowed time frame is short, and limitations are in place to preclude RCS boron reduction and draining activities.

This traveler preceded the current NRC practice of documenting approval in an SE. TSTF-349 was approved in a letter dated July 6, 2000 (ML003730788). The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.14.2 Proposed TS Changes and Variations

As described in LAR Section 2.14, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-349 changes to WBN, Units 1 and 2, TS 3.9.6 with non-technical variations.

The NRC staff reviewed the non-technical variations described by the licensee in Section 2.14.2 of the LAR. The NRC staff determined that the variation in WBN TSs using "NOTES" instead of "NOTE" and titling the TSTF-349 NOTE as "1" acceptable because of a second NOTE added to LCO 3.9.6 by adoption of TSTF-361, discussed in Section 3.16 of this SE. The NRC staff determined that the variation in WBN TSs using the phrase "All RHR pumps may be removed from operation..." instead of the TSTF-349 phrasing of "All RHR pumps may be *de-energized*..." acceptable because the rephrasing aligns with WBN's adoption of TSTF-438, discussed in Section 3.17 of this SE. The NRC staff determined that the variation in WBN TSs stating the second condition of the TSTF-349 NOTE: "No operations are permitted that would cause introduction of coolant into the Reactor Coolant System (RCS) with boron concentration less than that required to meet the minimum required boron concentration of LCO 3.9.1" instead of: "No operations are permitted that would cause a reduction of the Reactor Coolant System boron concentration" acceptable because the rephrasing aligns with the licensee's adoption of TSTF-286, discussed in Section 3.13 of this SE.

The NRC staff finds that the proposed changes to WBN TSs are consistent with the STS and TSTF-349, Revision 1, and the pertinent portions of STS and TSTF-349 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.14.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee's submittal and comparing it to the approved TSTF traveler. The staff determined that the proposed changes are consistent with the TS changes in the approved TSTF traveler and are consistent with the STS (NUREG-1431).

3.15 TSTF-352, Revision 1 – "Provide Consistent Completion Time to Reach MODE 4"

3.15.1 Description of TSTF

TSTF-352, Revision 1 (ML040630065), revised the PWR STS. The revised sections for NUREG-1431 are STS 3.4.10, "Pressurizer Safety Valves," STS 3.4.12, "Low Temperature Overpressure Protection (LTOP) System," STS 3.7.4, "Atmospheric Dump Valves (ADVs)," and STS 3.7.6, "Condensate Storage Tank (CST)."

The TSTF-352 changes were justified because the revisions increased the Completion Times (CTs) for Required Actions which direct entry into conditions below MODE 4 to allow sufficient time to safely perform the actions and increased the CTs for Required Actions which direct transitioning the plant from LTOP conditions to having the RCS vented to allow sufficient time to complete the activity.

This traveler preceded the current NRC practice of documenting approval in an SE. TSTF-352 was approved by letter dated February 16, 2000 (ML003684596). The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.15.2 Proposed TS Changes and Variations

As described in Section 2.15 of the LAR, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-352 changes to WBN, Units 1 and 2, TS 3.4.12 and TS 3.7.4 with non-technical variations.

3.15.3 Staff Evaluation of Proposed TS Changes and Variations

The NRC staff reviewed the non-technical variations described by the licensee in Section 2.15.2 of the LAR. The NRC staff determined that the variation in retaining the 12-hour CT for WBN TS 3.4.10, Condition B and the 18-hour CT for WBN TS 3.7.6, Condition B.2 is acceptable because it maintains the WBN licensing basis. The staff determined that the variation in WBN using the COMS naming convention is acceptable because it is a variation in name only. The staff determined that the variation in WBN Required Action lettering and numbering for WBN TS 3.7.4 is acceptable because WBN TS 3.7.4, Required Action D.2 corresponds to STS Required Action C.2.

The NRC staff finds that the proposed changes to WBN TSs are consistent with the STS and TSTF-352, Revision 1, and the pertinent portions of STS and TSTF-352 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.16 TSTF-361, Revision 2 – "Allow standby SDC/RHR/DHR [shutdown cooling/residual heat removal/decay heat removal] loop to inoperable to support testing"

3.16.1 Description of TSTF

TSTF-361, Revision 2 (ML003735328), changed the PWR STS by adding a NOTE to the RHR and Coolant Circulation – Low Water Level LCO to allow making the non-operating RHR loop inoperable for up to 2 hours to support performance of SRs. Specifically, for NUREG-1431, a NOTE was added to the LCO for TS 3.9.6 which stated, "One required RHR loop may be inoperable for up to 2 hours for surveillance testing, provided that the other RHR loop is OPERABLE and in operation."

TSTF-361 changes were justified because other RCS TS contained similar allowances and the change would support required outage activities while maintaining the plant in a safe condition with adequate time to respond to and mitigate expected failures.

This traveler preceded the current NRC practice of documenting approval in an SE. TSTF-349 was approved by letter dated October 31, 2000 (ML003775261). The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.16.2 Proposed TS Changes and Variations

As described in LAR Section 2.14, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-361 changes to WBN, Units 1 and 2, TS 3.9.6 with a non-technical variation.

3.16.3 Staff Evaluation of Proposed TS Changes and Variations

The NRC staff reviewed the non-technical variations described by the licensee in Section 2.14.2 of the LAR. The NRC staff determined that the variation in WBN TSs using "NOTES" instead of "NOTE" and titling the TSTF-361 NOTE as "2" acceptable because of the NOTE added to LCO 3.9.6 by adoption of TSTF-349, discussed in Section 3.14 of this SE.

The NRC staff finds that the proposed changes to WBN TSs are consistent with the STS and TSTF-361, Revision 2, and the pertinent portions of STS and TSTF-361 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.17 TSTF-438, Revision 0 – "Clarify Exception Notes to be Consistent with the Requirement Being Excepted"

3.17.1 Description of TSTF

TSTF-438, Revision 0 (ML021580308), changed STS by revising NOTEs to LCOs for RHR and RCS to clarify that the NOTEs are not restrictions. Specifically, for NUREG-1431 the phrase "…may be not in operation…" was changed to "…may be removed from operation…" and the phrase "may be de-energized…" was changed to "may be removed from operation…"

The NRC staff approved TSTF-438 in a letter dated October 21, 2002 (ML022940574). In the SE the staff stated:

> …This revision does not alter the intent of the existing notes, does not affect any safety analysis assumptions and does not alter any methods of plant operation. The proposed revisions are editorial and do not change the intent of the Technical Specifications or Bases. These changes cover NUREG-1430, NUREG-1431, NUREG-1432, NUREG-1433 and NUREG-1434.
>
> The NRC staff concludes that the proposal to revise the Notes to the RHR and RCS pumps LCOs contained in TSTF-438 are editorial and therefore are acceptable

The change was adopted into Revision 3 of NUREG-1431 and every subsequent revision.

3.17.2 Proposed TS Changes and Variations

As described in LAR Section 2.14, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-438 changes to WBN, Units 1 and 2, TSs 3.4.8 and 3.9.6 with non-technical variations.

3.17.3 Staff Evaluation of Proposed TS Changes and Variations

The NRC staff reviewed the non-technical variations described by the licensee in Section 2.14.2 of the LAR. The NRC staff determined that the variation in non-adoption of TSTF-438 changes to LCOs 3.4.6 and 3.4.7 in the WBN TSs is acceptable because the NOTEs for the WBN LCOs differ significantly from those in NUREG-1431. The NRC determined the variation in the current NOTE for WBN TS LCO 3.4.8 phrasing using “de-energized” is acceptable because the phrase would be changed by adoption of the TSTF-438 phrasing and the NOTE would not alter the intent of the existing NOTE.

The NRC staff finds that the proposed changes to WBN TSs are consistent with the STS and TSTF-438, Revision 0, and the pertinent portions of STS and TSTF-438 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.18 TSTF-20, Revision 0 – “Delete extraneous Action from Refueling Cavity Water Level”

3.18.1 Description of TSTF

TSTF-20, Revision 0 (ML040350519), deleted Required Action A.3 of TS 3.9.7 in NUREG-1431, Revision 1. Required Action A.3 stated: “Initiate action to restore refueling cavity water level to within limit.” The justification for deletion of Required Action A.3 was that the other two Required Actions, A.1 and A.2, result in exiting the APPLICABILITY of the LCO.

This traveler preceded the current NRC practice of documenting approval in an SE. The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision (ML15239A098).

3.18.2 Proposed TS Changes and Variations

As described in LAR Section 2.16, the licensee proposed to delete Required Action A.2 of WBN Units, 1 and 2, TS 3.9.7. The licensee stated that Required Action A.2 of WBN, Units 1 and 2, TS 3.9.7 correlates to Required Action A.3 of TS 3.9.7 in NUREG-1431, Revision 1, which

states: “Initiate action to restore refueling cavity water level to within limit.” The licensee stated the difference in Required Action numbers is a non-technical variation to the traveler.

3.18.3 Staff Evaluation of Proposed TS Changes and Variations

The licensee stated that the justification provided in the traveler is applicable to WBN. The NRC staff confirmed that the TSTF traveler is applicable to WBN by reviewing the licensee’s submittal and comparing it to the approved TSTF traveler. The staff determined that the proposed changes are identical to the TS changes in the approved TSTF traveler and are consistent with the STS (NUREG-1431).

The NRC staff reviewed the non-technical variation described by the licensee. The NRC staff determined that the variation is appropriate for WBN due to plant specific differences.

The NRC staff finds that the proposed deletion of Required Action A.2 is consistent with the STS and TSTF-20, Revision 0, because the remaining Required Action, A.1, will still result in exiting the APPLICABILITY of LCO 3.9.7. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.19 TSTF-247, Revision 0 – “Provide separate condition entry for each PORV and block valve”

3.19.1 Description of TSTF

TSTF-247, Revision 0 (ML040611022), added the phrase “and each block valve” to the existing TS 3.4.11 LCO NOTEs in NUREG-1431 and 1432 that state “Separate Condition Entry is allowed for each PORV [power-operated relief valve].” TSTF-247 also modified Condition F to apply when all block valves are inoperable and deleted the Required Action that stated, “Place associated PORVs in manual control.” Finally, TSTF-247 added bracketed text to the Conditions and Required Actions in TS 3.4.11 to accommodate plants with three PORV’s and associated block valves

The justification for the changes was that the existing Conditions and Required Actions that provide appropriate compensatory actions for separate condition entry for each PORV also provide appropriate compensatory actions for separate condition entry for each block valve. Deletion of the Required Action that stated, “Place associated PORVs in manual control” was justified because if the block valves are not restored within 2 hours a plant shutdown is required and the PORVs will be needed for Low Temperature Over Pressure protection and therefore, the PORVs should not be placed in manual control.

This traveler preceded the current NRC practice of documenting approval in an SE. The change was adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.19.2 Proposed TS Changes and Variations

As described in LAR Section 2.17, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-247 changes to WBN, Units 1 and 2, TS 3.4.11 with no variations.

3.19.3 Staff Evaluation of Proposed TS Changes

The NRC staff finds that the proposed changes to WBN TSs are consistent with the STS and TSTF-247, Revision 0, and the pertinent portions of STS and TSTF-247 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

3.20 TSTF-309, Revision 2, "Revise Pressurizer PORV Actions to not require cycling of block valve when closed for isolation"

3.20.1 Description of TSTF

TSTF-309, Revision 2 (ML040620179), changed NUREG-1431 by adding NOTES above TS 3.4.11 Required Actions C.1 and F.1 stating certain Required Actions do not apply if the PORV block valve was declared inoperable solely as a result of complying with other Required Actions in TS 3.4.11.

The TSTF-309 changes were justified because the Required Actions for inoperable PORVs are adequate to address the condition where the sole reason for the block valve being declared inoperable is a result of power being removed to comply with other Required Actions.

This traveler preceded the current NRC practice of documenting approval in an SE. The change was approved by NRC staff in letter dated July 26, 1999 (ML19067A141) and adopted into Revision 2 of NUREG-1431 and every subsequent revision.

3.20.2 Proposed TS Changes and Variations

As described in LAR Section 2.18, and illustrated in Attachments 1, 2, 3 and 4, the licensee proposed to adopt the TSTF-309 changes to WBN, Units 1 and 2, TS 3.4.11 with no variations.

3.20.3 Staff Evaluation of Proposed TS Changes and Variations

The NRC staff finds that the proposed changes to WBN TSs are consistent with the STS and TSTF-309 and the pertinent portions of STS and TSTF-309 are applicable to WBN, Units 1 and 2. Therefore, the NRC staff finds that the proposed changes, as described in the LAR, are acceptable because the TSs, as modified, will continue to meet 10 CFR 50.36.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Tennessee State official was notified of the proposed issuance of the amendments on May 27, 2026. The State official had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendments relate to changes to inspection or surveillance requirements. The NRC staff has determined that any ground disturbance is limited to previously disturbed areas. Additionally, the NRC staff has determined that the amendments involve no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, no significant increase in individual or cumulative public or occupational radiation exposure, and no significant increase in the potential for or consequences from radiological accidents. Finally,

the NRC staff has determined that a categorical exclusion applies and that special circumstances under 10 CFR 51.22, “Categorical exclusions,” are not present that would preclude reliance on the categorical exclusion. Accordingly, this action meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(d)(1). Pursuant to 10 CFR 51.22, no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the action.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Date: June 25, 2026

SUBJECT: WATTS BAR NUCLEAR PLANT, UNITS 1 AND 2 - ISSUANCE OF AMENDMENT NOS. 181 AND 86 REGARDING REBASELINE OF TECHNICAL SPECIFICATION SECTIONS 3.4 AND 3.9 (EPID L-2025-LLA-0138) DATED JUNE 25, 2026

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