



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

June 1, 2026

Mr. Charles V. McFeaters
President and Chief Nuclear Officer
PSEG Nuclear LLC – N09
Hope Creek Generating Station
P.O. Box 236
Hancocks Bridge, NJ 08038

SUBJECT: HOPE CREEK GENERATING STATION – PROPOSED ALTERNATIVE
REQUEST VR-01, RELATED TO EXCESS FLOW CHECK VALVES TESTING
AND SCHEDULE (EPID L-2026-LLR-0005)

Dear Mr. McFeaters:

By letter dated January 6, 2026, PSEG Nuclear LLC (PSEG, the licensee) submitted several requests (including Alternative Request VR-01) to the U.S. Nuclear Regulatory Commission (NRC) for authorization of alternatives to specific inservice testing (IST) requirements in the 2022 Edition of the American Society of Mechanical Engineers (ASME) Operation and Maintenance of Nuclear Power Plants, Division 1, OM Code: Section IST (OM Code) as incorporated by reference in Title 10 of the *Code of Federal Regulations* (10 CFR), Section 50.55a, "Codes and standards," for the Hope Creek Generating Station (Hope Creek).

Specifically, pursuant to subparagraph (1) in paragraph (z), "Alternatives to codes and standards requirements," of Section 55a, "Codes and standards," in Part 50, "Domestic Licensing of Production and Utilization Facilities," "Energy," of 10 CFR 50.55a(z)(1), the licensee requested in Alternative Request VR-01 for the testing and schedule of excess flow check valves at Hope Creek on the basis that the proposed alternative will provide an acceptable level of quality and safety.

The NRC staff has reviewed the subject request and concludes, as set forth in the enclosed safety evaluation, that Alternative Request VR-01 for Hope Creek provides an acceptable level of quality and safety for IST activities for the specific valves within the scope of the request. Accordingly, the NRC staff concludes that the licensee has adequately addressed all of the regulatory requirements set forth in 10 CFR 50.55a(z)(1) for this proposed alternative. Therefore, pursuant to 10 CFR 50.55a(z)(1), the NRC staff authorizes Hope Creek Alternative Request VR-01 for the Fifth IST Interval for the specified valves within the scope of the request in lieu of the applicable IST requirements in the 2022 Edition of the ASME OM Code, as incorporated by reference in 10 CFR 50.55a, for the Code of Record interval, as defined in 10 CFR 50.55a(y), "Definitions," that implements the 2022 Edition of the ASME OM Code. Use of this alternative with other codes of record is not authorized.

All other ASME OM Code requirements as incorporated by reference in 10 CFR 50.55a for which relief or an alternative was not specifically requested, and granted or authorized (as appropriate), remain applicable.

C. McFeaters

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If you have any questions, please contact the Project Manager, Robert Kuntz, at 301-415-3733 or via email at robert.kuntz@nrc.gov.

Sincerely,

Undine S. Shoop, Acting Chief
Plant Licensing Branch I
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket No. 50-354

Enclosure:
Safety Evaluation

cc: Listserv



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

ALTERNATIVE REQUEST VR-01

ASME OM CODE, 2022 EDITION

HOPE CREEK GENERATING STATION

PSEG NUCLEAR

DOCKET NUMBER 50-354

EPID L-2026-LLR-0005

1.0 INTRODUCTION

By letter dated January 6, 2026 (Agencywide Documents Access and Management System Accession No. ML26006A183), PSEG Nuclear LLC (the licensee) submitted Alternative Request VR-01 to the U.S. Nuclear Regulatory Commission (NRC) for an alternative to specific inservice testing (IST) requirements in the 2022 Edition of the American Society of Mechanical Engineers (ASME) *Operation and Maintenance of Nuclear Power Plants*, Division 1, OM Code: Section IST (OM Code), as incorporated by reference in Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50, Section 50.55a, "Codes and standards," at Hope Creek Generating Station (Hope Creek), associated with the Fifth IST Interval Program.

Specifically, pursuant to subparagraph (1) in paragraph (z), "Alternatives to codes and standards requirements," of 10 CFR 50.55a, the licensee requested to implement Alternative Request VR-01 for the testing and schedule of excess flow check valves (EFCVs) at Hope Creek on the basis that the proposed alternative will provide an acceptable level of quality and safety. The Hope Creek Fifth IST Interval Program is scheduled to begin on December 21, 2026.

2.0 REGULATORY REQUIREMENTS

The NRC regulations in 10 CFR 50.55a(f)(4), "Inservice testing standards requirement for operating plants," state that throughout the service life of a boiling or pressurized water-cooled nuclear power facility, pumps and valves that are within the scope of the ASME OM Code must meet the IST requirements (except design and access provisions) set forth in the ASME OM Code and addenda that become effective subsequent to editions and addenda specified in 10 CFR 50.55a(f)(2) and (3) and that are incorporated by reference in 10 CFR 50.55a(a)(1)(iv), to the extent practical within the limitations of design, geometry, and materials of construction of the components.

The NRC regulations in 10 CFR 50.55a(z), *Alternatives to codes and standards requirements*, state:

Alternatives to the requirements of paragraphs (b) through (h) of this section or portions thereof may be used when authorized by the Director, Office of Nuclear Reactor Regulation. A proposed alternative must be submitted and authorized prior to implementation. The applicant or licensee must demonstrate that:

(1) *Acceptable level of quality and safety.* The proposed alternative would provide an acceptable level of quality and safety; or

(2) *Hardship without a compensating increase in quality and safety.* Compliance with the specified requirements of this section would result in hardship or unusual difficulty without a compensating increase in the level of quality and safety.

The NRC regulations in 10 CFR 50.55a(b)(3)(xi), OM condition: Valve Position Indication, state in part the following:

When implementing paragraph ISTC-3700, "Position Verification Testing," in the ASME OM Code, 2012 Edition through the latest edition and addenda of the ASME OM Code incorporated by reference in paragraph (a)(1)(iv) of this section, licensees must verify that valve operation is accurately indicated by supplementing valve position indicating lights with other indications, such as flow meters or other suitable instrumentation to provide assurance of proper obturator position for valves with remote position indication within the scope of Subsection ISTC including its mandatory appendices and their verification methods and frequencies.

3.0 TECHNICAL EVALUATION

3.1 Licensee's Alternative Request VR-01

Applicable Code Edition

The applicable Code of Record for the Fifth IST Interval Program at Hope Creek is the 2022 Edition of ASME OM Code as incorporated by reference in 10 CFR 50.55a.

ASME Code Components Affected

In its submittal, the licensee proposed alternative testing for the following EFCVs:

Component ID	System	ASME Class	Category
1ABXV-3666A	EFCV	1	C
1ABXV-3666B	EFCV	1	C
1ABXV-3666C	EFCV	1	C
1ABXV-3666D	EFCV	1	C
1ABXV-3667A	EFCV	1	C
1ABXV-3667B	EFCV	1	C

Component ID	System	ASME Class	Category
1ABXV-3667C	EFCV	1	C
1ABXV-3667D	EFCV	1	C
1ABXV-3668A	EFCV	1	C
1ABXV-3668B	EFCV	1	C
1ABXV-3668C	EFCV	1	C
1ABXV-3668D	EFCV	1	C
1ABXV-3669A	EFCV	1	C
1ABXV-3669B	EFCV	1	C
1ABXV-3669C	EFCV	1	C
1ABXV-3669D	EFCV	1	C
1BBXV-3621	EFCV	1	C
1BBXV-3725	EFCV	1	C
1BBXV-3726A	EFCV	1	C
1BBXV-3726B	EFCV	1	C
1BBXV-3727A	EFCV	1	C
1BBXV-3727B	EFCV	1	C
1BBXV-3728A	EFCV	1	C
1BBXV-3728B	EFCV	1	C
1BBXV-3729A	EFCV	1	C
1BBXV-3729B	EFCV	1	C
1BBXV-3730A	EFCV	1	C
1BBXV-3730B	EFCV	1	C
1BBXV-3731A	EFCV	1	C
1BBXV-3731B	EFCV	1	C
1BBXV-3732A	EFCV	1	C
1BBXV-3732B	EFCV	1	C
1BBXV-3732C	EFCV	1	C
1BBXV-3732D	EFCV	1	C
1BBXV-3732E	EFCV	1	C
1BBXV-3732F	EFCV	1	C
1BBXV-3732G	EFCV	1	C
1BBXV-3732H	EFCV	1	C
1BBXV-3732J	EFCV	1	C

Component ID	System	ASME Class	Category
1BBXV-3732K	EFCV	1	C
1BBXV-3732L	EFCV	1	C
1BBXV-3732M	EFCV	1	C
1BBXV-3732N	EFCV	1	C
1BBXV-3732P	EFCV	1	C
1BBXV-3732R	EFCV	1	C
1BBXV-3732S	EFCV	1	C
1BBXV-3732T	EFCV	1	C
1BBXV-3732U	EFCV	1	C
1BBXV-3732V	EFCV	1	C
1BBXV-3732W	EFCV	1	C
1BBXV-3734A	EFCV	1	C
1BBXV-3734B	EFCV	1	C
1BBXV-3734C	EFCV	1	C
1BBXV-3734D	EFCV	1	C
1BBXV-3737A	EFCV	1	C
1BBXV-3737B	EFCV	1	C
1BBXV-3738A	EFCV	1	C
1BBXV-3738B	EFCV	1	C
1BBXV-3783	EFCV	1	C
1BBXV-3785	EFCV	1	C
1BBXV-3787	EFCV	1	C
1BBXV-3789	EFCV	1	C
1BBXV-3801A	EFCV	1	C
1BBXV-3801B	EFCV	1	C
1BBXV-3801C	EFCV	1	C
1BBXV-3801D	EFCV	1	C
1BBXV-3802A	EFCV	1	C
1BBXV-3802B	EFCV	1	C
1BBXV-3802C	EFCV	1	C
1BBXV-3802D	EFCV	1	C
1BBXV-3803A	EFCV	1	C
1BBXV-3803B	EFCV	1	C

Component ID	System	ASME Class	Category
1BBXV-3803C	EFCV	1	C
1BBXV-3803D	EFCV	1	C
1BBXV-3804A	EFCV	1	C
1BBXV-3804B	EFCV	1	C
1BBXV-3804C	EFCV	1	C
1BBXV-3804D	EFCV	1	C
1BBXV-3820	EFCV	1	C
1BBXV-3821	EFCV	1	C
1BBXV-3826	EFCV	1	C
1BBXV-3827	EFCV	1	C
1BCXV-4411A	EFCV	1	C
1BCXV-4411B	EFCV	1	C
1BCXV-4411C	EFCV	1	C
1BCXV-4411D	EFCV	1	C
1BCXV-4429A	EFCV	1	C
1BCXV-4429B	EFCV	1	C
1BCXV-4429C	EFCV	1	C
1BCXV-4429D	EFCV	1	C
1BEXV-F018A	EFCV	1	C
1BEXV-F018B	EFCV	1	C
1BGXV-3882	EFCV	1	C
1BGXV-3884A	EFCV	1	C
1BGXV-3884B	EFCV	1	C
1BGXV-3884C	EFCV	1	C
1BGXV-3884D	EFCV	1	C
1FCXV-4150A	EFCV	1	C
1FCXV-4150B	EFCV	1	C
1FCXV-4150C	EFCV	1	C
1FCXV-4150D	EFCV	1	C
1FDXV-4800A	EFCV	1	C
1FDXV-4800B	EFCV	1	C
1FDXV-4800C	EFCV	1	C
1FDXV-4800D	EFCV	1	C

Applicable ASME OM Code Requirements

The IST requirements in the ASME OM Code, as incorporated by reference in 10 CFR 50.55a, related to this alternative request are as follows:

- ASME OM Code, Subsection ISTC, “Inservice Testing of Valves in Water-Cooled Reactor Nuclear Power Plants,” paragraph ISTC-3510, “Exercising Test Frequency,” states, in part, that “Active Category A, Category B, and Category C check valves shall be exercised nominally every 3 months, except as provided by ISTC-3520; ISTC-3540; ISTC-3550; ISTC-3570; ISTC-5221; ISTC-5222; and Division 1, Mandatory Appendix III.”
- Paragraph ISTC-3522, “Category C Check Valves,” subparagraph (a) states, in part, “During operation at power, each check valve shall be exercised or examined in a manner that verifies obturator travel by using the methods in ISTC-5221.” Paragraph ISTC-3522, subparagraph (c) states “If exercising is not practicable during operation at power and cold shutdown outages, it shall be performed during refueling outages.”
- Paragraph ISTC-3700, “Position Verification Testing,” states, in part, that “Valves with remote position indicators shall be observed locally at least once every 2 yr [years] to verify that valve operation is accurately indicated. Where practicable, this local observation should be supplemented by other indications such as use of flow meters or other suitable instrumentation to verify obturator position.”

Licensee’s Proposed Alternative

As an alternative to testing all EFCVs during a single refueling outage, the licensee proposes that EFCVs will be tested on a representative sample basis at the frequency in accordance with the Hope Creek Surveillance Frequency Control Program (SFCP) as stated in Technical Specification (TS) Surveillance Requirement (SR) 3.6.1.3.9.

The licensee stated that a representative sample of EFCVs would be tested at Hope Creek every refueling outage such that all valves (except for EFCV 1BBXV-3649, (penetration J5C – reactor vessel head seal leak detection), as exempted by a TS note) would be tested once in 10 years per the Hope Creek SFCP. The representative sampling would encompass approximately equal numbers every refueling outage with all EFCVs being tested at least once within 10 years (nominal).

Licensee’s Basis for Use

Industry experience, as documented in General Electric (GE) Topical Report NEDO-32977-A, “DRF B21-00658-01, Class 1, Excess Flow Check Valve Testing Relaxation,” (ML003729011) indicates that EFCVs have a very low failure rate. A review of the maintenance history for Hope Creek EFCVs has shown that they have been extremely reliable over the life of the plant, showing less than 1 percent failure rate associated with testing of these valves. Examples of causes for the failures include alarm problems, position indication (limit switch adjustment), and bent instrument tubing. Review of surveillance test history shows no evidence of time-based failure mechanisms or common mode failures associated with EFCVs. The test experience at Hope Creek is consistent with the findings in NEDO-32977-A, which indicates similarly that many reported test failures at other plants were related to test methodologies and not actual EFCV failures. Thus, the licensee asserts

that the EFCVs at Hope Creek, consistent with the industry, have exhibited a high degree of reliability, availability, and provide an acceptable level of quality and safety.

The EFCVs have position indication at local panels in the reactor building. Check valve remote position indication is excluded from Regulatory Guide 1.97, "Criteria for Accident Monitoring Instrumentation for Nuclear Power Plants," (ML060750525) as a required parameter for evaluating containment isolation. The remote position indication will be verified in the closed direction at the same frequency as the exercise test, which will be performed at the frequency prescribed in the SFCP per SR 3.6.1.3.9. After the close position test, the valve will be reset, and the remote open position indication will be verified. The NRC regulations in 10 CFR 50.55a(b)(3)(xi) require obturator verification at the same frequency as ASME OM Code, Subsection ISTC, paragraph ISTC-3700 (every 2 years). Obturator verification for EFCVs equates to the position of the poppet assembly which is detected by switches to change the status of indicating lights. The current methodology used for functional testing of the EFCVs will be credited to meet the obturator verification requirements. However, this obturator verification for the EFCVs will be performed on the same sampling frequency as the functional testing prescribed in SR 3.6.1.3.9. Although inadvertent actuation of an EFCV during operation is highly unlikely due to the spring poppet design, corrective action is initiated for any EFCVs with abnormal position indication displays and repairs are scheduled for the next refueling outage.

In accordance with the IST Program Plan, EFCV failures will be evaluated to determine if additional testing in that test interval is warranted to ensure overall reliability. Adverse trends and EFCV performance are identified and dispositioned in the Hope Creek corrective action program.

Hope Creek SR 3.6.1.3.9 requires demonstration that a representative sample of reactor instrumentation line EFCVs are tested to demonstrate that the valve actuates to check flow on a simulated instrument line break. This SR provides assurance that the instrument line EFCVs will perform so that the predicted radiological consequences will not be exceeded during a postulated instrument line break event as evaluated in the Hope Creek Updated Final Safety Analysis Report (UFSAR). The surveillance frequency is based on operating experience, equipment reliability, and plant risk, and is controlled under the SFCP (TS 5.5.13). Operating experience has demonstrated that these components are highly reliable and that failures to isolate are very infrequent. Therefore, the licensee considers that testing of a representative sample is acceptable from a reliability standpoint.

In summary, considering the extremely low failure rate along with personnel and plant safety concerns to perform testing, the licensee asserts that the proposed alternative to perform EFCV testing on a sampling basis will continue to provide assurance of the EFCVs' operational readiness and will provide an acceptable level of quality and safety pursuant to 10 CFR 50.55a(z)(1).

Licensee's Reason for Request

Pursuant to 10 CFR 50.55a(z)(1), the licensee proposes an alternative to the requirements of ASME OM Code, Subsection ISTC, paragraphs ISTC-3522(c) and ISTC-3700, as well as 10 CFR 50.55a(b)(3)(xi) for the subject valves. The basis of this request is that the proposed alternative would provide an acceptable level of quality and safety.

The ASME OM Code requires check valves to be exercised quarterly during plant operation, or if valve exercising is not practicable during plant operation and cold shutdown, it shall be performed during refueling outages. The OM Code also requires verification of valve position indication at least once every 2 years. The NRC regulations in 10 CFR 50.55a(b)(3)(xi) require supplementing the ISTC-3700 testing with other indications to ensure valve position indicating lights accurately reflect valve operation.

The major components of EFCVs are the poppet and spring. The spring holds the poppet open under static conditions. The valve will close upon sufficient differential pressure across the poppet. Functional testing of the valve is accomplished by venting the instrument side of the valve. The resultant increase in flow imposes a differential pressure across the poppet, which compresses the spring and decreases flow through the valve.

The testing described above requires removal of the associated instrument or instruments from service. Since these instruments are in use during plant operation, removal of any of these instruments from service may cause a spurious signal, which could result in a plant trip or an unnecessary challenge to safety systems. Additionally, process fluid will be contaminated to some degree, requiring special measures to collect flow from the vented instrument side and also contribute to an increase in personnel radiation exposure.

The EFCVs are classified as ASME OM Code Category C and are also containment isolation valves. However, these valves are excluded from 10 CFR Part 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors," Type C leak rate testing due to the size of the instrument lines and upstream orifices. Therefore, they have no safety-related seat leakage criterion.

Alternative Request VR-01 lists Hope Creek Generating Station -NRC Authorization of Alternative regarding EFCVs for the Fourth 10-year IST interval, dated December 20, 2016 (ML16343A057), as a precedent for the current request. The previous authorization includes Hope Creek licensee responses to NRC requests for additional information (RAIs) dated June 10, 2016 (ML16162A709), and the licensee verbally confirmed that these RAI responses continue to be valid and applicable to the current request.

3.2 NRC Staff Evaluation

At nuclear power plants, EFCVs are installed on instrument lines to limit the release of fluid in the event of an instrument line break. Examples of EFCV installations include reactor pressure vessel level and pressure instrumentation, main steam line flow instrumentation, recirculation pump suction pressure, and reactor core isolation cooling (RCIC) steam line flow instrumentation. EFCVs are not required to close in response to a containment isolation signal, and are not required to operate under post-loss-of-coolant accident (post-LOCA) conditions.

The 2022 Edition of the ASME OM Code, as incorporated by reference in 10 CFR 50.55a, requires EFCVs to be tested in accordance with Subsection ISTC, paragraph ISTC-3510 which states, in part, that active Category A, Category B, and Category C check valves shall be exercised nominally every 3 months. The ASME OM Code recognizes that some valves cannot be tested at this frequency. Deferral of this requirement is allowed by paragraph ISTC-3522(c), which states if exercising is not practical during operation at power and cold shutdowns, it shall be performed during refueling outages.

The EFCVs listed Table 1 in Alternative Request VR-01 cannot be exercised during normal operation because closing these valves would isolate instrumentation required for power operation. These valves can only be tested during a refueling outage. The licensee has proposed an alternative to the ASME OM Code required test interval to provide for a "representative sample" of EFCVs to be tested every refueling outage. The "representative sample" is based on an approximate equal number of EFCVs (about 20 percent) being tested each refueling outage such that each individual valve is tested approximately every 10 years.

The licensee's justification for Alternative Request VR-01 is based on GE Topical Report NEDO-32977-A. The topical report provided (1) an estimate of steam release frequency (into the reactor building) due to a break in an instrument line concurrent with an EFCV failure to close, and (2) an assessment of the radiological consequences of such a release. The NRC staff reviewed the GE topical report and issued its evaluation on March 14, 2000 (ML003691722). In its evaluation, the NRC staff concluded that the test interval could be extended up to a maximum of 10 years. In conjunction with this conclusion, the NRC staff noted that each licensee that adopts the relaxed test interval program for EFCVs must have a failure feedback mechanism and corrective action program to ensure that EFCV performance continues to be bounded by the topical report results. Also, each licensee is required to perform (1) a plant-specific radiological dose assessment, (2) an EFCV failure analysis, and (3) a release frequency analysis to confirm that any EFCV failures are bounded by the generic analyses of the topical report.

The licensee stated that a review of the maintenance history for Hope Creek EFCVs has shown that those valves have been extremely reliable over the life of the plant, showing less than 1 percent failure rate associated with testing of these valves. Examples of causes for the failures include alarm problems, position indication (limit switch adjustment), and bent instrument tubing. Review of surveillance test history shows no evidence of time-based failure mechanisms or common mode failures associated with EFCVs. The test experience at Hope Creek is consistent with the findings in GE Topical Report NEDO-32977-A, which indicates similarly that many reported test failures at other plants were related to test methodologies and not actual EFCV failures. Thus, the EFCVs at Hope Creek, consistent with the industry, have exhibited a high degree of reliability and availability.

The licensee stated that the EFCVs have position indication at local panels in the reactor building. Check valve remote position indication is excluded from Regulatory Guide 1.97 as a required parameter for evaluating containment isolation. The remote position indication will be verified in the closed direction at the same frequency as the exercise test, which will be performed at the frequency prescribed in the Hope Creek SFCP per TS SR 3.6.1.3.9. After the close position test, the valve will be reset, and the remote open position indication will be verified. The NRC regulations in 10 CFR 50.55a(b)(3)(xi) require obturator verification at the same frequency as ASME OM Code, Subsection ISTC, paragraph ISTC-3700, which is every 2 years. Obturator verification for EFCVs equates to the position of the poppet assembly which is detected by switches to change the status of indicating lights. The current methodology used for functional testing of the EFCVs will be credited to meet the obturator verification requirements. However, this obturator verification for the EFCVs will be performed on the same sampling frequency as the functional testing prescribed in TS SR 3.6.1.3.9. Although inadvertent actuation of an EFCV during operation is highly unlikely due to the spring poppet design, corrective action is initiated for any EFCVs with abnormal position indication displays, and repairs are scheduled for the next refueling outage.

The Hope Creek TS SR 3.6.1.3.9 specifies the following requirement: "Verify a representative sample of reactor instrumentation line EFCVs actuates to the isolation position on an actual or simulated instrument line break signal." The frequency is specified to be in accordance with the SFCP. The NRC staff finds that the changes to the frequencies specified in the SFCP are made in accordance with Nuclear Energy Institute (NEI) 04-10, "Risk Informed Method for Control of Surveillance Frequencies," Revision 1. The NRC staff issued a safety evaluation describing its review of NEI 04-10 (Revision 1) on September 19, 2007 (ML072570267). In its evaluation, the NRC staff found that the methodology is acceptable, with conditions, for a licensee to amend its TS to establish an SFCP.

In summary, the NRC staff reviewed the licensee's proposal in Alternative Request VR-01 for its applicability to GE Topical Report NEDO-32977-A, and conformance with the NRC staff's guidance regarding radiological dose assessment, EFCV failure rate and release frequency, and the proposed failure feedback mechanism and corrective action program at Hope Creek. Based on its evaluation, the NRC staff has determined that the radiological consequences of an EFCV failure are sufficiently low and acceptable, and that the alternative testing in conjunction with the Hope Creek corrective action program and the SFCP provides a high degree of valve reliability and operability.

Based on the above, the NRC staff has determined that Alternative Request VR-01 may be authorized pursuant to 10 CFR 50.55a(z)(1) on the basis that the proposed alternative provides an acceptable level of quality and safety for the EFCVs within the scope of the request at Hope Creek.

4.0 ENVIRONMENTAL CONSIDERATION

This authorized alternative relates to actions under 10 CFR 50.55a. The NRC staff has determined that a categorical exclusion applies and that special circumstances are not present that would preclude reliance on the categorical exclusion. Accordingly, this action meets the eligibility criteria for categorical exclusion set forth in paragraph (a)(16) of 10 CFR 51.22, "Categorical exclusions." Pursuant to 10 CFR 51.22, no environmental impact statement or environmental assessment need be prepared in connection with the action.

5.0 CONCLUSION

As set forth above, the NRC staff determined that Alternative Request VR-01 for Hope Creek provides an acceptable level of quality and safety for IST activities for the specific valves within the scope of the request. Accordingly, the NRC staff concludes that the licensee has adequately addressed all of the regulatory requirements set forth in 10 CFR 50.55a(z)(1) for this proposed alternative. Therefore, pursuant to 10 CFR 50.55a(z)(1), the NRC staff authorizes Hope Creek Alternative Request VR-01 for the Fifth IST Interval for the specified valves within the scope of the request in lieu of the applicable IST requirements in the 2022 Edition of the ASME OM Code, as incorporated by reference in 10 CFR 50.55a, for the Code of Record interval, as defined in 10 CFR 50.55a(y), "Definitions," that implements the 2022 Edition of the ASME OM Code. Use of this alternative with other codes of record is not authorized.

All other ASME OM Code requirements as incorporated by reference in 10 CFR 50.55a for which relief or an alternative was not specifically requested, and granted or authorized (as appropriate), remain applicable.

Date: June 1, 2026

HOPE CREEK GENERATING STATION – PROPOSED ALTERNATIVE REQUEST VR-01,
RELATED TO EXCESS FLOW CHECK VALVES TESTING AND SCHEDULE (EPID L-2026-
LLR-0005) DATED JUNE 1, 2026

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Package: ML26138A001

Relief Request: ML26138A003

eConcurrence Case: 20260518-00001