# Public Meeting to Discuss Licensing Requests to Implement 24-Month Fuel Cycles for Millstone Power Station Unit 3

## May 20, 2026



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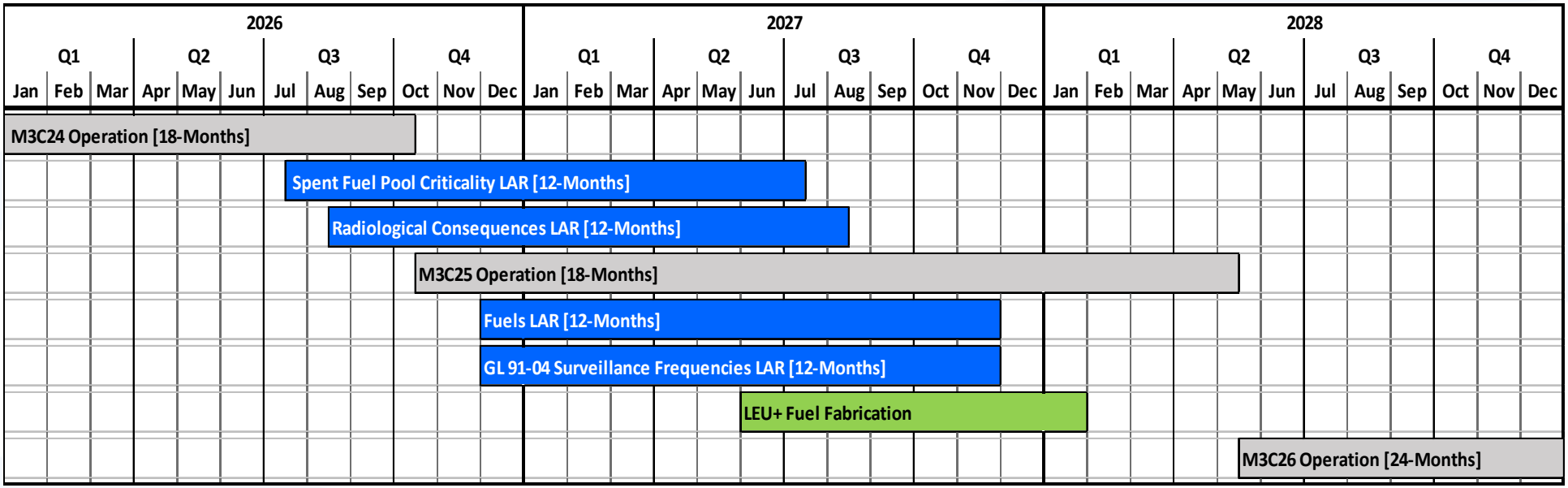
# Agenda

- Introductions and Overview of Agenda
- Planned Schedule
- Submittal Strategy
- Scope and Content for SFP Criticality LAR
- Scope and Content for Radiological Consequences LAR
- Scope and Content for Fuels LAR
- Scope and Content for TS SR LAR
- Fee Waiver

# Objective

- Transition Millstone Power Station to 24M fuel cycles
- Increases in MPS3 fuel enrichment and burnup are required to economically support 24M operation, resulting in the first full batches of HBU/LEU+ fuel
- The following LARs will be submitted to the NRC to support the transition to 24M cycles at MPS3:
  - SFP Criticality Analysis
  - Radiological Consequence Analysis
  - Fuels related analyses, TS revisions, and methodologies
  - Technical Specification surveillance extensions

# Planned Schedule




DENC intends to provide focused topic area submittals.



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# Submittal Strategy

1) Submit LAR for SFP Criticality:
   - Pre-submittal meeting held in July 2025.
   - Submittal scheduled for July 2026, requesting a 12-month review.
2) Submit LAR for Radiological Consequences:
   - Pre-submittal meeting held in September 2025.
   - Submittal scheduled for Q3 2026, requesting a 12-month review.
3) Submit LAR for Fuels:
   - Submittal scheduled for Q4 2026, requesting a 12-month review.
4) Submit LAR for updated TS SRs reflecting 24-month fuel cycles:
   - Submittal scheduled for Q4 2026, requesting a 12-month review.

The first 24-month fuel cycle for MPS3 is scheduled to begin in the Spring of 2028.

# SFP Criticality LAR - Overview

- Driver for LAR:  24 Month Fuel Cycles requiring LEU+/HBU fuel
- Maximum U-235 Enrichment:  6.5 wt%
- Maximum Credited Burnup:  <61 GWD/MTU
- Relevant Fuel Assembly Changes (GAIA-AFM):
  - Slightly larger fuel pellet outer diameter
  - Slightly denser fuel
  - Updated zirconium cladding material
- Storage Rack Changes:  None
- Identified precedent for HBU/LEU+ applications –
  VEGP Units 1 and 2, four LEU+ fuel pins per LTA, approved on August 1, 2023
  (ADAMS Accession No. ML23093A028)
- Schedule:
  - Submit LAR – July 2026
  - NRC Review Complete – July 2027 (12-month review)
  - 24-month Initial Startup – May 2028

# SFP Criticality LAR – Overview

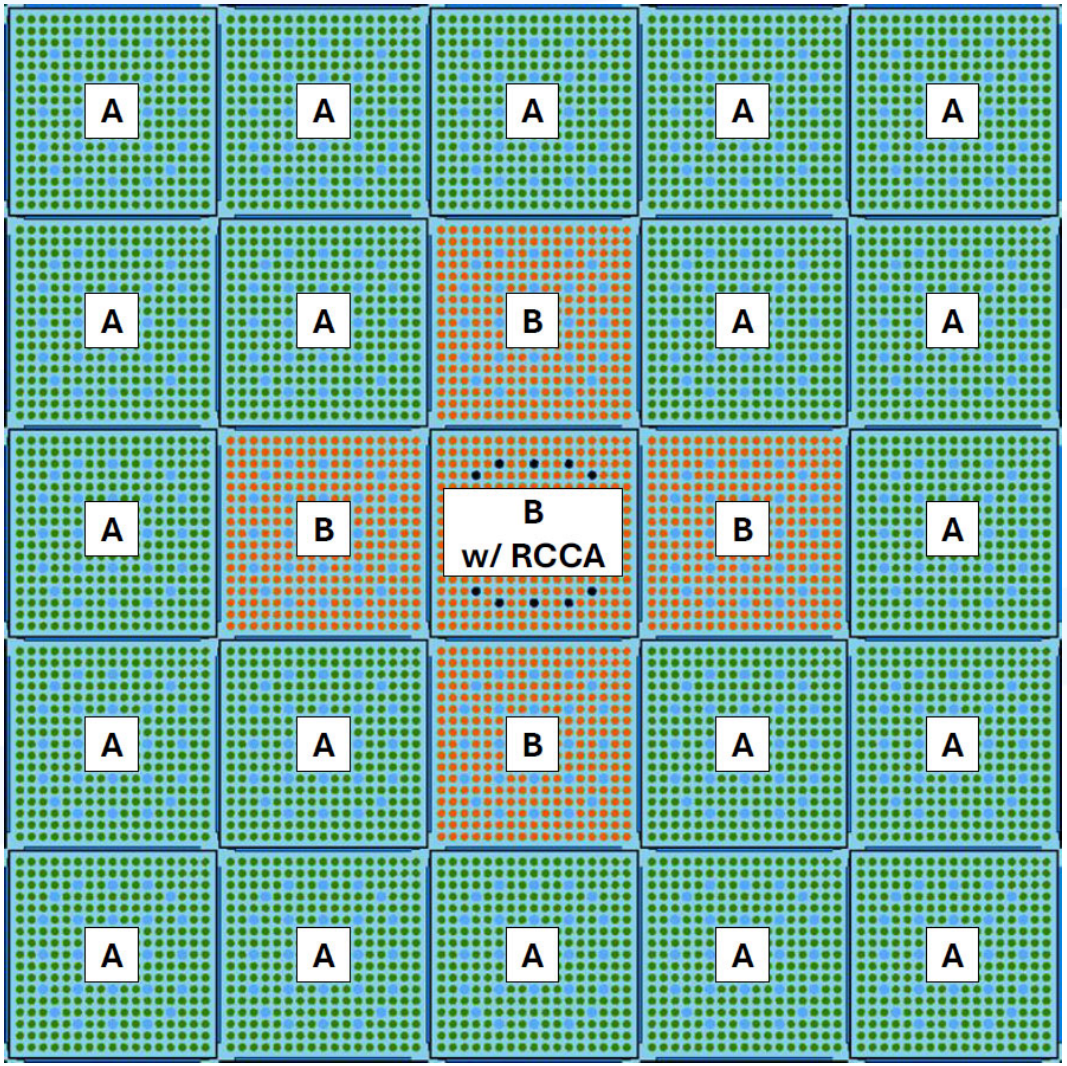
- New Fuel Storage Racks:
  - 12x8 array of cells
  - Cell pitch of 22-1/8” North-South, 24-1/16” East-West
  - Dry borated aluminum sheets about every other row
- SFP, Region 1:
  - Boral, flux trap rack design
  - Requires no burnup
  - Purpose:  Store all fuel including fresh and fuel that is reused in the core
- SFP, Region 2:
  - Boral, non-flux trap or “egg-crate” rack design
  - Requires some burnup
  - Purpose:  Store all discharge fuel
- SFP, Region 3:
  - Uncredited Boraflex, flux trap rack design
  - Requires the most burnup
  - Purpose:  Store the most discharge fuel

# SFP Criticality LAR
# Changes from 1st Pre-Submittal Meeting

- Requesting a 12-Month Review Time.
  - Aligning with regulatory drivers and industry peers.
- Removed option to load IFBA and Gadolinia BP in same assembly.
  - Incorporated feedback from 1st pre-submittal meeting.
- Added minimum BP requirement for Region 1N to make it consistent with the NFSR requirements (not required to meet k-eff limit).
  - Incorporated NRC staff's comments to make requirements more consistent across storage regions.
- Require Core Designers to use the analyzed BP pin pattern when loading the minimum number of BP pins.
  - Originally planned included a bias for BP pattern flexibility – this has been removed.
- Additional Region 2 control rod credit loading scheme.
  - More details on this in a follow up slide.

# SFP Criticality LAR
# Region 2, Rule Set 2 Control Rod Credit

- Rule Set 2 is a 5x5 fuel assembly storage pattern
- Purpose: Store a small number of higher enriched but lesser burned assemblies
- Reactivity Categories
  - "A" – Requires More Burnup
  - "B" – Requires Less Burnup
  - Center Reactivity Category B assembly requires an RCCA



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# SFP Criticality LAR – TS Changes

TS 3/4.9.13: Defines regions and requirements

- Revise to reflect new analysis
- Draft Surveillance Requirements on following pages

Figures 3.9-1: Defines Region 1 subregions

- Revise for new Region 1 subregions

Figures 3.9-2, 3.9-3, 3.9-7: Define storage patterns

- New storage patterns for Region 2 Rule Sets 2 and 3, and Region 3B

Figures 3.9-4, 3.9-5, 3.9-6, and 3.9-8: Define burnup requirements

- New curves for Region 2 Reactivity Categories and for Region 3B
- Maintain current curve for Region 3A

TS Bases 3/4.9.13: Describes regions and requirements

- Revise to reflect new analysis and describe transition details

TS 5.6.1: Description of New Fuel and Spent Fuel and requirements

- Add minimum required BP for the New Fuel Storage Racks description

# SFP Criticality LAR TS 4.9.13

**Insert #4 to TS 4.9.13 Spent Fuel Pool – Storage, Surveillance Requirements**

Region 1

4.9.13.1 Ensure that all fuel assemblies to be placed into Region 1N are stored in the correct rack module and cell consistent with the Rack Modules and Fuel Storage Pattern specified in Figure 3.9-1 AND ensure the fuel assemblies meet one of the following three criteria by checking the fuel assembly's storage location and design documentation:

i. initial enrichment less than or equal to 5.00 wt% U-235

ii. initial enrichment less than or equal to 6.00 wt% U-235 and containing:
- a minimum of 16 IFBA rods, or
- a minimum of 4 Gadolinia rods

iii. initial enrichment less than or equal to 6.50 wt% U-235 and containing:
- a minimum of 32 IFBA rods, or
- a minimum of 8 Gadolinia rods

4.9.13.2 Ensure that all fuel assemblies to be placed into Region 1S are stored in the correct rack module consistent with the Rack Modules and Fuel Storage Pattern specified in Figure 3.9-1 AND ensure the fuel assemblies meet one of the following two criteria by checking the fuel assembly's storage location and design documentation:

i. initial enrichment less than or equal to 4.70 wt% U-235

ii. initial enrichment less than or equal to 6.50 wt% U-235 and containing:
- a minimum of 48 IFBA rods, or
- a minimum of 12 Gadolinia rods

Region 2

4.9.13.3 Ensure that all fuel assemblies to be stored in Region 2 under Rule Set 1 constraints are Reactivity Category A assemblies. Reactivity Category A assemblies have enrichment, burnup, and decay time values in the acceptable region as defined in Figure 3.9-4. Transitioning to other Rule Set configurations is allowed, but the limits of the two involved Rule Sets must both be simultaneously satisfied to transition.

4.9.13.4 Ensure that all fuel assemblies to be stored in Region 2 under Rule Set 2 constraints meet the requirements defined in Figure 3.9-2. Each assembly in the 5x5 block must have an acceptable storage location; contain an RCCA if required; and acceptable burnup, enrichment, and decay time in accordance with Figure 3.9-4 or Figure 3.9-5 for Reactivity Category A and B fuel, respectively. Transitioning to other Rule Set configurations is allowed, but the limits of the two involved Rule Sets must both be simultaneously satisfied to transition.

# SFP Criticality LAR TS 4.9.13

**Insert #4 to TS 4.9.13 Spent Fuel Pool – Storage, Surveillance Requirements (continued)**

4.9.13.5 Ensure that all fuel assemblies to be stored in Region 2 under Rule Set 3 constraints meet the requirements defined in Figure 3.9-3. This includes each assembly's correct location in the 3x3 block; the presence of an RCCA in the Reactivity Category C assembly; acceptable loading of IFBA or Gadolinia rods for the Reactivity Category C assembly; and acceptable burnup, enrichment, and decay time in accordance with Figure 3.9-4 for the Reactivity Category A assemblies. Transitioning to other Rule Set configurations is allowed, but the limits of the two involved Rule Sets must both be simultaneously satisfied to transition.

Region 3

4.9.13.6 Ensure that all fuel assemblies to be stored in Region 3A rack modules have been exclusively depleted in or before Cycle 25 and are within the enrichment, burnup, decay time limits of Figure 3.9-6 by checking the fuel assembly's design, burnup, and decay time documentation. This rack module requires no empty cells. Transitioning whole Region 3A rack modules to and from Region 3B is allowed, but the limits of Region 3A and 3B must both be simultaneously satisfied to transition.

4.9.13.7 Ensure that all fuel assemblies to be stored in Region 3B rack modules meet the requirements defined in Figure 3.9-7. This includes empty cells in the correct locations of the rack module and acceptable burnup, enrichment, and decay time in accordance with Figure 3.9-8. This rack module has no restrictions on when the assemblies were depleted. Transitioning whole Region 3B rack modules to and from Region 3A is allowed, but the limits of Region 3A and 3B must both be simultaneously satisfied to transition.

# SFP Criticality LAR TS Bases 4.9.13

## Region 2 Rule Set Transition Description

To transition between two different Region 2 Rule Set configurations, the limits of the two involved Rule Sets must both be simultaneously satisfied to transition. This includes, but is not limited to, the following scenarios. Note that empty cells are acceptable in place of fuel assemblies for all Rule Set requirements.

- When transitioning from Rule Set 1 to Rule Set 2, all assemblies must initially be Reactivity Category A assemblies which are generally more restrictive than Reactivity Category B. Then an RCCA is placed in the middle of the 5x5 block to satisfy the requirements of both Rule Sets 1 and 2 simultaneously. Then assemblies only qualifying for Reactivity Category B can be moved into the locations specified in Figure 3.9-2 in compliance with SR 4.9.13.4.
- When transitioning from Rule Set 1 to Rule Set 2, an empty cell is made in the middle of a 5x5 block of Reactivity Category A assemblies to simultaneously satisfy the requirements of both Rule Sets. Then assemblies only qualifying for Reactivity Category B can be moved into the locations specified in Figure 3.9-2 with the center assembly already containing an RCCA before it is placed in Region 2 in compliance with SR 4.9.13.4.
- When transitioning between Rule Set 1 and Rule Set 3, an empty cell is made in the middle of a 3x3 block to simultaneously satisfy the requirements of both Rule Sets. Then a Reactivity Category C assembly already containing an RCCA can be placed into the empty center cell under Rule Set 3 in compliance with SR 4.9.13.5.

## Region 3 Sub-Region Transition Description

To transition between Region 3A and 3B, the limits of Region 3A and 3B must both be simultaneously satisfied to transition. This includes, but is not limited to, the following scenario. Note that empty cells are acceptable in place of fuel assemblies for both Region 3A and 3B requirements.

- When transitioning from Region 3A to 3B, all assemblies in the rack module must initially meet Region 3A's burnup requirement (Region 3A requires more burnup than Region 3B), be burned exclusively in or before Cycle 25, and empty cells must be made in the locations defined in Figure 3.9-7. After this transition point, assemblies can be stored in the rack module in compliance with SR 4.9.13.7.

# SFP Criticality LAR
# 10 CFR 50.68 Exemption Request

- 10 CFR 50.68(b)(7): "The maximum nominal U-235 enrichment of the fresh fuel assemblies is limited to five (5.0) percent by weight."

- NRC Rule Making change process in progress

- Exemption Request Outline:
  - Demonstrate the three criteria specified by 10 CFR 50.12 are met.
  - Utilizing the 50.68 exemption request for 2022 VEGP Units 1 and 2 as a framework.

# Radiological Consequences LAR

- Driver for LAR:  24 Month Fuel Cycles, Outside Applicability of RG 1.183, Revision 0.
- Maximum U-235 Enrichment: 6.5 wt%
- Peak Rod Average Burnup: Consistent with Framatome's Increased Burnup Topical Report (ANP-10358P-A).
- Schedule:
  - Submit LAR – Q3 2026
  - 12-month NRC Review Requested
  - 24-month Fuel Cycle Initial Startup – May 2028

- No identified precedent for LEU+ applications

# Radiological Consequences LAR Updates from 1st Pre-Submittal Meeting

- Requesting a 12-Month Review Time (not a change)
  - Aligning with regulatory drivers and industry peers.
- Follow-up Technical Basis Approach:
  - Non-LOCA gap fraction values.
    - Without tech basis document, will rely on DG-1425 for numerical values.
  - Natural Deposition modeling in Containment.
    - SAND2026-15948 for aerosol removal coefficients (plan to use the median recommended values).
  - Equipment Qualification
    - NRC Memorandum from March 5, 2026 (ML26057A158).
    - Technical basis for continued resolution applicability of GSI-187.

# Radiological Consequences LAR Updates from 1st Pre-Submittal Meeting

- Exemption to 10 CFR 50.67(2)(b)(iii) and GDC 19:
  - Requesting Control Room Acceptance Criterion increase from 5 rem TEDE to 10 rem TEDE.
  - Leverage NRC White Paper on CR Design Criteria and Radiological Health Effects (ML23027A059).
  - Anticipated Draft Rulemaking Effort for Public Comment will also specify acceptance criterion increase.
  - Provide supplement to withdraw exemption request after rulemaking is implemented.

# Radiological Consequences LAR Updates from 1st Pre-Submittal Meeting

- Proposal for RG conformance table content:
  - Include DG-1425 conformance table in initial LAR content.
  - Anticipate RG 1.183, Rev. 2 issuance during NRC review of LAR.
  - Dominion suggests providing supplemental conformance with differences between DG-1425 and RG 1.183, Revision 2 once released.

# Radiological Consequences LAR High Level Content

- Follow NRC presentation from September 2025
  - Summary of Radiological Consequence Analyses.
  - DG-1425 Conformance Table.
  - LAR Inputs, Changes from CLB, Basis of Change.
    - Last NRC review and approval was SPU circa 2007.
  - RADTRAD-NAI files will be provided.
  - Summary of margins and defense-in-depth.
    - Comparison of analyzed versus tested results (containment leakage, ECCS leakage, CR inleakage).
  - Exemption request basis for 10 CFR 50.67(2)(b)(iii).
    - Requesting 10 rem TEDE acceptance criterion for Control Room.

# Radiological Consequence LAR TS Changes (Anticipated)

TS Definition 1.10: Dose Equivalent I-131:

- Clarify Dose Conversion Factors Set

TS Surveillance Requirement 4.6.6.2.2: Secondary Containment:

- Relaxation of drawdown time for SLCRS

TS LCO 3.7.7: Control Room Emergency Ventilation System:

- Relaxation of applicability for movement of Recently Irradiated Fuel (RIF) Assemblies

TS LCO 3.9.3 and SR 4.9.3: Decay Time:

- Relocation of time to Technical Requirements Manual, consistent with NUREG-1431.

# Radiological Consequence LAR TS Changes (Anticipated)

DOSE EQUIVALENT I-131

1.10 DOSE EQUIVALENT I-131 shall be that concentration of I-131 (microCurie/gram) that alone would produce the same dose when inhaled as the combined activities of iodine isotopes I-131, I-132, I-133, I-134, and I-135 actually present. The determination of DOSE EQUIVALENT I-131 shall be performed using ~~Committed Dose Equivalent (CDE) or~~ Committed Effective Dose Equivalent (CEDE) dose conversion factors from Table 2.1 of EPA Federal Guidance Report No. 11 (FGR 11), "Limiting Values of Radionuclide Intake and Air Concentration and Dose Conversion Factors for Inhalation, Submersion and Ingestion."

# Radiological Consequence LAR TS Changes (Anticipated)

3.6.6.2 Secondary Containment shall be OPERABLE.

APPLICABILITY: MODES 1, 2, 3, and 4.

ACTION:

With Secondary Containment inoperable, restore Secondary Containment to OPERABLE status within 24 hours or be in at least HOT STANDBY within the next 6 hours and in COLD SHUTDOWN within the following 30 hours.

SURVEILLANCE REQUIREMENT

4.6.6.2.1 OPERABILITY of Secondary Containment shall be demonstrated at the frequency specified in the Surveillance Frequency Control Program by verifying that each door in each access opening is closed except when the access opening is being used for normal transit entry and exit.

4.6.6.2.2 At the frequency specified in the Surveillance Frequency Control Program, verify each Supplementary Leak Collection and Release System produces a negative pressure of greater than or equal to 0.4 inch water gauge in the Auxiliary Building at 24'-6" elevation within ~~120 seconds~~ 1 hour after a start signal.

# Fuels LAR – Overview

- Driver for LAR:  HBU and LEU+ supporting 24 Month Fuel Cycles
  - Maximum U-235 Enrichment:  6.5 wt%
  - Peak Rod Average Burnup: consistent with Framatome's Increased Burnup Topical Report (ANP-10358P-A)
- Purpose of the Fuels LAR is to submit remaining fuels-related analyses, method changes, and TS changes required to enable 24M fuel cycles.
- Schedule:
  - Submit LAR – Anticipated Q4 2026
  - NRC approval – Q4 2027 (12-month review)
  - 24-month Initial Startup – May 2028
- No identified precedent for HBU/LEU+ applications

# Fuels LAR – Anticipated Elements

- Address MPS3 specific considerations impacted by burnup above 62 GWD/MTU and LEU+, including:
  - Application of Framatome's Increased Burnup Topical Report (IBTR, ANP-10358P-A)
  - Approval of Spent Fuel Pool Heatload analysis method
  - Assessment of decay heat model
  - Associated TS changes
- Additional elements to the Fuels LAR may be required to support 24M fuel cycles at MPS3 to address core design or operational margin constraints associated with 24M fuel cycles.

# Fuels LAR – Increased Burnup Topical

- License Framatome's IBTR (ANP-10358P-A):
  - Add to the TS 6.9.1.6.b list of NRC-approved COLR methodologies.
  - Confirm limitations and conditions of Framatome's IBTR are met.
  - Address LOCA Fuel dispersal for MPS3 consistent with the IBTR.
  - Address non-LOCA fuel dispersal for MPS3 by restricting cladding failure of rods with burnup greater than 62 GWD/MTU.

# Fuels LAR – SFP Heatload

- Update methodology for the SFP heatload licensing basis analysis:
  - Preserve FSAR-credited bulk pool temperature limits as analysis acceptance criterion.
  - Provide demonstration cases for 24-month cycle conditions with decay heat calculation using ORIGEN and evaluation of bulk pool temperature response using a 3-D GOTHIC thermal-hydraulic model.
  - Present an alternate method to calculate decay heat on a reload basis using SNF, which calculates isotopic concentrations and decay heat of spent fuel.

# Fuels LAR – Decay Heat Models

- Confirm continued applicability of Decay Heat Models for HBU/LEU+:
  - ANS 1979 Standard Decay Heat Model applied in:
    - Mass and Energy Release Analyses for Containment Response (WCAP-10325 and DOM-NAF-3-P-A)
    - Dominion Energy Methods for Non-LOCA Analyses (VEP-FRD-41-P-A)
  - Utilize ORIGEN to determine impact of higher enrichment and burnup
    - ORNL/TM-2020-1833, Volume 1 provides an industry benchmark for decay heat investigations using ORIGEN on extended enrichment and higher burnup LWR fuel
    - Goal to confirm a MPS3-specific decay heat application remains bounded by identified standards
  - IBTR addresses decay heat for Framatome methods

# Fuels LAR – Initial List of TS Changes

- TS 2.1.1.2, Reactor Core Safety Limits:
  - Peak fuel centerline temperature limit change, as necessary, to support application of the NRC-approved GALILEO Fuel Rod Performance Methodology.
- TS 5.3.1, Reactor Core Design Features:
  - Adopt NUREG-1431 (Westinghouse Standard TS) language related to the U-235 enrichment limit.
- TS 6.9.1.6.b, Core Operating Limits Report:
  - Add Framatome High Burnup Topical Report (ANP-10358P-A) as an NRC-approved COLR methodology.

# Fuels LAR
# TS Changes (Anticipated)

DESIGN FEATURES

5.3 REACTOR CORE

FUEL ASSEMBLIES

or slightly enriched uranium dioxide ($UO_2$)

5.3.1 The core shall contain 193 fuel assemblies. Each fuel assembly shall consist of 264 fuel rods (with zircaloy-4, ZIRLO®, Optimized ZIRLO™, or M5™ cladding) with an initial composition of natural ~~uranium dioxide or a maximum nominal enrichment of 5.0 weight percent U-235~~ as fuel material. Limited substitutions of zircaloy-4, ZIRLO®, M5™, or stainless steel filler rods for fuel rods, in accordance with NRC-approved applications of fuel rod configurations, may be used. Fuel assembly configurations shall be limited to those fuel designs that have been analyzed with applicable NRC staff-approved codes and methods, and shown by test or cycle-specific reload analyses to comply with all fuel safety design bases. Each fuel rod shall have a nominal active fuel length of 144 inches. A limited number of lead test assemblies that have not completed representative testing may be placed in nonlimiting core regions.

# Additional Topical Reports to be Included

- The following topical reports are needed to support 24M cycles at MPS3:
  - ANP-10358P-A, Revision 0, “Increased Burnup for PWRs,” March 2026.
  - ANP-10323, Revision 1, Supplement 1P, “One GALILEO Fuel Rod Thermal-Mechanical Methodology for Pressurized Water Reactors,” April 2025. (Under NRC review.)
  - SSP-14-P01/028-TR-P-A, Revision 0, Supplement 1, Revision 0, “Generic Application of the Studsvik Scandpower Core Management System to Pressurized Water Reactors: Supplement for Extended Enrichment, Burnup, and SMRs,” January 2025. (Draft SE issued March 2026.)
- These topical reports will be either submitted to the NRC for inclusion into the COLR or adopted into the MPS3 licensing basis under the provisions of 10 CFR 50.59.

# TS Surveillance Requirements LAR

- Update SR frequencies using GL 91-04 guidance for non-calibration SRs and calibration SRs (including instrument drift) as the main LAR enclosure.
- Approach:
  - GL 91-04 guidance used for calibration and non-calibration frequencies.
  - Drift analysis will be performed where required.
  - Setpoints are being verified, methodology based on RG 1.105, Rev 3.
  - Evaluations consider TS Section 6.8.4 Programs as needed.
- Work in progress, no additional items requiring NRC approval have been identified to date.
- Remaining work expected to be completed and available for audit by the time of submittal.

# TS Surveillance Requirements LAR

- Identified recent precedents:
  - St. Lucie Plant Unit 2, Issuance of amendment to increase certain SR frequencies to support 24M cycles in the SFCP (ADAMS Accession No. ML26023A004)
  - Prairie Island Nuclear Generating Station Units 1 and 2, Issuance of amendments to support 24-month operating cycle (ADAMS Accession No. ML22166A389)
  - Fermi 2, Issuance of amendment to TS to change certain surveillance intervals to accommodate a 24-month fuel cycle (ADAMS Accession No. ML20358A155)

# Fee Waiver

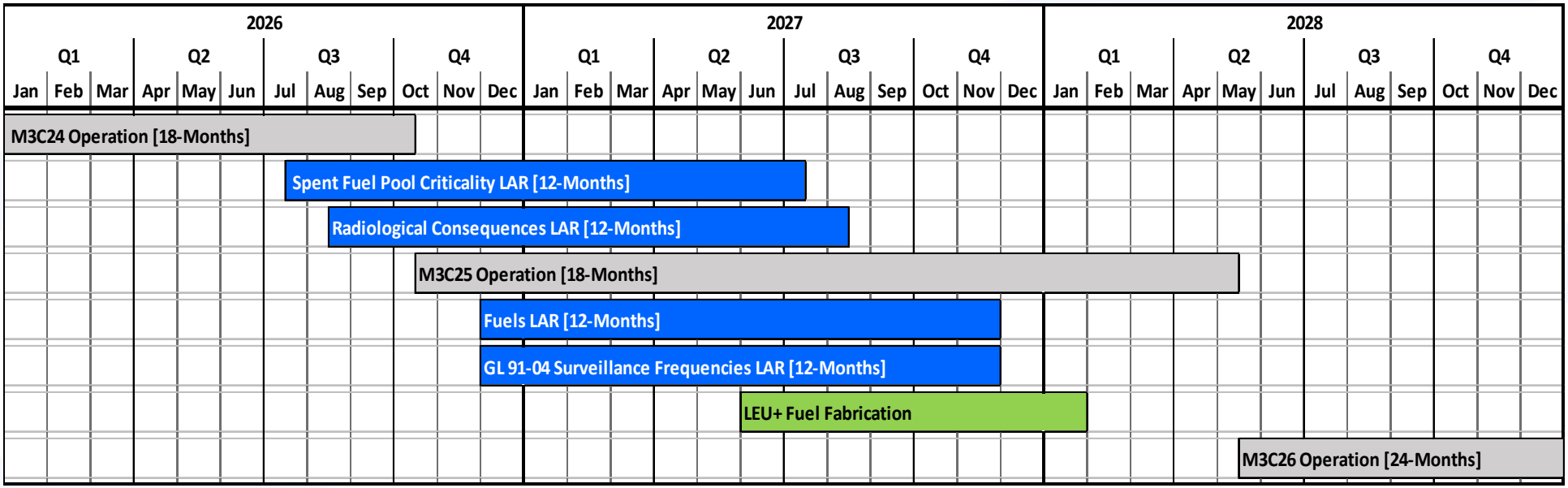
- DENC intends to request a fee waiver for the SFP Criticality and Radiological Consequence LARs pursuant to 10 CFR 170.11(b) on the basis that the fee waiver is authorized by law and is otherwise in the public interest.

- The review of the LARs will assist in implementing generic regulatory improvements, with MPS3 as the lead plant utilizing fuel assemblies greater than 5.0 wt% U-235 enrichment limit set in 10 CFR 50.68(b)(7) in a full batch, with DENC requesting an exemption to 10 CFR 50.68(b)(7).

- The SFP criticality LAR will support the implementation of LEU+ fuel at MPS3 up to 6.5 wt%, with MPS3 being the lead plant to implement this change.

- Revised radiological consequence analyses for LEU+ and high burnup fuel apply DG-1425 in a first use LAR case, with DENC requesting an exemption to 10 CFR 50.67(b)(2)(iii) and GDC 19.

# Summary




- The first 24-month fuel cycle for MPS3 is set to begin in the Spring of 2028.
- Higher enrichment (LEU+) and higher burnup limits are required to meet 24-month cycle energy requirements.
- Four unique LARs are anticipated.



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# Acronyms

| | |
|---|---|
| 24M | 24-month fuel cycles |
| BP | Burnable Poison |
| BWR | Boiling Water Reactor |
| CLB | Current Licensing Basis |
| CFR | Code of Federal Regulations |
| COLR | Core Operating Limits Report |
| CR | Control Room |
| DENC | Dominion Energy Nuclear Connecticut |
| DG | Draft Guide |
| ECCS | Emergency Core Cooling System |
| FFRD | Fuel Fragmentation, Relocation, and Dispersal |
| FSAR | Final Safety Analysis Report |
| Gad | Gadolinia |
| GL | Generic Letter |
| GDC | General Design Criteria |
| GSI | Generic Safety Issue |
| GWD/MTU | Gigawatt-Day per Metric Ton of Uranium |
| HBU | High Burnup |
| IBTR | Increased Burnup Topical Report |
| IFBA | Integrated Fuel Burnable Absorber |
| LAR | License Amendment Request |

# Acronyms

| | |
|---|---|
| LCO | Limiting Condition for Operation |
| LEU+ | Fuel Enriched between 5-10 wt% |
| LOCA | Loss of Coolant Accident |
| LTA | Lead Test Assembly |
| MPS3 | Millstone Power Station Unit 3 |
| MTU | Metric Ton of Uranium |
| NFSR | Nuclear Facility Surveillance Requirements |
| NRC | Nuclear Regulatory Commission |
| ORNL | Oak Ridge National Laboratory |
| PWR | Pressurized Water Reactor |
| RG | Regulatory Guide |
| SFP | Spent Fuel Pool |
| SLCRS | Supplementary Leak Collection and Release System |
| SNF | Software that calculates decay heat using Casmo/Simulate models |
| SPU | Stretch Power Uprate |
| SR | Surveillance Requirement |
| TEDE | Total Effective Dose Equivalent |
| TS | Technical Specifications |
| VEGP | Vogtle Electric Generating Plant |
| wt% | U-235 weight percent enrichment |