UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 19, 2026

Mr. Christopher H. Mudrick. Sr.
Senior Vice President
Constellation Energy Generation, LLC
President and Chief Nuclear Officer
Constellation Nuclear
200 Exelon Way
Kennett Square, PA 19348

SUBJECT: BRAIDWOOD STATION, UNITS 1 AND 2; AND BYRON STATION, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENT NOS. 248, 247, 245 AND 245 TO ADOPT CHANGES TO THE QUADRANT POWER TILT RATIO TECHNICAL SPECIFICATION (EPID L-2025-LLA-0194)

Dear Mr. Mudrick:

The U.S. Nuclear Regulatory Commission (NRC or Commission) has issued the enclosed amendments (listed below) in response to the Constellation Energy Generation, LLC (CEG) application dated December 12, 2025 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML25346A095):

1. Amendment No. 248 to Renewed Facility Operating License No. NPF-72 and Amendment No. 247 to Renewed Facility Operating License No. NPF-77 for Braidwood Station, Units 1 and 2, respectively;

2. Amendment No. 245 to Renewed Facility Operating License No. NPF-37 and Amendment No. 245 to Renewed Facility Operating License No. NPF-66 for Byron Station, Unit Nos. 1 and 2, respectively;

The amendments revise the technical specifications (TSs) for each facility requirements associated with TS 3.2.4, “Quadrant Power Tilt Ratio (QPTR).”

A copy of the NRC staff's Safety Evaluation is also enclosed. The Notice of Issuance will be included in a future Commission *Federal Register* notice.

Sincerely,

/RA/

Scott P. Wall, Senior Project Manager
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos.: STN 50-456, STN 50-457,
STN 50-454, and STN 50-455

Enclosures:
1. Amendment No. 248 to NPF-72
2. Amendment No. 247 to NPF-77
3. Amendment No. 245 to NPF-37
4. Amendment No. 245 to NPF-66
5. Safety Evaluation

cc: Listserv

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CONSTELLATION ENERGY GENERATION, LLC

DOCKET NO. STN 50-456

BRAIDWOOD STATION, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 248
Renewed License No. NPF-72

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Constellation Energy Generation, LLC dated December 12, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the renewed license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-72 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 248 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

ILKA BERRIOS

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Date: 2026.05.19 14:58:10 -04'00'

Ilka Berrios, Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility Operating License and Technical Specifications

Date of Issuance: May 19, 2026

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CONSTELLATION ENERGY GENERATION, LLC

DOCKET NO. STN 50-457

BRAIDWOOD STATION, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 247
Renewed License No. NPF-77

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Constellation Energy Generation, LLC dated December 12, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the renewed license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-77 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 247 and the Environmental Protection Plan contained in Appendix B, both of which are attached to Renewed License No. NPF-72, dated January 27, 2016, are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

ILKA BERRIOS

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Date: 2026.05.19 14:58:56 -04'00'

Ilka Berrios, Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility
Operating License and
Technical Specifications

Date of Issuance: May 19, 2026

ATTACHMENT TO LICENSE AMENDMENT NOS. 248 AND 247

RENEWED FACILITY OPERATING LICENSE NOS. NPF-72 AND NPF-77

BRAIDWOOD STATION, UNITS 1 AND 2

DOCKET NOS. STN 50-456 AND STN 50-457

Replace the following pages of the Renewed Facility Operating Licenses and Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Renewed Facility Operating Licenses

REMOVE	INSERT
License NPF-72 Page 3	License NPF-72 Page 3
License NPF-77 Page 3	License NPF-77 Page 3

Technical Specifications

REMOVE	INSERT
3.2.4 – 1	3.2.4 – 1
3.2.4 – 2	3.2.4 – 2
3.2.4 – 4	3.2.4 – 4

(2) Constellation Energy Generation, LLC, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

(3) Constellation Energy Generation, LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(4) Constellation Energy Generation, LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(5) Constellation Energy Generation, LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of 3645 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 248 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(2) Constellation Energy Generation, LLC, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;

(3) Constellation Energy Generation, LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(4) Constellation Energy Generation, LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(5) Constellation Energy Generation, LLC, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. The renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of 3645 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 247 and the Environmental Protection Plan contained in Appendix B, both of which are attached to Renewed License No. NPF-72, dated January 27, 2016, are hereby incorporated into the renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3.2 POWER DISTRIBUTION LIMITS

3.2.4 QUADRANT POWER TILT RATIO (QPTR)

LCO 3.2.4 The QPTR shall be ≤ 1.02.

---NOTE---
The QPTR limit is not required to be met following refueling until after initial performance of SR 3.3.1.6.

APPLICABILITY: MODE 1 with THERMAL POWER > 50% RTP.

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. QPTR not within limit.	A.1	Reduce THERMAL POWER ≥ 3% from RTP for each 1% of QPTR > 1.02.	2 hours after each QPTR determination
	AND		
	A.2	Determine QPTR.	Once per 12 hours
	AND		
	A.3	Perform SR 3.2.1.1, SR 3.2.1.2, and SR 3.2.2.1.	24 hours after achieving equilibrium conditions from a THERMAL POWER reduction per Required Action A.1 AND Once per 7 days thereafter
	AND		

(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. (continued)	A.4 Re-evaluate safety analyses and confirm results remain valid for duration of operation under this condition.	Prior to increasing THERMAL POWER above the limit of Required Action A.1
	AND	
	A.5 --------NOTES-------- 1. Perform Required Action A.5 only after Required Action A.4 is completed. 2. Required Action A.6 shall be completed whenever Required Action A.5 is performed. ----------	
	Normalize excore detectors to restore QPTR to within limits.	Prior to increasing THERMAL POWER above the limit of Required Action A.1
	AND	

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.2.4.1	----------NOTES---------- 1. With the calibrated output of one Power Range Neutron Flux channel unavailable and THERMAL POWER ≤ 75% RTP, the remaining three Power Range Neutron Flux channels can be used to determine QPTR. 2. The movable incore detector system can be used to determine QPTR. ----------	
	Verify QPTR is within limit.	In accordance with the Surveillance Frequency Control Program AND ------NOTE------ Only required to be performed if calibrated output of one Power Range Neutron Flux channel is unavailable, and THERMAL POWER > 75% RTP. ---------- Once within 12 hours following any rod motion ≥ 12 steps if an associated digital rod position indicator is inoperable using the movable incore detector system.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CONSTELLATION ENERGY GENERATION, LLC

DOCKET NO. STN 50-454

BYRON STATION, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 245
Renewed License No. NPF-37

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Constellation Energy Generation, LLC dated December 12, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the renewed license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-37 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 245 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 90 days of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

ILKA BERRIOS

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Date: 2026.05.19 15:00:20 -04'00'

Ilka Berrios, Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility Operating License and Technical Specifications

Date of Issuance: May 19, 2026

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

CONSTELLATION ENERGY GENERATION, LLC

DOCKET NO. STN 50-455

BYRON STATION, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 245
Renewed License No. NPF-66

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Constellation Energy Generation, LLC dated December 12, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 245, and the Environmental Protection Plan contained in Appendix B, both of which were attached to Renewed License No. NPF-37, dated November 19, 2015, are hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of the date of its issuance and shall be implemented no later than 30 days after the implementation of Byron Amendment 238, which removes the use of the Power Distribution Monitoring System from the Byron Technical Specifications.

FOR THE NUCLEAR REGULATORY COMMISSION

ILKA BERRIOS

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Ilka Berrios, Chief
Plant Licensing Branch III
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Attachment:
Changes to the Renewed Facility Operating License and Technical Specifications

Date of Issuance: May 19, 2026

ATTACHMENT TO LICENSE AMENDMENT NOS. 245 AND 245

RENEWED FACILITY OPERATING LICENSE NOS. NPF-37 AND NPF-66

BYRON STATION, UNIT NOS. 1 AND 2

DOCKET NOS. STN 50-454 AND STN 50-455

Replace the following pages of the Renewed Facility Operating Licenses and Appendix A, Technical Specifications, with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Renewed Facility Operating Licenses

REMOVE	INSERT
License NPF-37 Page 3	License NPF-37 Page 3
License NPF-66 Page 3	License NPF-66 Page 3

Technical Specifications

REMOVE	INSERT
3.2.4 – 1	3.2.4 – 1
3.2.4 – 2	3.2.4 – 2
3.2.4 – 4	3.2.4 – 4

(2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report, as supplemented and amended;

(3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. The renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of 3645 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A as revised through Amendment No. 245 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Deleted.

(4) Deleted.

(2) Pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Updated Final Safety Analysis Report, as supplemented and amended;

(3) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;

(4) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components; and

(5) Pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.

C. The renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporated below:

(1) Maximum Power Level

The licensee is authorized to operate the facility at reactor core power levels not in excess of 3645 megawatts thermal (100 percent rated power) in accordance with the conditions specified herein.

(2) Technical Specifications

The Technical Specifications contained in Appendix A (NUREG-1113), as revised through Amendment No. 245, and the Environmental Protection Plan contained in Appendix B, both of which were attached to Renewed License No. NPF-37, dated November 19, 2015, are hereby incorporated into this renewed license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Deleted.

3.2 POWER DISTRIBUTION LIMITS

3.2.4 QUADRANT POWER TILT RATIO (QPTR)

LCO 3.2.4 The QPTR shall be ≤ 1.02.

---NOTE---
The QPTR limit is not required to be met following refueling until after initial performance of SR 3.3.1.6.

APPLICABILITY: MODE 1 with THERMAL POWER > 50% RTP

ACTIONS

CONDITION	REQUIRED ACTION		COMPLETION TIME
A. QPTR not within limit.	A.1	Reduce THERMAL POWER ≥ 3% from RTP for each 1% of QPTR > 1.02.	2 hours after each QPTR determination
	AND		
	A.2	Determine QPTR	Once per 12 hours
	AND		
	A.3	Perform SR 3.2.1.1, SR 3.2.1.2, and SR 3.2.2.1.	24 hours after achieving equilibrium conditions from a THERMAL POWER reduction per Required Action A.1 AND Once per 7 days thereafter
	AND		

(continued)

ACTIONS

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. (continued)	A.4 Re-evaluate safety analyses and confirm results remain valid for duration of operation under this condition.	Prior to increasing THERMAL POWER above the limit of Required Action A.1
	AND	
	A.5 --------NOTES-------- 1. Perform Required Action A.5 only after Required Action A.4 is completed. 2. Required Action A.6 shall be completed whenever Required Action A.5 is performed. ----------	
	Normalize excore detectors to restore QPTR to within limits.	Prior to increasing THERMAL POWER above the limit of Required Action A.1
	AND	

(continued)

SURVEILLANCE REQUIREMENTS

SURVEILLANCE		FREQUENCY
SR 3.2.4.1	----------NOTES---------- 1. With the calibrated output of one Power Range Neutron Flux channel unavailable and THERMAL POWER ≤ 75% RTP, the remaining three Power Range Neutron Flux channels can be used to determine QPTR. 2. The movable incore detector system can be used to determine QPTR. ----------	
	Verify QPTR is within limit.	In accordance with the Surveillance Frequency Control Program AND ------NOTE----- Only required to be performed if calibrated output of one Power Range Neutron Flux channel is unavailable, and THERMAL POWER > 75% RTP. ---------- Once within 12 hours following any rod motion ≥ 12 steps if an associated digital rod position indicator is inoperable using the movable incore detector system.

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION

RELATED TO

AMENDMENT NO. 248 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-72

AMENDMENT NO. 247 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-77

AMENDMENT NO. 245 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-37

AMENDMENT NO. 245 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-66

CONSTELLATION ENERGY GENERATION, LLC

BRAIDWOOD STATION, UNITS 1 AND 2

BYRON STATION, UNIT NOS. 1 AND 2

DOCKET NOS. STN 50-456, STN 50-457, STN 50-454, AND STN 50-455

1.0 PROPOSED CHANGES

By letter dated December 12, 2025 (Agencywide Documents Access and Management System Accession No. ML25346A095), Constellation Energy Generation, LLC (CEG, the licensee) submitted a license amendment request (LAR) to the U.S. Nuclear Regulatory Commission (NRC, the Commission) for Braidwood Station, Units 1 and 2 (Braidwood) and Byron Station, Unit Nos. 1 and 2 (Byron). The proposed changes would revise the Technical Specifications (TS) based on the Pressurized Water Reactor Owners Group (PWROG) Topical Report (TR) PWROG-22021-P-A/NP-A, Revision 0, “Justifications for the Proposed Changes to the Quadrant Power Tilt Ratio [QPTR] Technical Specification” (PWROG-22021) (ML23338A207), and the associated NRC staff safety evaluation (SE) of TR PWROG-22021-P-A/NP-A (ML25148A288).

A notice of consideration of proposed issuance of amendments with proposed no significant hazards consideration determination was published in the *Federal Register* on February 24, 2026 (91 FR 8916), and there has been no public comment on such finding.

1.1 System Description

The QPTR is one of the TS-required power distribution limits designed to ensure that the reactor core remains within the bounds of the approved safety analyses and that operation does not violate fuel design criteria. The QPTR limit ensures that gross radial power distribution remains consistent with the design values used in the safety analyses by detecting changes in the radial power distribution between calibrated states. The QPTR is determined periodically during the

fuel cycle in between periodic surveillances of core peaking factors, the Heat Flux Hot Channel Factor ($F_Q(Z)$) and the Nuclear Enthalpy Rise Hot Channel Factor ($F_{\Delta H}$), to ensure that the previously performed measurement of these factors remain valid. The Nuclear Instrumentation System (NIS) Power Range (PR) Excore Detectors (channel designations N41, N42, N43, and N44) are used to measure the QPTR. Each NIS PR detector channel has an upper and lower chamber. The calibrated output is the ratio of the measured PR detector current to the full power, zero axial offset PR detector currents. The full power, zero axial offset PR detector currents are calculated via a flux map for the incore-excore calibration that is performed in accordance with Surveillance Requirement (SR) 3.3.1.6. The purpose of this incore-excore calibration is to make the PR excore detector indication of the axial flux distribution equal to the axial flux distribution that is measured by the incore detectors and performed in accordance with SR 3.3.1.6. The QPTR is defined in NUREG-1431, Revision 5, "Standard Technical Specifications, Westinghouse Plants" (ML21259A155), as follows:

> QPTR shall be the ratio of the maximum upper excore detector calibrated output to the average of the upper excore detector calibrated outputs, or the ratio of the maximum lower excore detector calibrated output to the average of the lower excore detector calibrated outputs, whichever is greater.

1.2 PWROG-22021 and the Licensee's Proposed TS Changes

PWROG-22021 provided technical justifications for changes to the Limiting Conditions for Operation (LCO), Required Actions, Completion Times and SRs of TS 3.2.4, "Quadrant Power Tilt Ratio (QPTR)" and changes to the Required Actions of TS 3.3.1, "Reactor Trip System (RTS) Instrumentation" in NUREG-1431, Revision 5, "Standard Technical Specifications Westinghouse Plants" to allow licensees to reference the TR for regulatory actions.

PWROG-22021-related changes to NUREG-1431 included:

- Addition of a NOTE to LCO 3.2.4 which states that the QPTR limit is not required to be met following refueling until after the initial performance of SR 3.3.1.6.
- Required Action A.1 of TS 3.2.4 was revised to state that for each 1% that QPTR exceeds 1.02, the required reduction in thermal power is 3%.
- A logical "OR" separating Required Action A.1 and new Required Action, A.1.2 was added. New Required Action, A.1.2 requires reducing the limits on $F_Q(Z)$ and $F^N_{\Delta H}$ by at least 3% for each 1% that QPTR exceeds the limit of 1.02.
- The elements of SR 3.2.4.1 and SR 3.4.2.2 were consolidated into one SR and SR 3.4.2.2 was deleted.
- TS 3.3.1 Required Actions D.1.2, D.2.1 and D.2.2 were deleted.
- Editorial changes, relettering and renumbering to accommodate the additions and deletions described above were also made.

The NRC staff reviewed PWROG-22021 and provided an SE for the changes. The NRC's SE evaluated the justifications for the changes and provided a set of limitations and conditions for use of the TR to justify plant specific adoptions. The NRC staff approved PWROG-22021. The approved version of PWROG-22021, which includes the NRC's SE, is available at (ML25231A213).

The licensee's proposed changes to Braidwood and Byron TSs, LCOs, and SRs consistent with PWROG-22021 are described below where additions are in **bold text** while deletions are in ~~strikethrough text~~. Specifically, the licensee's proposed changes to TS 3.2.4, "QPTR":

1.2.1 Add the NOTE given below to Limiting Condition for Operation (LCO) 3.2.4

LCO 3.2.4 The QPTR shall be $\leq$ 1.02.

----------NOTE----------
The QPTR limit is not required to be met following refueling until after initial performance of SR 3.3.1.6

1.2.2 Revise, in part, LCO 3.2.4 REQUIRED ACTION A.1 as follows:

CONDITION	REQUIRED ACTION	COMPLETION TIME
A. QPTR not within limit.	A.1 Reduce THERMAL POWER $\geq$ 3% from RTP for each 1% of QPTR > 1.0**2**~~0~~	2 hours after each QPTR determination

1.2.3 Deleting SR 3.2.4.2 and revising SR 3.2.4.1 as follows

SURVEILLANCE		FREQUENCY
SR 3.2.4.1	----------NOTES---------- 1. With **the calibrated output of** ~~input from~~ one Power Range Neutron Flux channel **unavailable** ~~inoperable~~ and THERMAL POWER ≤ 75% RTP, the remaining ~~p~~**P**ower ~~r~~**R**ange Neutron Flux channels can be used **to determine** ~~for calculating~~ QPTR. 2. **The moveable incore detector system can be used to determine QPTR.** ~~SR 3.2.4..2 may be performed in lieu of this Surveilance.~~ ----------	
	Verify QPTR is within limit. ~~by calculation.~~	In accordance with the Surveillance Frequency Control Program **AND** **----------NOTE----------** **Only required to be performed if calibrated output of one Power Range Neutron Flux channel is unavailable, and THERMAL POWER > 75% RTP.** **----------** **Once within 12 hours following any rod motion ≥ 12 steps if an associated digital rod position indicator is inoperable using the movable incore detector system.**

1.3 Variations

In addition to the changes proposed consistent with PWROG-22021 discussed in section 1.2, the licensee proposed the following variations.

1.3.1 Editorial Variations

1. PWROG-22021 includes a proposed change to remove Required Actions D.1.2, D.2.1, and D.2.2 from LCO 3.3.1, "Reactor Trip System (RTS) Instrumentation." These changes to LCO 3.3.1 are not applicable because the suggested changes already exist in Braidwood and Byron TSs.

2 PWROG-22021 consolidates SR 3.2.4.1 and 3.2.4.2. The Frequency of the updated SR 3.2.4.1 is modified as a result. CEG is adding a statement to the SR 3.2.4.1 Frequency to clarify that the incore movable detector system should be used if an associated digital rod position indicator is inoperable. While this statement is implied and added as SR 3.2.4.1 Note 2 in PWROG-22021, CEG is choosing to explicitly state this in the Surveillance Requirement Frequency.

3 To be consistent with the wording in NUREG-1431, as illustrated in Attachment 4 of the LAR, CEG is removing the word "the" from the Completion Times of LCO 3.2.4 Required Actions A.4 and A.5.

1.3.2 Content Variations

- PWROG-22021 includes a proposed change that to relabels LCO 3.2.4 Required Action A.1 to A.1.1 and add an "OR" between Required Action A.1.1 and A.1.2 with Required Action A.1.2 stating the following:

  "Reduce the limits for $F_Q(Z)$ and $F_{\Delta H}$ $\geq$ 3% from RTP for each 1% or QPTR > 1.02"

  CEG is choosing not to adopt the addition of alternative Required Action A.1.2.

2.0 REGULATORY EVALUATION

Per Title 10 of the *Code of Federal Regulations* (10 CFR) Part 50.90, whenever a holder of a license wishes to amend the license, including TSs in the license, an application for amendment must be filed, fully describing the changes desired. Under 10 CFR Part 50.92(a), determinations on whether to grant an applied-for license amendment are to be guided by the considerations that govern the issuance of initial licenses or construction permits to the extent applicable and appropriate. Both the common standards for licenses and construction permits in 10 CFR Part 50.40(a), and those specifically for issuance of operating licenses in 10 CFR Part 50.57(a)(3), provide that there must be 'reasonable assurance' that the activities at issue will not endanger the health and safety of the public.

The regulations in 10 CFR 50.36(c)(2), "Limiting conditions for operation," Limiting conditions for operation are the lowest functional capability or performance levels of equipment required for safe operation of the facility. When a limiting condition for operation of a nuclear reactor is not met, the licensee shall shut down the reactor or follow any remedial action permitted by the technical specifications until the condition can be met.

The regulations in 10 CFR 50.36(c)(3), "Surveillance Requirements," SRs are requirements relating to test, calibration, or inspection to assure that the necessary quality of systems and components is maintained, that facility operation will be within safety limits, and that the limiting conditions for operation will be met.

The following 10 CFR Part 50, Appendix A, General Design Criteria (GDC) are applicable:

- GDC 10, "Reactor design." The reactor core and associated coolant, control, and protection systems shall be designed with appropriate margin to assure that specified acceptable fuel design limits are not exceeded during any condition of normal operation, including the effects of anticipated operational occurrences.

- GDC 11, "Reactor inherent protection." The reactor core and associated coolant systems shall be designed so that in the power operating range the net effect of the prompt inherent nuclear feedback characteristics tends to compensate for a rapid increase in reactivity.

- GDC 12, "Suppression of reactor power oscillations." The reactor core and associated coolant, control, and protection systems shall be designed to assure that power oscillations which can result in conditions exceeding specified acceptable fuel design limits are not possible or can be reliably and readily detected and suppressed.

The NRC staff's guidance for the review of TSs is in Chapter 16.0, "Technical Specifications," of NUREG-0800, Revision 3, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR [Light-Water Reactor] Edition" (SRP), March 2010 (ML100351425). As described therein, the NRC staff has prepared standard TSs (STSs) for each of the LWR nuclear designs. Accordingly, the NRC staff's review includes consideration of whether the proposed changes are consistent with NUREG-1431, "Standard Technical Specifications, Westinghouse Plants," Volume 1, "Specifications," and Volume 2, "Bases," Revision 5, September 2021 (ML21259A155 and ML21259A159, respectively), as modified by NRC-approved travelers.

3.0 TECHNICAL EVALUATION

3.1 Proposed TS Changes Similar to PWROG-22021

The current version of TS 3.2.4 at Braidwood and Byron contains six Required Actions, A.1 through A.6, the licensee must take when the QPTR is not within limit. These are the remedial measures permitted by the TS that allow the licensee to continue plant operation as an alternative to reducing THERMAL POWER below 50 percent Rated Thermal Power (RTP) when the LCO is not met. The current TS 3.2.4 also contains two SRs that must be met while the plant is in MODE 1 with THERMAL POWER greater that 50 percent RTP. The licensee proposed changes to the current TS 3.2.4 based on PWROG-22021. The NRC staff compared the proposed TS changes described in section 1.2 of this SE against the changes approved in PWROG-22021.

3.1.1 Add a NOTE to LCO 3.2.4

The licensee proposed to add a NOTE change to LCO 3.2.4 stating that the QPTR limit is not required to be met following refueling until after the initial performance of SR 3.3.1.6. The staff reviewed the proposed change as described in section 3.1 of the LAR and illustrated in Attachments 4 and 5 of the LAR. The staff compared the proposed Byron and Braidwood TS changes to the PWROG-22021 changes.

In Section 3.1 of the NRC SE for PWROG-22021, the NRC staff found that the addition of the proposed NOTE in LCO 3.2.4 is justified because the full power currents for the new cycle are not determined until the first incore-excore calibration is performed per SR 3.3.1.6. Therefore, this LCO would not apply for a new fuel cycle following a refueling outage (RFO) until SR 3.3.1.6 is performed. Performance of this SR ensures that the full power currents used by the PR NIS to calculate the QPTR are representative of the new core.

The staff reviewed the LAR and determined that the justification above remains valid for Byron and Braidwood.

The NRC staff finds that the requirements of 10 CFR 50.36(c)(2) will continue to be met at Braidwood and Byron because the QPTR LCO will continue to be the lowest functional capability or performance levels of equipment required for safe operation of the facility. The change to this LCO neither affects the plant reactor protection system (RPS) trip setpoints nor the excore signal inputs to the RPS. Therefore, the NRC staff concludes that adding a NOTE to LCO 3.2.4 is acceptable.

3.1.2 Revise LCO 3.2.4 REQUIRED ACTION A.1

The licensee proposed to change LCO 3.2.4, Required Action A.1 to reduce THERMAL POWER ≥ 3 percent from RTP for each 1 percent of QPTR > 1.02, instead of for each 1 percent QPTR > 1.00.

The staff reviewed the proposed change as described in Sections 3.2 of the LAR and illustrated in Attachments 4 and 5 of the LAR. The staff compared the proposed Byron and Braidwood TS changes to the PWROG-22021 changes.

In Section 3.2 of the SE for PWROG-22021, the NRC staff found that the change is acceptable for licensees that use the reload methodology described in TR WCAP-9272-P-A, “Westinghouse Reload Safety Evaluations Methodology,” July 1985 (Westinghouse Electric Company (Westinghouse) Non- Public - Proprietary). Specifically, the NRC staff found it acceptable that the Westinghouse reload safety analysis methodologies conservatively bound a QPTR value up to 1.02 because the QPTR limit of 1.02, at which corrective action is required, provides a margin of protection for both the departure from nucleate boiling (DNB) ratio and the linear heat generation rate. The NRC staff also found it acceptable that there is no need to reduce power for a QPTR value between 1.00 and 1.02 because its 1.02 value does not result in challenging the core peaking factor uncertainties.

For licensees that use a different reload methodology, PWROG-22021 has limitations and conditions (L&Cs) that require discussion of how the allowances and uncertainties support a safety analysis with QPTR up to 1.02 is necessary (L&C 1). Additionally, the L&Cs require a discussion and confirmation of the plant’s RPS and nuclear instrumentation (NI) circuitry stating that they are consistent with information provided in PWROG-22021 (L&C 2).

Reload Methodology Different from WCAP-9272

In the LAR, the licensee indicated that the current reload methodology for Braidwood and Byron Vantage+ fuel is based on WCAP-9272. Beginning with Braidwood, Unit 1 (fall 2025 RFO), the licensee will introduce Framatome GAIA fuel to Braidwood, Unit 2 (spring 2026 RFO), Byron. Unit No. 1 (spring 2026 RFO), and Byron, Unit No. 2 (fall 2026 RFO). TS 5.6.5.b, "Core Operating Limits Report," contains a list of analytical methods used in-core operating limits development which have been previously approved for use by NRC staff at Braidwood and Byron. By amendments dated August 21, 2025 (ML25153A008), the staff authorized the addition of 14 NRC-approved topical reports for developing core operating limits for Framatome GAIA fuel.

In Section 2.5 of the LAR, in accordance with Section 4.0 of the Staff's SE for PWROG 22021, the licensee provided a discussion of how the allowances and uncertainties that support a QPTR of up to 1.02 are addressed in the safety analysis. The staff reviewed the discussion and determined PWROG 22021 changes are justified at Byron and Braidwood.

RPS and NI Circuitry

In Section 2.5 of the LAR, the licensee states:

> CEG confirms that the Reactor Protection System (RPS) and Nuclear Instrumentation (NI) circuitry for Braidwood and Byron align with the information provided in PWROG-22021-P-A. The transition to GAIA fuel does not impact the RPS and NI circuitry. All GAIA transients have been performed to demonstrate compliance with the existing RPS setpoints.

The NRC staff determined that the licensee has adequately addressed the L&Cs 1 and 2 of PWROG-22021.

Based on the above, the NRC staff finds that the requirements of 10 CFR 50.36(c)(2) will continue to be met at Braidwood and Byron because the reload safety analysis methodologies authorized in Braidwood and Byron TS 5.6.5.b will conservatively bound a QPTR value up to 1.02. Therefore, the NRC staff concludes that the changes to LCO 3.2.4 are acceptable.

3.1.3 Consolidation of SR 3.2.4.1 and SR 3.2.4.2

The licensee proposed to consolidate SR 3.2.4.1 and 3.4.2.2.

The staff reviewed the proposed change, as described in Sections 3.3 of the LAR and illustrated in Attachments 4 and 5 of the LAR. The staff compared the proposed Byron and Braidwood TS changes to the PWROG-22021 changes.

Both SR 3.2.4.1 and SR 3.2.4.2 require verification that QPTR is within its limit. For the situation where THERMAL POWER is above 75 percent and the input from one or more Power Range Neutron Flux channels are inoperable, SR 3.2.4.2 requires a more frequent verification of QPTR. The proposed changes to SRs 3.2.4.1 and 3.2.4.2 would modify the NOTES, SR description, and SR frequency of 3.2.4.1 to account for the corresponding SR 3.2.4.2 elements so that all SR 3.2.4.2 requirements would be contained in the revised SR 3.4.2.1.

In the NRC SE for PWROG-22021, Section 3.5, the NRC staff found that the consolidated SR is acceptable because it would continue to provide assurance that the necessary quality of systems and components is maintained, such that facility operation will be continue to be within safety limits, and that the LCOs will be met.

The Frequency for the consolidated SR 3.2.4.1 in PWROG-22021 does not state which method should be used when THERMAL POWER is above 75 percent and the input from one or more Power Range Neutron Flux channels are unavailable. The licensee described a variation between the TS changes in the TR and those proposed for Byron and Braidwood in Section 2.6 of the LAR. The licensee stated:

> In PWROG-22021, SR 3.2.4.2 is removed and incorporated into SR 3.2.4.1. The Frequency of the updated SR 3.2.4.1 is modified as a result. CEG is adding a statement to the SR 3.2.4.1 Frequency to clarify that the incore movable detector system should be used if an associated digital rod position indicator is inoperable. While this statement is implied and added as SR 3.2.4.1 Note 2 in PWROG-22021, CEG is choosing to explicitly state this in the Surveillance Requirement Frequency.

The staff reviewed the addition of the phrase to the end of the frequency for the consolidated SR 3.2.4.1 at Byron and Braidwood. The staff determined that stipulating use of movable incore detector system is an acceptable variation. Therefore, the NRC staff finds that the requirements of 10 CFR 50.36(c)(3) will continue to be met and concludes that deleting SR 3.2.4.2 and revising SR 3.2.4.1 into one consolidated SR is acceptable.

3.1.4 Technical Conclusion

In the NRC SE of PWROG-22021, the NRC staff found that:

- Appropriate justification was provided for the addition of the proposed NOTE to LCO 3.2.4.

- A QPTR value up to 1.02 provides a margin of protection for both the DNB ratio and the linear heat generation rate.

- Appropriate justification was to the increase in the QPTR value from 1.00 to 1.02 before thermal power is reduced.

- The changes to TS 3.2.4, “QPTR,” do not increase the risk to the health and safety of the public.

- GDC 10 continues to be satisfied because there is appropriate margin to assure that Specified Acceptable Fuel Design Limits are not exceeded during any condition of steady state normal operation.

- GDC 11 continues to be satisfied because in the power operating range, the net effect of the prompt inherent nuclear feedback characteristics tends to compensate for a rapid increase in reactivity.

- GDC 12 continues to be satisfied because any power oscillations which can result in conditions exceeding specified acceptable fuel design limits are not possible or can be reliably and readily detected and suppressed

For these same reasons, the NRC staff concludes that the corresponding proposed changes to the Braidwood and Byron TSs in section 1.2 of this SE continue to meet the requirements of 10 CFR 50.36(c)(2) and 10 CFR 50.36(c)(3).

3.2 Additional Proposed TS Changes

In addition to the changes proposed consistent with PWROG-22021 discussed in section 1.2, the licensee proposed the variations discussed in section 1.3, which are discussed below.

3.2.1 Editorial Variations

As discussed in Section 1.3.1 of this SE, the NRC staff noted editorial variations from PWROG-22021. The NRC staff reviewed these variations and finds that they are acceptable because they are administrative in nature and do not affect the applicability of PWROG-22021 to the Braidwood and Byron TSs.

3.2.2 Content Variations

PWROG-22021 proposed to change TS 3.2.4 by adding a new Required Action A.1.2 that applies a penalty to the F_Q and $F_{\Delta H}$ limits as an alternative to the current Required Action A.1, with a QPTR value greater than 1.02. Rather than reducing power to restore the QPTR to within the LCO limit, the limits for the heat flux hot channel factor (F_Q) and nuclear enthalpy rise hot channel factor ($F_{\Delta H}$) are reduced and the peaking factor surveillances are performed to determine if the peaking factors are within the reduced limits without reducing reactor power.

The licensee is choosing not to adopt the addition of alternative Required Action A.1.2. The NRC staff reviewed this variation and finds that it is acceptable because the existing Required Actions, as modified by the changes discussed in Sections 3.1.1 and 3.1.2, are acceptable remedial measures the licensee can take when the LCO is not met. PWROG-22021 provides the options to either reduce power or apply penalties, therefore not adopting the addition of the alternative of applying penalties does not affect the applicability of PWROG-22021 to the Braidwood and Byron TSs.

3.3 TS Change Consistency

The NRC staff reviewed the proposed TS changes for technical clarity and consistency with the existing requirements for customary terminology and formatting. The NRC staff finds that the proposed changes are consistent with chapter 16.0 of the SRP and are therefore acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission’s regulations, the Illinois State officials were notified of the proposed issuance of the amendments on April 21, 2026. The State officials had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes to inspection or surveillance requirements and authorizations under, or changes to requirements in 10 CFR Part 50 or 52 with respect to installation or use of a facility component. The NRC staff has determined that any ground disturbance is limited to previously disturbed areas. Additionally, the NRC staff has determined that the action involves no significant change in the types or significant increase in the amounts of any effluents that may be released offsite, no significant increase in individual or cumulative public or occupational radiation exposure, and no significant increase in the potential for or consequences from radiological accidents. Finally, the NRC staff has determined that a categorical exclusion applies and that special circumstances are not present that would preclude reliance on the categorical exclusion. Accordingly, this action meets the eligibility criteria for categorical exclusion set forth in paragraphs (d)(1) and (d)(8) of 10 CFR 51.22, “Categorical exclusions.” Pursuant to 10 CFR 51.22, no environmental impact statement or environmental assessment need be prepared in connection with the action.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission’s regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or to the health and safety of the public.

Date: May 19, 2026

SUBJECT: BRAIDWOOD STATION, UNITS 1 AND 2; AND BYRON STATION, UNIT NOS. 1 AND 2 - ISSUANCE OF AMENDMENT NOS. 248, 247, 245 AND 245 TO ADOPT CHANGES TO THE QUADRANT POWER TILT RATIO TECHNICAL SPECIFICATION (EPID L-2025-LLA-0194) DATED MAY 19, 2026

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