VIRGINIA ELECTRIC AND POWER COMPANY
RICHMOND, VIRGINIA 23261

April 30, 2026

United States Nuclear Regulatory Commission
Attention: Document Control Desk
Washington, DC 20555-0001

Serial No. 26-119
SPS/MMT R0
Docket Nos. 50-280
50-281
License Nos. DPR-32
DPR-37

VIRGINIA ELECTRIC AND POWER COMPANY
SURRY POWER STATION UNITS 1 AND 2
2025 ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

Enclosed is the Surry Power Station Annual Radioactive Effluent Release Report for January 1, 2025, through December 31, 2025. The report submitted pursuant to Surry Power Station Technical Specification 6.6.B.3, includes a summary of the quantities of radioactive liquid and gaseous effluents and solid waste released during the 2025 calendar year, as outlined in Regulatory Guide 1.21, Revision 1, June 1974.

If you have any further questions, please contact William Terry at 757-365-2010.

Sincerely,

Geoffrey R. Hill
Director Nuclear Safety & Licensing
Surry Power Station

Attachment

Commitments made in this letter: None

cc: U. S. Nuclear Regulatory Commission
Region II
Marquis One Tower
ATTN: Division of Reactor Safety – Radiation Safety Branch
245 Peachtree Center Ave., NE Suite 1200
Atlanta, Georgia 30303-1257

NRC Senior Resident Inspector
Surry Power Station

ATTACHMENT

2025 Annual Radioactive Effluent Release Report

Surry Power Station

SURRY POWER STATION UNITS 1 AND 2
VIRGINIA ELECTRIC AND POWER COMPANY

2025 Annual Radioactive Effluent Release Report

Surry Power Station

Dominion Energy®

ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

SURRY POWER STATION

January 1, 2025 through December 31, 2025

Prepared By: _____

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ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

FOR

SURRY POWER STATION

January 1, 2025 through December 31, 2025

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FORWARD

This report is submitted as required by Appendix A to Operating License Nos. DPR-32 and DPR-37, Technical Specifications for Surry Power Station, Units 1 and 2, Virginia Electric and Power Company, Docket Nos. 50-280, 50-281, Section 6.6.B.3.

EXECUTIVE SUMMARY
ANNUAL RADIOACTIVE EFFLUENT RELEASE REPORT

The Annual Radioactive Effluent Release Report describes the radiological effluent control program conducted at Surry Power Station during the 2025 calendar year. This document summarizes the quantities of radioactive liquid and gaseous effluents and solid waste released from Surry Power Station in accordance with Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactivity in Solid Wastes and Releases of Radioactive Materials in Liquid and Gaseous Effluents from Light-Water-Cooled Nuclear Power Plants", Revision 1, June 1974. The report also includes an assessment of radiation doses to the maximum exposed member of the public due to the radioactive liquid and gaseous effluents.

During this reporting period, there were no unplanned liquid effluent releases and no unplanned gaseous effluent releases as classified according to the criteria in the Offsite Dose Calculation Manual.

Based on the 2025 effluent release data, 10CFR50 Appendix I dose calculations were performed in accordance with the Offsite Dose Calculation Manual. The dose calculations are as follows:

1. The total body dose due to liquid effluents is 3.70E-04 mrem, which is 0.006% of the 6 mrem dose limit. The critical organ dose due to liquid effluents is 4.15E-04 mrem to the GI-LLI, which is 0.002% of the 20 mrem dose limit.

2. The air dose due to noble gases in gaseous effluents is 2.52E-03 mrad gamma, which is 0.013% of the 20 mrad gamma dose limit, and 8.97E-04 mrad beta, which is 0.002% of the 40 mrad beta dose limit.

3. The critical organ dose from gaseous effluents due to I-131, I-133, H-3, and particulates with half-lives greater than 8 days is 2.45E-01 mrem to the bone, which is 0.817% of the 30 mrem dose limit.

There were no major changes to the radioactive liquid, gaseous or solid waste treatment systems during this reporting period.

No changes were made the Offsite Dose Calculation Manual (ODCM), VPAP-2103S, during this reporting period.

In accordance with the Nuclear Energy Institute (NEI) Industry Ground Water Protection Initiative, analysis results of ground water monitoring locations not included in the Radiological Environmental Monitoring Program (REMP), will be included in this report. Ground water monitoring well sample results are provided in Attachment 8.

The operation of Surry Power Station in 2025 resulted in a negligible radiation dose consequence to the maximum exposed member of the public in unrestricted areas. This is based on measured radioactivity and dose calculations performed.

Purpose and Scope

Attachment 1 includes a summary of the quantities of radioactive liquid and gaseous effluents and solid waste as outlined in Regulatory Guide 1.21, with data summarized on a quarterly or annual basis following the format of Tables 1, 2 and 3 of Appendix B, thereof.

Attachment 2 of this report includes an assessment of radiation doses to the maximum exposed member of the public due to radioactive liquid and gaseous effluents released from the site during 2025.

As required by Technical Specification 6.8.B, changes to the Offsite Dose Calculation Manual (ODCM) for the time period covered by this report are included in Attachment 3.

Major changes to the radioactive liquid, gaseous and solid waste treatment systems are reported in Attachment 4, as required by the ODCM, Section 6.7.2. If changes are made to these systems, the report shall include information to support the reason for the change and a summary of the 10CFR50.59 evaluation. In lieu of reporting major changes in this report, major changes to the radioactive waste treatment systems may be submitted as part of the annual FSAR update.

Attachment 5 identifies and explains why specific radioactive liquid and gaseous effluent monitoring instrumentation, as required by the ODCM in Attachments 1 and 5, was determined to be inoperable and not returned to operable status within 30 days.

Attachment 6 provides a list and description of unplanned releases from the site to unrestricted areas, during the reporting period, that meet the criteria listed in Step 6.7.3 of the ODCM.

Attachment 7 provides the typical lower limit of detection (LLD) capabilities of the radioactive effluent analysis instrumentation.

As required by the ODCM, Section 6.7.5, a summary is provided in Attachment 8 of on-site radioactive leaks or spills and ground water sample analyses that were communicated in accordance with the Industry Ground Water Protection Initiative reporting protocol. Sample analyses from ground water wells that are not part of the Radiological Environmental Monitoring Program are also provided in Attachment 8.

Discussion

The basis for the gaseous critical organ percent technical specification calculation in Table 1A on Attachment 1 is the ODCM. The requirements of Section 6.3.1 of the ODCM, are site boundary critical organ dose rate for iodine-131, iodine-133, for tritium, and for all radionuclides in particulate form with half-lives greater than 8 days shall be less than or equal to 1500 mrem/yr. The maximum critical receptor was the teen for the 1st, 2nd, and 3rd quarters and the child for the 4th quarter.

The basis for the calculation of the percent of technical specification for the total body and skin in Table 1A on Attachment 1 is the ODCM, Section 6.3.1, which requires that the dose rate for noble gases to areas at or beyond site boundary shall be less than or equal to 500 mrem/yr to the total body and less than or equal to 3000 mrem/yr to the skin.

In Regulatory Guide 1.21, "Measuring, Evaluating, and Reporting Radioactive Material in Liquid and Gaseous Effluents and in Solid Waste," the NRC has recommended that U.S. nuclear power plants evaluate whether C-14 is a "principal radionuclide," and if so, report the amount of C-14 released. At Surry Power Station (SPS), improvements in fuel performance and in effluent management practices over the years have resulted in a decrease in the amount and distribution of radionuclides released to the environment in gaseous effluents. As a result, C-14 has become a "principal radionuclide" for the gaseous effluent pathway at SPS, as defined in Regulatory Guide 1.21. However, because the dose contribution of C-14 from liquid radioactive waste is a small fraction of the dose compared to other radionuclides, evaluation of C-14 in liquid effluents is not required by Regulatory Guide 1.21.

The quantity of gaseous C-14 released to the environment can be estimated by use of normalized C-14 source term and scaling factors based on power generation. Surry Power Station utilized the methodology provided in EPRI Report 1021106, "Estimation of Carbon-14 in Nuclear Power Gaseous Effluents," to determine the production of C-14 based on Reactor Coolant System parameters and the capacity factor of each unit. This yielded an estimated 17.97 Ci of C-14 produced and released from SPS in 2025. Surry Power Station assumed that the fractional release of gaseous C-14 in any calendar quarter and pathway could be approximated by the fraction of noble gases released via that pathway in that quarter. Doses for C-14 via inhalation and consumption pathways were then determined using the standard methodology and equations provided Regulatory Guide 1.109.

Discussion

Carbon-14 is produced in the reactor coolant during power operation, and its production rate increases during the fuel cycle due to increasing neutron flux and ingress of nitrogen. Most C-14 species initially produced in a Pressurized Water Reactor (PWR) are organic, e.g., methane. C-14 releases for PWRs occur primarily as a mix of organic carbon and carbon dioxide released from the waste gas system. C-14 in the primary coolant is essentially all organic with a large fraction as a gaseous species. Any time the Reactor Coolant System liquid or gas is exposed to an oxidizing environment, a slow transformation from an organic to an inorganic chemical form can occur. Various studies documenting measured C-14 releases from PWRs suggest a range of 70% to 95% organic; therefore, Surry Power Station uses a value of 70% organic and 30% CO_2 in its calculations.

The basis for the calculation of the percent of technical specification in Table 2A of Attachment 1 is the ODCM, Section 6.2.1, which states that the concentration of radioactive material released in liquid effluents to unrestricted areas shall not exceed ten times the concentrations specified in 10CFR20, Appendix B, Table 2, Column 2, for radionuclides other than dissolved or entrained noble gases. For dissolved or entrained noble gases, the concentration shall be limited to 2.00E-04 Ci/mL.

Percent of technical specification calculations are based on the total gaseous or liquid effluents released for the respective quarter.

The annual and quarterly doses, as reported in Attachment 2, were calculated according to the methodology presented in the ODCM. The beta and gamma air doses due to noble gases released from the site were calculated at the site boundary. The maximum exposed member of the public from the release of airborne iodine-131, iodine-133, tritium and all radionuclides in particulate form with half-lives greater than 8 days, was modeled as a child at site boundary with the critical organ being the bone via the inhalation pathway. The maximum exposed member of the public from radioactive materials in liquid effluents in unrestricted areas was modeled as an adult at site boundary, exposed by either the invertebrate or fish pathway, with the critical organ typically being the gastrointestinal-lower large intestine (GI-LLI). The total body dose was also determined for this individual.

No effluent radiation monitors were inoperable for greater than 30 days in 2025. This is reported in Attachment 5 as required by the ODCM, Section 6.2.2 and 6.3.2.

There were no unplanned liquid releases and no unplanned gaseous releases in 2025. This is reported in Attachment 6 as required by the ODCM, Section 6.7.2.

The typical lower limit of detection (LLD) capabilities of the radioactive effluent analysis instrumentation are presented in Attachment 7. These LLD values are based upon conservative conditions (i.e., minimum sample volumes and maximum delay time prior to analysis). Actual LLD values may be lower. If a radioisotope was not detected when effluent samples were analyzed, then the activity of the radioisotope was reported as Not Detected (N/D) on Attachment 1 of this report. When all isotopes listed on Attachment 1 for a particular quarter and release mode are less than the lower limit of detection, then the totals for this period will be designated as Not Applicable (N/A).

Supplemental Information

Section 6.6.1 of the ODCM requires the identification of the cause(s) for the unavailability of milk, or if required, leafy vegetation samples, and the identification for obtaining replacement samples. Milk samples were available for collection during this reporting period. Therefore, leafy vegetation sampling was not required.

As required by the ODCM, Section 6.6.2, evaluation of the Land Use Census is made to determine if new sample location(s) must be added to the Radiological Environmental Monitoring Program. A review of the Land Use Census for this reporting period identified there were no changes to the sample locations used in the Radiological Environmental Monitoring Program.

EFFLUENT RELEASE DATA

January 1, 2025 through December 31, 2025

Attachment 1 provides a summary of the quantities of radioactive liquid and gaseous effluents and solid waste as outlined in Regulatory Guide 1.21, Appendix B.

TABLE 1A
EFFLUENT AND WASTE DISPOSAL ANNUAL
REPORT PERIOD: 1/1/25 TO 12/31/25
GASEOUS EFFLUENT-SUMMATION OF ALL RELEASES

SURRY POWER STATION UNITS 1&2	UNIT	FIRST QUARTER	SECOND QUARTER	% EST. ERROR
A. FISSION & ACTIVATION GASES				
1. TOTAL RELEASE	Ci	2.30E-03	1.15E-02	1.80E+01
2. AVE RELEASE RATE FOR PERIOD	µCi/sec	2.96E-04	1.46E-03	
B. IODINE				
1. TOTAL I-131	Ci	N/D	N/D	2.80E+01
2. AVE RELEASE RATE FOR PERIOD	µCi/sec	N/A	N/A	
C. PARTICULATE				
1. HALF-LIFE >8 DAYS	Ci	N/D	1.52E-08	2.80E+01
2. AVE RELEASE RATE FOR PERIOD	µCi/sec	N/A	1.93E-09	
3. GROSS ALPHA RADIOACTIVITY	Ci	N/D	N/D	
D. TRITIUM				
1. TOTAL RELEASE	Ci	2.22E+01	7.10E+00	3.10E+01
2. AVE RELEASE RATE FOR PERIOD	µCi/sec	2.86E+00	9.03E-01	
E. CARBON-14				
1. TOTAL RELEASE	Ci	5.61E-02	2.81E-01	
2. AVE RELEASE RATE FOR PERIOD	µCi/sec	7.21E-03	3.57E-02	
PERCENTAGE OF T.S. LIMITS				
CRITICAL ORGAN DOSE RATE	%	4.36E-03	1.38E-03	
TOTAL BODY DOSE RATE	%	4.18E-09	2.07E-08	
SKIN DOSE RATE	%	1.64E-09	8.13E-09	

TABLE 1A

EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT
PERIOD: 1/1/25 TO 12/31/25
GASEOUS EFFLUENT-SUMMATION OF ALL RELEASES

SURRY POWER STATION UNITS 1&2	UNIT	THIRD QUARTER	FOURTH QUARTER	% EST. ERROR
A. FISSION & ACTIVATION GASES				
1. TOTAL RELEASE	Ci	1.83E-02	7.04E-01	1.80E+01
2. AVE RELEASE RATE FOR PERIOD	µCi/sec	2.30E-03	8.86E-02	
B. IODINE				
1. TOTAL I-131	Ci	N/D	3.93E-09	2.80E+01
2. AVE RELEASE RATE FOR PERIOD	µCi/sec	N/A	4.94E-10	
C. PARTICULATE				
1. HALF-LIFE >8 DAYS	Ci	3.81E-05	5.81E-06	2.80E+01
2. AVE RELEASE RATE FOR PERIOD	µCi/sec	4.80E-06	7.31E-07	
3. GROSS ALPHA RADIOACTIVITY	Ci	N/D	N/D	
D. TRITIUM				
1. TOTAL RELEASE	Ci	9.59E+01	2.38E+01	3.10E+01
2. AVE RELEASE RATE FOR PERIOD	µCi/sec	1.21E+01	2.99E+00	
E. CARBON-14				
1. TOTAL RELEASE	Ci	4.46E-01	1.72E+01	
2. AVE RELEASE RATE FOR PERIOD	µCi/sec	5.61E-02	2.16E+00	
PERCENTAGE OF T.S. LIMITS				
CRITICAL ORGAN DOSE RATE	%	1.85E-02	6.41E-02	
TOTAL BODY DOSE RATE	%	1.90E-05	1.88E-03	
SKIN DOSE RATE	%	4.65E-06	4.58E-04	

TABLE 1B

EFFLUENT AND WASTE DISPOSAL ANNUAL
REPORT PERIOD: 1/1/25 TO 12/31/25
GASEOUS EFFLUENTS-MIXED MODE RELEASES

		CONTINUOUS MODE		BATCH MODE	
SURRY POWER STATION UNITS 1&2	UNIT	FIRST QUARTER	SECOND QUARTER	FIRST QUARTER	SECOND QUARTER
1. FISSION & ACTIVATION GASES					
Kr-85	Ci	N/D	N/D	N/D	N/D
Kr-85m	Ci	N/D	N/D	N/D	N/D
Kr-87	Ci	N/D	N/D	N/D	N/D
Kr-88	Ci	N/D	N/D	N/D	N/D
Xe-133	Ci	N/D	N/D	2.30E-03	1.15E-02
Xe-135	Ci	N/D	N/D	N/D	N/D
Xe-135m	Ci	N/D	N/D	N/D	N/D
Xe-138	Ci	N/D	N/D	N/D	N/D
Xe-131m	Ci	N/D	N/D	N/D	N/D
Xe-133m	Ci	N/D	N/D	N/D	N/D
Ar-41	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	N/A	N/A	2.30E-03	1.15E-02
2. IODINES					
I-131	Ci	N/D	N/D	N/D	N/D
I-133	Ci	N/D	N/D	N/D	N/D
I-135	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	N/A	N/A	N/A	N/A
3. PARTICULATES					
Sr-89	Ci	N/D	N/D	N/D	N/D
Sr-90	Ci	N/D	N/D	N/D	N/D
Cs-134	Ci	N/D	N/D	N/D	N/D
Cs-137	Ci	N/D	N/D	N/D	N/D
Ba-140	Ci	N/D	N/D	N/D	N/D
La-140	Ci	N/D	N/D	N/D	N/D
Co-58	Ci	N/D	1.52E-08	N/D	N/D
Co-60	Ci	N/D	N/D	N/D	N/D
Mn-54	Ci	N/D	N/D	N/D	N/D
Fe-59	Ci	N/D	N/D	N/D	N/D
Zn-65	Ci	N/D	N/D	N/D	N/D
Mo-99	Ci	N/D	N/D	N/D	N/D
Ce-141	Ci	N/D	N/D	N/D	N/D
Ce-144	Ci	N/D	N/D	N/D	N/D
C-14	Ci	N/D	N/D	5.61E-02	2.81E-01
TOTAL FOR PERIOD	Ci	N/A	1.52E-08	5.61E-02	2.81E-01

TABLE 1B

EFFLUENT AND WASTE DISPOSAL ANNUAL
REPORT PERIOD: 1/1/25 TO 12/31/25
GASEOUS EFFLUENTS-MIXED MODE RELEASES

		CONTINUOUS MODE		BATCH MODE	
SURRY POWER STATION UNITS 1&2	UNIT	THIRD QUARTER	FOURTH QUARTER	THIRD QUARTER	FOURTH QUARTER
1. FISSION & ACTIVATION GASES					
Kr-85	Ci	N/D	N/D	N/D	N/D
Kr-85m	Ci	N/D	N/D	8.47E-05	N/D
Kr-87	Ci	N/D	N/D	N/D	N/D
Kr-88	Ci	N/D	N/D	N/D	N/D
Xe-133	Ci	N/D	7.23E-02	9.36E-03	1.47E-01
Xe-135	Ci	N/D	N/D	3.31E-03	5.18E-04
Xe-135m	Ci	N/D	N/D	N/D	N/D
Xe-138	Ci	N/D	N/D	N/D	N/D
Xe-131m	Ci	N/D	N/D	N/D	N/D
Xe-133m	Ci	N/D	N/D	2.38E-04	N/D
Ar-41	Ci	N/D	N/D	5.48E-04	1.49E-03
TOTAL FOR PERIOD	Ci	N/A	7.23E-02	1.35E-02	1.49E-01
2. IODINES					
I-131	Ci	N/D	3.93E-09	N/D	N/D
I-133	Ci	N/D	N/D	N/D	N/D
I-135	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	N/A	3.93E-09	N/A	N/A
3. PARTICULATES					
Sr-89	Ci	N/D	N/D	N/D	N/D
Sr-90	Ci	N/D	N/D	N/D	N/D
Cs-134	Ci	N/D	N/D	N/D	N/D
Cs-137	Ci	N/D	N/D	N/D	N/D
Ba-140	Ci	N/D	N/D	N/D	N/D
La-140	Ci	N/D	N/D	N/D	N/D
Co-58	Ci	2.14E-08	2.75E-08	N/D	N/D
Co-60	Ci	N/D	N/D	N/D	N/D
Mn-54	Ci	N/D	N/D	N/D	N/D
Fe-59	Ci	N/D	N/D	N/D	N/D
Zn-65	Ci	N/D	N/D	N/D	N/D
Mo-99	Ci	N/D	N/D	N/D	N/D
Ce-141	Ci	N/D	N/D	N/D	N/D
Ce-144	Ci	N/D	N/D	N/D	N/D
C-14	Ci	N/D	1.76E+00	3.30E-01	3.65E+00
TOTAL FOR PERIOD	Ci	2.14E-08	1.76E+00	3.30E-01	3.65E+00

TABLE 1C

EFFLUENT AND WASTE DISPOSAL ANNUAL
REPORT PERIOD: 1/1/25 TO 12/31/25
GASEOUS EFFLUENTS-GROUND LEVEL RELEASES

SURRY POWER STATION UNITS 1&2	UNIT	CONTINUOUS MODE FIRST QUARTER	CONTINUOUS MODE SECOND QUARTER	BATCH MODE FIRST QUARTER	BATCH MODE SECOND QUARTER
1. FISSION & ACTIVATION GASES					
Kr-85	Ci	N/D	N/D	N/D	N/D
Kr-85m	Ci	N/D	N/D	N/D	N/D
Kr-87	Ci	N/D	N/D	N/D	N/D
Kr-88	Ci	N/D	N/D	N/D	N/D
Xe-133	Ci	N/D	N/D	N/D	N/D
Xe-135	Ci	N/D	N/D	N/D	N/D
Xe-135m	Ci	N/D	N/D	N/D	N/D
Xe-138	Ci	N/D	N/D	N/D	N/D
Xe-131m	Ci	N/D	N/D	N/D	N/D
Xe-133m	Ci	N/D	N/D	N/D	N/D
Ar-41	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	N/A	N/A	N/A	N/A
2. IODINES					
I-131	Ci	N/D	N/D	N/D	N/D
I-133	Ci	N/D	N/D	N/D	N/D
I-135	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	N/A	N/A	N/A	N/A
3. PARTICULATES					
Sr-89	Ci	N/D	N/D	N/D	N/D
Sr-90	Ci	N/D	N/D	N/D	N/D
Cs-134	Ci	N/D	N/D	N/D	N/D
Cs-137	Ci	N/D	N/D	N/D	N/D
Ba-140	Ci	N/D	N/D	N/D	N/D
La-140	Ci	N/D	N/D	N/D	N/D
Co-58	Ci	N/D	N/D	N/D	N/D
Co-60	Ci	N/D	N/D	N/D	N/D
Mn-54	Ci	N/D	N/D	N/D	N/D
Fe-59	Ci	N/D	N/D	N/D	N/D
Zn-65	Ci	N/D	N/D	N/D	N/D
Mo-99	Ci	N/D	N/D	N/D	N/D
Ce-141	Ci	N/D	N/D	N/D	N/D
Ce-144	Ci	N/D	N/D	N/D	N/D
Nb-95	Ci	N/D	N/D	N/D	N/D
Zr-95	Ci	N/D	N/D	N/D	N/D
C-14	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	N/A	N/A	N/A	N/A

TABLE 1C

EFFLUENT AND WASTE DISPOSAL ANNUAL
REPORT PERIOD: 1/1/25 TO 12/31/25
GASEOUS EFFLUENTS-GROUND LEVEL RELEASES

		CONTINUOUS MODE		BATCH MODE	
SURRY POWER STATION UNITS 1&2	UNIT	THIRD QUARTER	FOURTH QUARTER	THIRD QUARTER	FOURTH QUARTER
1. FISSION & ACTIVATION GASES					
Kr-85	Ci	N/D	N/D	N/D	N/D
Kr-85m	Ci	N/D	N/D	N/D	N/D
Kr-87	Ci	N/D	N/D	N/D	N/D
Kr-88	Ci	N/D	N/D	N/D	N/D
Xe-133	Ci	N/D	N/D	N/D	1.37E-02
Xe-135	Ci	N/D	N/D	N/D	N/D
Xe-135m	Ci	N/D	N/D	N/D	N/D
Xe-138	Ci	N/D	N/D	N/D	N/D
Xe-131m	Ci	N/D	N/D	N/D	N/D
Xe-133m	Ci	N/D	N/D	N/D	N/D
Ar-41	Ci	4.75E-03	4.69E-01	N/D	N/D
TOTAL FOR PERIOD	Ci	4.75E-03	4.69E-01	N/A	1.37E-02
2. IODINES					
I-131	Ci	N/D	N/D	N/D	N/D
I-133	Ci	N/D	N/D	N/D	N/D
I-135	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	N/A	N/A	N/A	N/A
3. PARTICULATES					
Sr-89	Ci	N/D	N/D	N/D	N/D
Sr-90	Ci	N/D	N/D	N/D	N/D
Cs-134	Ci	N/D	N/D	N/D	N/D
Cs-137	Ci	N/D	N/D	N/D	N/D
Ba-140	Ci	N/D	N/D	N/D	N/D
La-140	Ci	N/D	N/D	N/D	N/D
Co-58	Ci	3.81E-05	5.78E-06	N/D	N/D
Co-60	Ci	N/D	N/D	N/D	N/D
Mn-54	Ci	N/D	N/D	N/D	N/D
Fe-59	Ci	N/D	N/D	N/D	N/D
Zn-65	Ci	N/D	N/D	N/D	N/D
Mo-99	Ci	N/D	N/D	N/D	N/D
Ce-141	Ci	N/D	N/D	N/D	N/D
Ce-144	Ci	N/D	N/D	N/D	N/D
C-14	Ci	1.16E-01	1.14E+01	N/D	3.33E-01
TOTAL FOR PERIOD	Ci	1.16E-01	1.14E+01	N/A	3.33E-01

TABLE 2A

EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT
PERIOD: 1/1/24 TO 12/31/24
LIQUID EFFLUENTS-SUMMATION OF ALL RELEASES

SURRY POWER STATION UNITS 1&2	UNIT	FIRST QUARTER	SECOND QUARTER	% EST. ERROR
A. FISSION AND ACTIVATION PRODUCTS				
1. TOTAL RELEASE (NOT INCLUDING TRITIUM, GASES, ALPHA)	Ci	1.22E-03	1.89E-03	2.00E+01
2. AVE DIL. CONC. DURING PERIOD	µCi/mL	5.18E-13	7.91E-13	
3. PERCENT OF APPLICABLE LIMIT	%	2.18E-06	4.48E-06	
B. TRITIUM				
1. TOTAL RELEASE	Ci	4.69E+01	5.31E+01	2.00E+01
2. AVE DIL. CONC. DURING PERIOD	µCi/mL	1.99E-08	2.22E-08	
3. PERCENT OF APPLICABLE LIMIT	%	1.99E-04	2.22E-04	
C. DISSOLVED AND ENTRAINED GASES				
1. TOTAL RELEASE	Ci	N/D	N/D	2.00E+01
2. AVE DIL. CONC. DURING PERIOD	µCi/mL	N/A	N/A	
3. PERCENT OF APPLICABLE LIMIT	%	N/A	N/A	
D. GROSS ALPHA RADIOACTIVITY				
1. TOTAL RELEASE	Ci	N/D	N/D	2.00E+01
E. VOLUME OF WASTE RELEASED (PRIOR TO DILUTION)	LITERS	5.23E+07	5.29E+07	3.00E+00
F. VOLUME OF DILUTION WATER USED DURING PERIOD	LITERS	2.36E+12	2.39E+12	3.00E+00

TABLE 2A

EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT
PERIOD: 1/1/25 TO 12/31/25
LIQUID EFFLUENTS-SUMMATION OF ALL RELEASES

SURRY POWER STATION UNITS 1&2	UNIT	THIRD QUARTER	FOURTH QUARTER	% EST. ERROR
A. FISSION AND ACTIVATION PRODUCTS				
1. TOTAL RELEASE (NOT INCLUDING TRITIUM, GASES, ALPHA)	Ci	1.76E-03	1.65E-03	2.00E+01
2. AVE DIL. CONC. DURING PERIOD	µCi/mL	5.74E-13	1.06E-12	
3. PERCENT OF APPLICABLE LIMIT	%	1.42E-06	1.69E-06	
B. TRITIUM				
1. TOTAL RELEASE	Ci	1.21E+03	4.06E+02	2.00E+01
2. AVE DIL. CONC. DURING PERIOD	µCi/mL	3.95E-07	2.61E-07	
3. PERCENT OF APPLICABLE LIMIT	%	3.95E-03	2.61E-03	
C. DISSOLVED AND ENTRAINED GASES				
1. TOTAL RELEASE	Ci	3.92E-06	1.30E-04	2.00E+01
2. AVE DIL. CONC. DURING PERIOD	µCi/mL	1.28E-15	8.36E-14	
3. PERCENT OF APPLICABLE LIMIT	%	6.41E-10	4.18E-08	
D. GROSS ALPHA RADIOACTIVITY				
1. TOTAL RELEASE	Ci	N/D	N/D	2.00E+01
E. VOLUME OF WASTE RELEASED (PRIOR TO DILUTION)	LITERS	5.46E+07	5.40E+07	3.00E+00
F. VOLUME OF DILUTION WATER USED DURING PERIOD	LITERS	3.06E+12	1.56E+12	3.00E+00

TABLE 2B

EFFLUENT AND WASTE DISPOSAL ANNUAL
REPORT PERIOD: 1/1/25 TO 12/31/25
LIQUID EFFLUENTS

SURRY POWER STATION UNITS 1&2	UNIT	CONTINUOUS MODE		BATCH MODE	
		FIRST QUARTER	SECOND QUARTER	FIRST QUARTER	SECOND QUARTER
Sr-89	Ci	N/D	N/D	N/D	N/D
Sr-90	Ci	N/D	N/D	N/D	N/D
Fe-55	Ci	N/D	N/D	N/D	N/D
Cs-134	Ci	N/D	N/D	N/D	N/D
Cs-137	Ci	1.25E-04	1.12E-04	2.84E-04	8.01E-04
I-131	Ci	N/D	N/D	N/D	N/D
Co-58	Ci	N/D	N/D	3.10E-04	1.50E-04
Co-60	Ci	N/D	N/D	2.41E-04	4.10E-04
Fe-59	Ci	N/D	N/D	N/D	N/D
Zn-65	Ci	N/D	N/D	N/D	N/D
Mn-54	Ci	N/D	N/D	N/D	N/D
Cr-51	Ci	N/D	N/D	N/D	N/D
Zr-95	Ci	N/D	N/D	N/D	N/D
Nb-95	Ci	N/D	N/D	N/D	N/D
Mo-99	Ci	N/D	N/D	N/D	N/D
Tc-99m	Ci	N/D	N/D	N/D	N/D
Ba-140	Ci	N/D	N/D	N/D	N/D
La-140	Ci	N/D	N/D	N/D	N/D
Ce-141	Ci	N/D	N/D	N/D	N/D
Ce-144	Ci	N/D	N/D	N/D	N/D
Sb-124	Ci	N/D	N/D	N/D	N/D
Sb-125	Ci	N/D	N/D	2.62E-04	4.17E-04
Nd-147	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	1.25E-04	1.12E-04	1.10E-03	1.78E-03
Xe-133	Ci	N/D	N/D	N/D	N/D
Xe-135	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	N/A	N/A	N/A	N/A

TABLE 2B

EFFLUENT AND WASTE DISPOSAL ANNUAL
REPORT PERIOD: 1/1/225TO 12/31/25
LIQUID EFFLUENTS

SURRY POWER STATION UNITS 1&2	UNIT	CONTINUOUS MODE THIRD QUARTER	CONTINUOUS MODE FOURTH QUARTER	BATCH MODE THIRD QUARTER	BATCH MODE FOURTH QUARTER
Sr-89	Ci	N/D	N/D	N/D	N/D
Sr-90	Ci	N/D	N/D	N/D	N/D
Fe-55	Ci	N/D	N/D	N/D	N/D
Cs-134	Ci	N/D	N/D	N/D	N/D
Cs-137	Ci	1.21E-04	9.20E-05	1.37E-05	8.52E-07
I-131	Ci	N/D	N/D	N/D	N/D
Co-58	Ci	N/D	N/D	4.91E-04	3.41E-04
Co-60	Ci	N/D	N/D	7.95E-04	3.76E-04
Fe-59	Ci	N/D	N/D	N/D	N/D
Zn-65	Ci	N/D	N/D	N/D	N/D
Mn-54	Ci	N/D	N/D	N/D	N/D
Cr-51	Ci	N/D	N/D	N/D	N/D
Zr-95	Ci	N/D	N/D	N/D	N/D
Nb-95	Ci	N/D	N/D	N/D	N/D
Mo-99	Ci	N/D	N/D	N/D	N/D
Tc-99m	Ci	N/D	N/D	N/D	N/D
Ba-140	Ci	N/D	N/D	N/D	N/D
La-140	Ci	N/D	N/D	N/D	N/D
Ce-141	Ci	N/D	N/D	N/D	N/D
Ce-144	Ci	N/D	N/D	N/D	N/D
Sb-124	Ci	N/D	N/D	N/D	N/D
Sb-125	Ci	N/D	N/D	3.35E-04	8.41E-04
Nd-147	Ci	N/D	N/D	N/D	N/D
Co-57	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	1.21E-04	9.20E-05	1.63E-03	1.56E-03
Xe-133	Ci	N/D	N/D	3.92E-06	1.30E-04
Xe-135	Ci	N/D	N/D	N/D	N/D
TOTAL FOR PERIOD	Ci	N/A	N/A	3.92E-06	1.30E-04

TABLE 3

EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT

SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

PERIOD: 1/1/25 - 12/31/25

SURRY POWER STATION
A. SOLID WASTE SHIPPED OFFSITE FOR BURIAL OR DISPOSAL (Not irradiated fuel)

1. Type of waste		12 month Period	Est. Total Error, %
a. Spent resins, filter sludges, evaporator bottoms, etc.	m^3	8.53E+00	1.00E+01
	Ci	2.64E-03	3.00E+01
b. Dry compressible waste, contaminated equip., etc.	m^3	5.63E+02	1.00E+01
	Ci	1.63E-01	3.00E+01
c. Irradiated components, control rods, etc.	m^3	0.00E+00	1.00E+01
	Ci	0.00E+00	3.00E+01
d. Other	m^3	6.56E+00	1.00E+01
	Ci	1.86E-04	3.00E+01

2. Estimate of major nuclide composition (by type of waste)

a.	H-3	%	9.80E+01
	Sr-90	%	1.75E+00
b.	Mn-54	%	1.13E+00
	Fe-55	%	1.59E+01
	Co-58	%	8.88E+00
	Co-60	%	6.33E+01
	Ni-63	%	4.12E+00
	Nb-95	%	1.39E+00
	Sb-125	%	1.08E+00
c.	n/a	%	n/a
d.	C-14	%	9.89E+01

TABLE 3

EFFLUENT AND WASTE DISPOSAL ANNUAL REPORT

SOLID WASTE AND IRRADIATED FUEL SHIPMENTS

PERIOD: 1/1/25 - 12/31/25

SURRY POWER STATION

A. SOLID WASTE SHIPPED OFFSITE FOR BURIAL OR DISPOSAL (Not irradiated fuel)

3. Solid Waste Disposition

Number of Shipments	Mode of Transportation	Destination
10	Truck	EnergySolutions at Oak Ridge, TN (Bear Creek Operations)
9	Truck	EnergySolutions at Memphis, TN (Memphis Facility)

B. IRRADIATED FUEL SHIPMENT (Disposition)

Number of Shipments	Mode of Transportation	Destination
0		

ANNUAL AND QUARTERLY DOSES

An assessment of radiation doses to the maximum exposed member of the public due to radioactive liquid and gaseous effluents released from the site for each calendar quarter for the calendar year of this report, along with an annual total of each effluent pathway is made pursuant to the ODCM, Section 6.7.2, requirement.

2025	LIQUID		
	Maximum Receptor - Adult		
	Total Body (mrem)	GI-LLI (mrem)	Liver (mrem)
1st Quarter	1.38E-05	2.03E-05	1.45E-05
2nd Quarter	1.75E-05	2.28E-05	1.96E-05
3rd Quarter	2.14E-04	2.31E-04	2.13E-04
4th Quarter	1.25E-04	1.42E-04	1.25E-04
Annual	3.70E-04	4.15E-04	3.71E-04

2025	GASEOUS - Air Dose	
	Gamma (mrad)	Beta (mrad)
1st Quarter	6.18E-09	1.84E-08
2nd Quarter	3.09E-08	9.20E-08
3rd Quarter	2.53E-05	9.04E-06
4th Quarter	2.49E-03	8.88E-04
Annual	2.52E-03	8.97E-04

2025	GASEOUS - Organ Dose		
	Annual Maximum	Maximum by Quarter	
	Child / Bone (mrem)	(mrem)	Receptor / Organ
1st Quarter	1.15E-05	1.61E-02	Teen/Liver, Thy, Kid, Lung, GI-LLI
2nd Quarter	5.76E-05	5.15E-03	Teen/Liver, Thy, Kid, Lung, GI-LLI
3rd Quarter	2.45E-03	6.98E-02	Teen / Lung
4th Quarter	2.42E-01	2.42E-01	Child / Bone
Annual	2.45E-01		

REVISIONS TO OFFSITE DOSE CALCULATION MANUAL (ODCM)

As required by Technical Specification 6.8.B, revisions to the ODCM, effective for the time period covered by this report, are included with this attachment. There were no revisions to the ODCM implemented during this reporting period.

MAJOR CHANGES TO RADIOACTIVE LIQUID, GASEOUS AND SOLID WASTE TREATMENT SYSTEMS

There were no major changes to the radioactive liquid, gaseous or solid waste treatment systems during the reporting period.

INOPERABILITY OF RADIOACTIVE LIQUID AND GASEOUS EFFLUENT MONITORING INSTRUMENTATION

The Annual Radioactive Effluent Release Report shall explain why monitoring instrumentation required by Attachments 1 and 5 of the ODCM were determined to be inoperable and were not returned to operable status within 30 days.

No radiation monitors as referenced on Attachment 1 and Attachment 5 of the ODCM were inoperable greater than 30 days during the reporting period.

UNPLANNED RELEASES

In accordance with the ODCM reporting requirements, no unplanned liquid releases or unplanned gaseous releases occurred during the reporting period.

LOWER LIMIT OF DETECTION (LLD) FOR EFFLUENT SAMPLE ANALYSIS

	Isotope	Required LLD	Typical LLD
GASEOUS:	Kr-87	1.00E-04	2.96E-06 - 2.23E-05
	Kr-88	1.00E-04	2.06E-06 - 2.01E-05
	Xe-133	1.00E-04	9.66E-07 - 1.32E-05
	Xe-133m	1.00E-04	4.48E-06 - 4.06E-05
	Xe-135	1.00E-04	5.36E-07 - 7.71E-06
	Xe-135m	1.00E-04	1.96E-05 - 8.06E-05
	Xe-138	1.00E-04	2.72E-05 - 9.90E-05
	I-131	1.00E-12	4.06E-13 - 4.06E-13
	I-133	1.00E-10	4.06E-11 - 4.06E-11
	Sr-89	1.00E-11	1.20E-14 - 2.31E-12
	Sr-90	1.00E-11	2.68E-15 - 5.43E-13
	Cs-134	1.00E-11	2.20E-14 - 3.10E-13
	Cs-137	1.00E-11	1.51E-13 - 4.86E-13
	Mn-54	1.00E-11	1.79E-13 - 4.02E-13
	Fe-59	1.00E-11	2.76E-13 - 7.26E-13
	Co-58	1.00E-11	1.57E-13 - 4.28E-13
	Co-60	1.00E-11	2.13E-14 - 5.85E-13
	Zn-65	1.00E-11	3.97E-13 - 8.97E-13
	Mo-99	1.00E-11	4.06E-12 - 4.06E-12
	Ce-141	1.00E-11	1.67E-13 - 4.17E-13
	Ce-144	1.00E-11	4.79E-13 - 1.63E-12
	Alpha	1.00E-11	1.67E-14 - 1.68E-14
	Tritium	1.00E-06	4.89E-08 - 5.77E-08
LIQUID:	Sr-89	5.00E-08	3.82E-08 - 3.03E-07
	Sr-90	5.00E-08	9.93E-09 - 1.81E-08
	Cs-134	5.00E-07	2.43E-09 - 5.22E-08
	Cs-137	5.00E-07	5.58E-09 - 7.82E-08
	I-131	1.00E-06	9.56E-09 - 6.23E-08
	Co-58	5.00E-07	3.54E-09 - 5.98E-08
	Co-60	5.00E-07	2.70E-09 - 8.63E-08
	Fe-59	5.00E-07	1.91E-08 - 1.12E-07
	Zn-65	5.00E-07	2.00E-08 - 1.69E-07
	Mn-54	5.00E-07	1.14E-08 - 6.80E-08
	Mo-99	5.00E-07	4.95E-07 - 4.95E-07
	Ce-141	5.00E-07	2.50E-08 - 7.64E-08
	Ce-144	5.00E-07	8.15E-08 - 3.47E-07
	Fe-55	1.00E-06	2.58E-07 - 7.64E-07
	Alpha	1.00E-07	2.29E-08 - 2.43E-08
	Tritium	1.00E-05	1.21E-06 - 1.43E-06
	Xe-133	1.00E-05	5.67E-08 - 1.65E-07
	Xe-135	1.00E-05	2.65E-08 - 4.31E-08
	Xe-133m	1.00E-05	1.63E-07 - 3.83E-07
	Xe-135m	1.00E-05	1.39E-06 - 3.05E-06
	Xe-138	1.00E-05	2.39E-06 - 6.99E-06
	Kr-87	1.00E-05	1.32E-07 - 2.26E-07
	Kr-88	1.00E-05	1.04E-07 - 2.17E-07

INDUSTRY GROUND WATER PROTECTION INITIATIVE

The following is a summary of 2025 sample analyses of ground water monitoring wells that are not a part of the Radiological Environmental Monitoring Program (REMP).

Well Designation	Sample Date	Tritium pCi/Liter	Gamma pCi/Liter	Fe-55 pCi/Liter	Ni-63 pCi/Liter	Sr-90 pCi/Liter	TRU pCi/Liter
1-PL-Piez-08	1/14/25	0	NA	NA	NA	NA	NA
1-PL-Piez-23	1/14/25	538	NA	NA	NA	NA	NA
1-PL-Piez-41	1/14/25	664	NA	NA	NA	NA	NA
G-08	2/6/25	8600	NA	NA	NA	NA	NA
1-PL-Piez-44	2/6/25	3550	NA	NA	NA	NA	NA
G-08	3/5/25	9780	NA	NA	NA	NA	NA
1-PL-Piez-44	3/12/25	2450	ND	<169.3	<4.77	<0.84	NA
1-PL-Piez-46	3/12/25	475	ND	<86.13	<4.65	<0.91	NA
1-PL-Piez-49	3/12/25	2250	ND	<44.76	<4.96	<0.82	NA
1-PL-Piez-44	3/12/25	2150	NA	NA	NA	NA	NA
1-PL-Piez-49	3/12/25	1840	NA	NA	NA	NA	NA
1-PL-Piez-46	3/12/25	<1070	NA	NA	NA	NA	NA
1-PL-Piez-25	3/19/25	5500	NA	NA	NA	NA	NA
1-PL-Piez-48	3/19/25	535	NA	NA	NA	NA	NA
1-PL-Piez-50	3/19/25	671	NA	NA	NA	NA	NA
1-PL-Piez-52	3/19/25	135	NA	NA	NA	NA	NA
1-PL-Piez-48	3/19/25	<1290	ND	NA	NA	NA	NA
1-PL-Piez-52	3/19/25	<1270	ND	NA	NA	NA	NA
1-PL-Piez-29	3/19/25	1790	ND	<31.95	<4.83	<0.83	NA
1-PL-Piez-50	3/19/25	<1290	ND	NA	NA	NA	NA
1-PL-Piez-25	3/19/25	<1270	NA	NA	NA	NA	NA
1-PL-Piez-04	3/20/25	0	ND	NA	NA	NA	NA
1-PL-Piez-05	3/20/25	2780	ND	<99.63	<3.81	<0.93	NA
1-PL-Piez-43	3/20/25	304	NA	NA	NA	NA	NA
1-PL-Piez-04	3/20/25	<1270	NA	NA	NA	NA	NA
1-PL-Piez-05	3/20/25	1990	NA	NA	NA	NA	NA
1-PL-Piez-43	3/20/25	<1290	ND	NA	NA	NA	NA
1-PL-Piez-06	3/25/25	252	NA	NA	NA	NA	NA
1-PL-Piez-29	3/25/25	2130	NA	NA	NA	NA	NA
1-PL-Piez-45	3/25/25	532	NA	NA	NA	NA	NA
1-PL-Piez-06	3/25/25	<958	ND	<111.7	<4.28	<0.85	NA
1-PL-Piez-45	3/25/25	<958	ND	<148.7	<4.58	<0.73	NA
1-PL-Piez-47	3/26/25	2530	ND	NA	NA	NA	NA

NA = analysis not required.
ND = No non-natural gamma emitting nuclides detected when analyzed to REMP LLDs.
TRU = Transuranics (Am-241, Cm-242, Cm-243/244, Pu-238, Pu-239/240 and Pu-241)

INDUSTRY GROUND WATER PROTECTION INITIATIVE

Well Designation	Sample Date	Tritium pCi/Liter	Gamma pCi/Liter	Fe-55 pCi/Liter	Ni-63 pCi/Liter	Sr-90 pCi/Liter	TRU pCi/Liter
1-PL-Piez-47	3/26/25	<959	ND	<77.87	<4.44	<0.89	NA
G-08	4/2/25	9610	NA	NA	NA	NA	NA
1-PL-Piez-44	4/2/25	2400	NA	NA	NA	NA	NA
1-PL-Piez-49	4/2/25	1880	NA	NA	NA	NA	NA
G-08	4/16/25	7530	NA	NA	NA	NA	NA
1-PL-Piez-44	4/16/25	4610	NA	NA	NA	NA	NA
1-PL-Piez-25	4/17/25	0	NA	NA	NA	NA	NA
1-PL-Piez-49	4/17/25	2180	NA	NA	NA	NA	NA
G-08	4/29/25	7250	NA	NA	NA	NA	NA
1-PL-Piez-44	4/29/25	3780	NA	NA	NA	NA	NA
1-PL-Piez-49	4/29/25	2160	NA	NA	NA	NA	NA
G-08	5/20/25	5790	NA	NA	NA	NA	NA
1-PL-Piez-33	5/20/25	289	NA	NA	NA	NA	NA
1-PL-Piez-41	5/20/25	247	NA	NA	NA	NA	NA
1-PL-Piez-42	5/20/25	246	NA	NA	NA	NA	NA
1-PL-Piez-44	5/20/25	2980	NA	NA	NA	NA	NA
1-PL-Piez-49	5/20/25	4020	NA	NA	NA	NA	NA
G-08	5/21/25	5790	NA	NA	NA	NA	NA
1-PL-Piez-33	5/21/25	289	NA	NA	NA	NA	NA
1-PL-Piez-41	5/21/25	247	NA	NA	NA	NA	NA
1-PL-Piez-42	5/21/25	246	NA	NA	NA	NA	NA
1-PL-Piez-44	5/21/25	2980	NA	NA	NA	NA	NA
1-PL-Piez-49	5/21/25	4020	NA	NA	NA	NA	NA
1-PL-Piez-05	6/3/25	2490	NA	NA	NA	NA	NA
1-PL-Piez-06	6/3/25	444	ND	NA	NA	NA	NA
1-PL-Piez-43	6/3/25	527	NA	NA	NA	NA	NA
1-PL-Piez-44	6/3/25	3640	ND	NA	NA	NA	NA
1-PL-Piez-47	6/3/25	2130	NA	NA	NA	NA	NA
1-PL-Piez-50	6/3/25	550	ND	NA	NA	NA	NA
1-PL-Piez-45	6/12/26	758	ND	NA	NA	NA	NA
1-PL-Piez-29	6/12/25	3910	ND	NA	NA	NA	NA
1-PL-Piez-52	6/12/25	472	ND	NA	NA	NA	NA
1-PL-Piez-48	6/12/25	5400	ND	NA	NA	NA	NA

NA = Analysis not required.
ND = No non-natural gamma emitting nuclides detected when analyzed to REMP LLDs.
TRU = Transuranics (Am-241, Cm-242, Cm-243/244, Pu-238, Pu-239/240 and Pu-241)

INDUSTRY GROUND WATER PROTECTION INITIATIVE

Well Designation	Sample Date	Tritium pCi/Liter	Gamma pCi/Liter	Fe-55 pCi/Liter	Ni-63 pCi/Liter	Sr-90 pCi/Liter	TRU pCi/Liter
1-PL-Piez-49	6/12/25	2630	ND	NA	NA	NA	NA
1-PL-Piez-46	6/12/25	668	ND	NA	NA	NA	NA
1-PL-Piez-29	6/30/25	3910	NA	NA	NA	NA	NA
1-PL-Piez-45	6/30/25	758	NA	NA	NA	NA	NA
1-PL-Piez-46	6/30/25	668	NA	NA	NA	NA	NA
1-PL-Piez-48	6/30/25	5400	NA	NA	NA	NA	NA
1-PL-Piez-49	6/30/25	2630	NA	NA	NA	NA	NA
1-PL-Piez-52	6/30/25	472	NA	NA	NA	NA	NA
1-PL-Piez-48	7/8/25	755	NA	NA	NA	NA	NA
1-PL-Piez-29	7/17/25	2800	NA	NA	NA	NA	NA
1-PL-Piez-46	7/17/25	390	NA	NA	NA	NA	NA
G-08	8/11/25	6350	NA	NA	NA	NA	NA
1-PL-Piez-44	8/11/25	3220	NA	NA	NA	NA	NA
G-08	8/25/25	5510	NA	NA	NA	NA	NA
1-PL-Piez-44	8/25/25	2780	NA	NA	NA	NA	NA
1-PL-Piez-46	9/8/25	340	ND	NA	NA	NA	NA
1-PL-Piez-49	9/8/25	4960	ND	NA	NA	NA	NA
1-PL-Piez-06	9/9/25	926	ND	NA	NA	NA	NA
1-PL-Piez-43	9/9/25	364	ND	NA	NA	NA	NA
G-08	9/23/25	7900	NA	NA	NA	NA	NA
1-PL-Piez-44	9/23/25	0	NA	NA	NA	NA	NA
1-PL-Piez-49	9/23/25	2520	NA	NA	NA	NA	NA
1-PL-Piez-04	9/25/25	309	ND	NA	NA	NA	NA
1-PL-Piez-05	9/25/25	3110	ND	NA	NA	NA	NA
1-PL-Piez-44	9/25/25	2210	ND	NA	NA	NA	NA
1-PL-Piez-45	9/25/25	1150	ND	NA	NA	NA	NA
1-PL-Piez-47	9/29/25	2465	ND	NA	NA	NA	NA
1-PL-Piez-50	9/29/25	436	ND	NA	NA	NA	NA
1-PL-Piez-07	9/30/25	<1185	ND	NA	NA	NA	NA
1-PL-Piez-20	9/30/25	1322	NA	NA	NA	NA	NA
1-PL-Piez-25	9/30/25	465	NA	NA	NA	NA	NA
1-PL-Piez-27	9/30/25	75	NA	NA	NA	NA	NA
1-PL-Piez-29	9/30/25	2400	ND	NA	NA	NA	NA
1-PL-Piez-48	9/30/25	171	ND	NA	NA	NA	NA
1-PL-Piez-52	9/30/25	107	ND	NA	NA	NA	NA
1-PL-Piez-34	10/2/25	433	NA	NA	NA	NA	NA
1-PL-Piez-36	10/2/25	260	NA	NA	NA	NA	NA
1-PL-Piez-37	10/2/25	65	NA	NA	NA	NA	NA
1-PL-Piez-38	10/2/25	119	NA	NA	NA	NA	NA

NA = Analysis not required.
ND = No non-natural gamma emitting nuclides detected when analyzed to REMP LLDs.
TRU = Transuranics (Am-241, Cm-242, Cm-243/244, Pu-238, Pu-239/240 and Pu-241)

INDUSTRY GROUND WATER PROTECTION INITIATIVE

Well Designation	Sample Date	Tritium pCi/Liter	Gamma pCi/Liter	Fe-55 pCi/Liter	Ni-63 pCi/Liter	Sr-90 pCi/Liter	TRU pCi/Liter
1-PL-Piez-39	10/2/25	375	NA	NA	NA	NA	NA
1-PL-Piez-53	10/2/25	0	NA	NA	NA	NA	NA
1-PL-Piez-03	10/9/25	380	NA	NA	NA	NA	NA
G-08	10/21/25	2130	NA	NA	NA	NA	NA
1-PL-Piez-44	10/21/25	1930	NA	NA	NA	NA	NA
1-PL-Piez-49	10/21/25	4690	NA	NA	NA	NA	NA
G-08	11/19/25	5300	NA	NA	NA	NA	NA
1-PL-Piez-23	11/19/25	839	NA	NA	NA	NA	NA
1-PL-Piez-33	11/19/25	824	NA	NA	NA	NA	NA
1-PL-Piez-40	11/19/25	0	NA	NA	NA	NA	NA
1-PL-Piez-41	11/19/25	630	NA	NA	NA	NA	NA
1-PL-Piez-42	11/19/25	147	NA	NA	NA	NA	NA
1-PL-Piez-44	11/19/25	2520	NA	NA	NA	NA	NA
1-PL-Piez-51	11/19/25	5400	NA	NA	NA	NA	NA
1-PL-Piez-09	11/24/25	844	NA	NA	NA	NA	NA
1-PL-Piez-22	11/24/25	620	NA	NA	NA	NA	NA
1-PL-Piez-28	11/24/25	236	NA	NA	NA	NA	NA
1-PL-Piez-06	11/25/25	120	ND	NA	NA	NA	NA
1-PL-Piez-43	11/25/25	0	ND	NA	NA	NA	NA
1-PL-Piez-24	11/27/25	560	NA	NA	NA	NA	NA
1-PL-Piez-47	11/27/25	2510	ND	NA	NA	NA	NA
1-PL-Piez-50	11/27/25	511	ND	NA	NA	NA	NA
1-PL-Piez-05	12/4/25	1670	ND	NA	NA	NA	NA
1-PL-Piez-29	12/4/25	1560	ND	NA	NA	NA	NA
1-PL-Piez-44	12/4/25	3400	ND	NA	NA	NA	NA
1-PL-Piez-45	12/4/25	1460	ND	NA	NA	NA	NA
1-PL-Piez-48	12/4/25	0	ND	NA	NA	NA	NA
1-PL-Piez-51	12/4/25	2980	NA	NA	NA	NA	NA
1-PL-Piez-52	12/4/25	0	ND	NA	NA	NA	NA
1-PL-Piez-08	12/12/25	0	NA	NA	NA	NA	NA
1-PL-Piez-20	12/12/25	0	NA	NA	NA	NA	NA

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TRU = Transuranics (Am-241, Cm-242, Cm-243/244, Pu-238, Pu-239/240 and Pu-241)