



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

April 15, 2026

Jamie M. Coleman
Regulatory Affairs Director
Southern Nuclear Operating Company
3535 Colonnade Parkway
Birmingham, AL 35243

SUBJECT: VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2 – CORRECTION
OF UNIT 1 AMENDMENT NO. 230 AND UNIT 2 AMENDMENT NO. 212
RE: ISSUANCE OF AMENDMENTS REGARDING ALTERNATIVE SOURCE
TERM USING REGULATORY GUIDE 1.183, REVISION 1
(EPID L-2025-LLA-0080)

Dear Ms. Coleman:

On March 26, 2026, the U.S. Nuclear Regulatory Commission (NRC) issued Amendments No. 230 to Renewed Facility Operating License (RFOL) NPF-68 and Amendment No. 212 to RFOL NPF-81 to Southern Nuclear Operating Company (SNC, the licensee) for the Vogtle Electric Generating Plant (Vogtle), Units 1 and 2, respectively (Agencywide Documents Access and Management System (ADAMS) Accession No. ML26062A818). This license amendment allowed SNC to revise the licensing basis to support a full scope application of an Alternative Source Term (AST) methodology consistent with the guidance of Regulatory Guide (RG) 1.183, Revision 1, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors."

In the letter dated March 26, 2026, in section 2.2, "AST Regulatory Evaluation," of the safety evaluation (SE), it states, in part:

Vogtle, Units 1 and 2, current [design basis accident] DBA radiological consequence analyses are based on the source term from U.S. Atomic Energy Commission Technical Information Document (TID)-14844, "Calculation of Distance Factors for Power and Test Reactor Sites," dated March 23, 1962 (ML021720780).

By email dated March 31, 2026 (ML26096A025), SNC pointed out that this reference is incorrect.

By letter dated July 31, 2023 (ML23158A018), the NRC staff issued Amendment Nos. 219 and 202 for Vogtle, Units 1 and 2, respectively. Amendment Nos. 219 and 202 revised the licensing basis to support a selective scope application of the Alternative Source Term (AST) radiological analysis methodology and modified TS 1.1, "Definitions," TS 3.3.6, "Containment Ventilation Isolation Instrumentation," TS 3.4.16, "RCS [Reactor Coolant System] Specific Activity," TS 3.9.1, "Boron Concentration," TS 3.9.2, "Unborated Water Source Isolation Valves," TS 3.9.3, "Nuclear Instrumentation," and TS 3.9.4, "Containment Penetrations," consistent with Technical

Specifications Task Force (TSTF) Travelers TSTF-51-A, Revision 2, "Revise containment requirements during handling irradiated fuel and core alterations," TSTF-471-A, Revision 1, "Eliminate use of term CORE ALTERATIONS in ACTIONS and Notes," and TSTF-490-A, Revision 0, "Deletion of E Bar Definition and Revision to RCS Specific Activity Tech Spec."

Amendment Nos. 219 and 202 revised the licensing basis to use an AST in evaluating the offsite and Control Room (CR) radiological consequences of the Vogtle, Units 1 and 2, design basis accidents (DBAs) as allowed by Title 10 of the Code of Federal Regulations (10 CFR) Section 50.67, "Accident source term," and described in NRC RG)1.183, Revision 0, "Alternate Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors," July 2000 ((ML003716792).

Amendment Nos. 219 and 202 safety evaluation states:

SNC has performed its evaluation based on full implementation of the AST as defined in RG 1.183, with the exception of the equipment qualification (EQ). The licensee has determined that the current TID-14844 AST will remain the licensing basis for EQ.

...

Based on the above, the NRC staff finds that it is acceptable for the TID-14844 accident source term to remain the licensing basis for EQ at Vogtle, Units 1 and 2.

The NRC staff has confirmed that the reference in section 2.2 of the SE dated March 26, 2026, as approved by Amendment Nos. 230 and 212, for Vogtle, Units 1 and 2, respectively, needed more context and should be replaced with the correct reference.

Therefore, the correct reference in section 2.2 of the SE dated March 26, 2026, is the following:

Vogtle, Units 1 and 2, current DBA radiological consequence analyses are based on the source term from NRC RG 1.183, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors," July 2000 (ML003716792) with the exception of the equipment qualification (EQ) that remains Technical Information Document (TID)-14844, "Calculation of Distance Factors for Power and Test Reactor Sites," dated March 23, 1962 (ML021720780).

This administrative correction is addressed by this letter providing the more accurate reference in section 2.2 of the SE dated March 26, 2026.

Consistent with NRC staff guidance dated January 16, 1997 (ML103260096), based on the NRC's policy established by SECY-96-238 (ML20134M324) and the corresponding Staff Requirements Memorandum (ML003754054), this more accurate reference can be corrected by a letter to the licensee from the NRC staff.

Therefore, above is the correct reference for section 2.2 of the SE dated March 26, 2026, for Vogtle, Units 1 and 2. Since Amendment Nos. 230 and 212 for Vogtle, Units 1 and 2, respectively. There are no RFOL pages changed or technical specification pages changed.

J. Coleman

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This correction does not change any of the conclusions in the SE or no significant hazards consideration determination associated with Amendment Nos. 230 or 212 as published in the *Federal Register*.

Enclosed is the corrected SE page.

If you have any questions, please contact me at 301-415-3100 or John.Lamb@nrc.gov.

Sincerely,

/RA/

John G. Lamb, Senior Project Manager
Plant Licensing Branch II-1
Division of Operating Reactor Licensing
Office of Nuclear Reactor Regulation

Docket Nos. 50-424 and 50-425

Enclosure: As stated

cc: Listserv

2.2 AST Regulatory Evaluation

SNC's request was submitted pursuant to 10 CFR 50.67, "Accident source term," which provides a mechanism for licensed power reactors to replace the traditional source term used in the radiological consequence analyses of DBAs discussed in Section 3.0 below, as described in NRC RG 1.183, Rev. 1. Vogtle, Units 1 and 2, current DBA radiological consequence analyses are based on the source term from NRC RG 1.183, "Alternative Radiological Source Terms for Evaluating Design Basis Accidents at Nuclear Power Reactors," July 2000 (ML003716792) with the exception of the equipment qualification (EQ) that remains Technical Information Document (TID)-14844, "Calculation of Distance Factors for Power and Test Reactor Sites," dated March 23, 1962 (ML021720780).

The NRC staff evaluated SNC's analysis of the radiological consequences of the affected DBAs for implementation of the AST methodology, against the radiological dose requirements specified in 10 CFR 50.67(b)(2), and dose limits specified in 10 CFR Part 50, Appendix A, "General Design Criteria (GDC) for Nuclear Power Plants," Criterion 19, "Control Room."

Section 50.67(b)(2) states:

The NRC may issue the amendment only if the applicant's analysis demonstrates with reasonable assurance that:

- (iii) An individual located at any point on the boundary of the exclusion area for any 2-hour period following the onset of the postulated fission product release, would not receive a radiation dose in excess of 0.25 Sv (25 rem)^[1] total effective dose equivalent (TEDE).
- (iii) An individual located at any point on the outer boundary of the low population zone, who is exposed to the radioactive cloud resulting from the postulated fission product release (during the entire period of its passage), would not receive a radiation dose in excess of 0.25 Sv (25 rem) TEDE.
- (iii) Adequate radiation protection is provided to permit access to and occupancy of the control room under accident conditions without personnel receiving radiation exposures in excess of 0.05 Sv (5 rem) total effective dose equivalent (TEDE) for the duration of the accident.

The NRC staff's AST evaluation is based on the following regulations, RGs, and standards:

- The regulations in 10 CFR 50.67, "Accident source term,"
- The regulations in 10 CFR Part 50, Appendix A, "General Design Criteria for Nuclear Power Plants," GDC 19, "Control room,"
- NRC RG 1.145, "Atmospheric Dispersion Models for Potential Accident Consequence Assessments at Nuclear Power Plants," Revision 1, November 1982 (Reissued February 1983) (ML003740205),
- The regulation 10 CFR 50.49, "Environmental qualification of electric equipment important to safety for nuclear power plants,"

[1] The use of 0.25 SV (25 rem) TEDE is not intended to imply that this value constitutes an acceptable limit for emergency doses to the public under accident conditions. Rather, this 0.25 Sv (25 rem) TEDE value has been stated in this section as a Reference value, which can be used in the evaluation of proposed design basis changes with respect to potential reactor accidents of exceedingly low probability of occurrence and low risk of public exposure to radiation.

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ADAMS Accession Nos.:

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ML26096A225 (Letter)

e-Concurrence case: 20260406-70001