



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
REGION IV  
1600 EAST LAMAR BOULEVARD  
ARLINGTON, TEXAS 76011-4511

January 23, 2026

Cleve Reasoner, Chief Executive Officer  
and Chief Nuclear Officer  
Wolf Creek Nuclear Operating Corp.  
P.O. Box 411  
Burlington, KS 66839

SUBJECT: WOLF CREEK GENERATING STATION – INTEGRATED INSPECTION  
REPORT 05000482/2025004

Dear Cleve Reasoner:

On December 31, 2025, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Wolf Creek Generating Station. On January 8, 2026, the NRC inspectors discussed the results of this inspection with Robert Bayer, Vice President of Engineering, and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, “Public Inspections, Exemptions, Requests for Withholding.”

Sincerely,

Signed by Agrawal, Ami  
on 01/23/26

Ami N. Agrawal, Chief  
Reactor Projects Branch B  
Division of Operating Reactor Safety

Docket No. 05000482  
License No. NPF-42

Enclosure:  
As stated

cc w/ encl: Distribution via GovDelivery

WOLF CREEK GENERATING STATION – INTEGRATED INSPECTION REPORT  
05000482/2025004 – DATED JANUARY 23, 2026

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DOCUMENT NAME: WOLF CREEK GENERATING STATION – INTEGRATED INSPECTION REPORT  
05000482/2025004

ADAMS ACCESSION NUMBER: **ML26016A522**

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**U.S. NUCLEAR REGULATORY COMMISSION**  
**Inspection Report**

Docket Number: 05000482

License Number: NPF-42

Report Number: 05000482/2025004

Enterprise Identifier: I-2025-004-0006

Licensee: Wolf Creek Nuclear Operating Corp.

Facility: Wolf Creek Generating Station

Location: Burlington, KS

Inspection Dates: October 1, 2025 to December 31, 2025

Inspectors: J. Drake, Senior Reactor Inspector  
D. Grau, Health Physicist  
C. Henderson, Senior Resident Inspector  
J. O'Donnell, Senior Health Physicist  
C. Sigel, Resident Inspector

Approved By: Ami N. Agrawal, Chief  
Reactor Projects Branch B  
Division of Operating Reactor Safety

Enclosure

## SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Wolf Creek Generating Station, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

### List of Findings and Violations

No findings or violations of more than minor significance were identified.

### Additional Tracking Items

Type	Issue Number	Title	Report Section	Status
LER	05000482/2025-001-00	Improperly Breached Control Building Doors Cause a Condition Prohibited by Technical Specifications	71153	Closed
LER	05000482/2025-003-00	Loss of Voltage on ESF [Engineered Safety Feature] Bus Results in Automatic Actuation of Emergency Diesel Generator	71153	Closed

## PLANT STATUS

Wolf Creek Generating Station began the inspection period with a power coast down to refueling outage 27. On October 2, 2025, the unit was shut down to begin refueling outage 27. On November 14, 2025, the unit commenced reactor startup, and the reactor was made critical. The unit returned to full power on November 19, 2025, where it remained for the rest of the inspection period.

## INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors performed activities described in IMC 2515, Appendix D, "Plant Status," observed risk-significant activities, and completed onsite portions of IPs. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

## REACTOR SAFETY

### 71111.04 - Equipment Alignment

#### Partial Walkdown Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

- (1) trains A and B residual heat removal shutdown cooling alignment during refueling outage on October 19, 2025
- (2) reactor coolant system pressure control system and pressurizer spray line on November 3, 2025
- (3) trains A and B containment spray system following containment integrated leak rate testing on December 15, 2025

### 71111.05 - Fire Protection

#### Fire Area Walkdown and Inspection Sample (IP Section 03.01) (2 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

- (1) main transformer deluge testing, fire area yard, on November 3, 2025
- (2) condensate pump C, fire area "Turb", on November 21, 2025

## 71111.08P - Inservice Inspection Activities (PWR)

Due to the government shutdown, this inspection was completed remotely via in office document review. The inspectors verified that the reactor coolant system boundary, reactor vessel internals, risk-significant piping system boundaries, and containment boundary are appropriately monitored for degradation and that repairs and replacements were appropriately fabricated, examined and accepted by reviewing documents associated with the following activities in Unit 1 during refueling outage 1R27.

### PWR Inservice Inspection Activities Sample - Nondestructive Examination and Welding Activities (IP Section 03.01) (1 Sample)

The inspectors verified that the following nondestructive examination and welding activities were performed appropriately:

(1) Ultrasonic Examination

- steam generator, EBB01A-SEAM-3-W, stub barrel to shell section weld
- steam generator, EBB01A-SEAM-6-W, stub barrel to shell section weld
- steam generator, EBB01A-SEAM-2-W, tubesheet to stub barrel weld
- steam generator, EBB01A-11-W, feedwater nozzle to shell weld

Dye Penetrant Examination

- residual heat removal, EEJ01A-SKIRT, skirt support weld

Magnetic Particle Examination

- steam generator, EBB01A-11-W, feedwater nozzle to shell weld

Visual 3 Examination

- accumulator safety injection, EP-01-R008, piping support
- accumulator safety injection, EP-01-H006, piping support

Welding Activities

- gas tungsten arc welding process
- safety injection, valve BGHV8153B, socket welds, SW-1A, SW-2, SW-3a

### PWR Inservice Inspection Activities Sample - Vessel Upper Head Penetration Inspection Activities (IP Section 03.02)

There was no ASME Code reactor vessel head inspection required this outage.

### PWR Inservice Inspection Activities Sample - Boric Acid Corrosion Control Inspection Activities (IP Section 03.03) (1 Sample)

The inspectors verified the licensee is managing the boric acid corrosion control program through a review of the following evaluations:

(1)

- 10033393 / 20047316 BBV0311
- 10044591 / 20053937 BBPT0150
- 10044585 / 20053950 EMV0223
- 10044582 BGV0340

- 10044585 EMV0223
- 10044588 BBV0259
- 10044591 BBPT0150
- 10044602 BB8378A
- 10044609 BBV0007
- 10044612 BGV0437
- 10044616 BB8016B
- 10044642 Canopy Seal Weld
- 10044645 BBPCV0455C
- 10044826 EP8956A
- 10044867 BBV0224
- 10045692 ENV0109
- 10045916 EMV0179
- 10046153 BBV0311
- 10046278 BGPIS0140
- 10046430 ENV0027

PWR Inservice Inspection Activities Sample - Steam Generator Tube Inspection Activities (Section 03.04)

There were no ASME Code required steam generator tube inspections required for this outage.

71111.11Q - Licensed Operator Requalification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (2 Samples)

- (1) The inspectors observed and evaluated licensed operator performance in the control room during reactor plant shutdown for refueling outage 27 on October 1, 2025.
- (2) The inspectors observed and evaluated licensed operator performance in the control room during reactor startup from refueling outage 27 on November 14, 2025.

Licensed Operator Requalification Training/Examinations (IP Section 03.02) (1 Sample)

- (1) The inspectors observed and evaluated reactor startup from refueling outage 27 just-in-time training on October 28, 2025.

71111.12 - Maintenance Effectiveness

Maintenance Effectiveness (IP Section 03.01) (1 Sample)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

- (1) emergency diesel generators A and B maintenance effectiveness and operating experience review on October 24, 2025

### 71111.13 - Maintenance Risk Assessments and Emergent Work Control

#### Risk Assessment and Management Sample (IP Section 03.01) (2 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

- (1) emergent work and troubleshooting of the turbine-driven auxiliary feedwater pump tripping on overspeed during surveillance testing on November 19, 2025
- (2) auxiliary building and control room pressurization test with multiple doors breached to verify emergency exhaust and control room emergency ventilation past operability on November 28, 2025

### 71111.15 - Operability Determinations and Functionality Assessments

#### Operability Determination or Functionality Assessment (IP Section 03.01) (1 Sample)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

- (1) emergency diesel generator B intercooler heat exchanger power pill and air start relay failure past operability review on December 16, 2025

### 71111.18 - Plant Modifications

#### Temporary Modifications and/or Permanent Modifications (IP Section 03.01 and/or 03.02) (1 Sample)

The inspectors evaluated the following temporary or permanent modifications:

- (1) main transformer and main generator automatic voltage regulator replacement on October 15, 2025

### 71111.20 - Refueling and Other Outage Activities

#### Refueling/Other Outage Sample (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated refueling outage 27 from October 1 to November 16, 2025.

### 71111.24 - Testing and Maintenance of Equipment Important to Risk

The inspectors evaluated the following testing and maintenance activities to verify system operability and/or functionality:

#### Post-Maintenance Testing (PMT) (IP Section 03.01) (7 Samples)

- (1) train A and B residual heat removal maintenance outage post-maintenance testing on October 15, 2025
- (2) emergency diesel generator B major overhaul post-maintenance testing on October 23, 2025

- (3) train B essential service water maintenance outage post-maintenance testing on October 23, 2025
- (4) train B component cooling water system maintenance outage post-maintenance testing on October 24, 2025
- (5) reactor coolant system loop 2 pressurizer spray valve, BBPCV0455B, replacement post-maintenance testing on November 14, 2025
- (6) main generator automatic voltage regulator and main transformer replacement post-maintenance testing on November 18, 2025
- (7) train A motor-driven auxiliary feedwater discharge header to steam generator C motor-operated isolation valve actuator, ALHV0011, replacement post-maintenance testing on December 15, 2025

#### Surveillance Testing (IP Section 03.01) (4 Samples)

- (1) borated refueling water storage tank system inservice valve leakage surveillance testing on October 15, 2025
- (2) containment integrated leak rate testing on November 1, 2025
- (3) train B load shedder and emergency sequencer card replacement post-maintenance testing and engineering safety feature actuation instrumentation surveillance testing on November 7, 2025
- (4) excess letdown solenoid operated valves inservice testing following replacement on December 4, 2025

#### Containment Isolation Valve (CIV) Testing (IP Section 03.01) (1 Sample)

- (1) local leak rate testing of reactor coolant pump loop B seal water supply valve, containment penetration 22, on October 24, 2025

### **OTHER ACTIVITIES – BASELINE**

#### 71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

#### MS08: Heat Removal Systems (IP Section 02.07) (1 Sample)

- (1) November 1, 2024, through June 30, 2025

#### MS09: Residual Heat Removal Systems (IP Section 02.08) (1 Sample)

- (1) November 1, 2024, through June 30, 2025

#### OR01: Occupational Exposure Control Effectiveness Sample (IP Section 02.15) (1 Sample)

- (1) April 1, 2024, through September 30, 2025

#### PR01: Radiological Effluent Technical Specifications/Offsite Dose Calculation Manual Radiological Effluent Occurrences (RETS/ODCM) Radiological Effluent Occurrences Sample (IP Section 02.16) (1 Sample)

- (1) April 1, 2024, through September 30, 2025

## 71152A - Annual Follow-up Problem Identification and Resolution

### Annual Follow-up of Selected Issues (Section 03.03) (1 Sample)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issues:

- (1) pressurizer spray valve leakage and bypass valve flow setting on November 3, 2025

## 71152S - Semiannual Trend Problem Identification and Resolution

### Semiannual Trend Review (Section 03.02) (1 Sample)

- (1) The inspectors reviewed the licensee's corrective action program to identify potential trends in opened items in the operations deficiency log and corrective actions closed to work orders backlog that might be indicative of a more significant safety issue.

## 71153 - Follow-Up of Events and Notices of Enforcement Discretion

### Event Follow-up (IP Section 03.01) (1 Sample)

- (1) The inspectors evaluated the loss of construction loop electrical circuit due to breaker 13-4 tripping open and actuation of control room ventilation isolation signal, containment purge isolation signal, fuel building isolation signal, and train A control room air condition and pressurizer fan trip due to blown fuse and licensee's response on November 20, 2025.

### Event Report (IP Section 03.02) (2 Samples)

The inspectors evaluated the following licensee event reports (LERs):

- (1) LER 05000482/2025-001-00, "Improperly Breached Control Building Doors Cause a Condition Prohibited by Technical Specifications" (Agencywide Documents Access and Management System (ADAMS) Accession No. ML25351A282). The inspection conclusions associated with this LER are documented in this report under Inspection Results Section 71153. This LER is Closed.
- (2) LER 05000482/2025-003-00, "Loss of Voltage on ESF Bus Results in Automatic Actuation of Emergency Diesel Generator" (ML25351A150). The inspectors determined that the cause of the condition described in the LER was not reasonably within the licensee's ability to foresee and correct, and therefore was not reasonably preventable. No performance deficiency nor violation of NRC requirements was identified. This LER is Closed.

## **INSPECTION RESULTS**

Minor Violation	71153
Minor Violation: The licensee identified a failure to implement administrative controls for an opening in the control room envelope and control building envelope boundaries, ensuring they are closed consistently with the safety analysis during core alterations. This was a performance deficiency.	

**Screening:** The inspectors determined the performance deficiency was minor. The performance deficiency is minor because it did not adversely impact the Barrier Integrity cornerstone for maintaining the radiological barrier functionality of the control room. Specifically, the licensee performed Procedure STS PE-004, "Auxiliary Building and Control Room Pressure Test," Revision 20, Doors 33012 and 31011 open under administrative controls to demonstrate the control room envelope would maintain the required technical specification pressurization of greater than or equal to 0.25 inches water gauge.

**Enforcement:** Technical Specification 3.7.10, "Control Room Emergency Ventilation System (CREVS)," requires that while the plant is in MODES 1, 2, 3, and 4, during core alterations, during movement of irradiated fuel assemblies, two CREVS trains shall be operable and the control room envelope and control building envelope boundaries may be opened intermittently under administrative controls that ensure the building boundary can be closed with the safety analysis. Condition E requires when one or more CREVS trains are inoperable due to an inoperable control room envelope boundary during core alterations, it immediately suspends core alterations.

Contrary to the above, on October 10, 2025, from 9:06 a.m. to 9:19 a.m., during core alterations the licensee failed to implement administrative controls for when Doors 33012 and 31011 were opened to ensure they are closed consistently with the safety analysis during core alterations. The licensee immediately closed Doors 33012 and 31011 upon discovery to restore compliance with Technical Specification 3.7.10, Condition E. This failure to comply with Technical Specification 3.7.10 constitutes a minor violation that is not subject to enforcement action in accordance with the NRC's Enforcement Policy.

## **EXIT MEETINGS AND DEBRIEFS**

The inspectors verified no proprietary information was retained or documented in this report.

- On January 8, 2026, the inspectors presented the integrated inspection results to Robert Bayer, Vice President of Engineering, and other members of the licensee staff.
- On December 4, 2025, the inspectors presented the Exit meeting for inservice inspection results to Josh Turner, Regulatory Affairs Manager, and other members of the licensee staff.

## DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.04	Corrective Action Documents	Condition Report (CR)	67683, 10034501, 10043396, 10045196, 10046236, 10046237, 10046246, 10017814, 10025638, 10021234	
	Corrective Action Documents Resulting from Inspection	Condition Report (CR)	10045290, 10045591, 10045601, 10045603, 10046136, 10047948	
	Drawings	M-12BB01	Piping and Instrumentation Diagram Reactor Coolant System	49
		M-12BB02	Piping and Instrumentation Diagram Reactor Coolant System	39
		M-12EJ01	Piping and Instrumentation Diagram Residual Heat Removal System	57
	Engineering Changes	CP 020865	BBPCV0455B Valve Replacement	0
	Miscellaneous	M-10EJ	System Description Residual Heat Removal	7
		M-724-00448	Instruction Manual for Pressurizer Spray Valve	W10
		WCAP-17659-P	Wolf Creek Generating Station Transition of Core Design and Safety Analyses to Westinghouse Methods and Alternate Source Term Engineering Report	0
		WCRE-34	Fourth 10-Year Interval Inservice Testing Basis Document	15
	Procedures	ALR 00-033C	PZR Press LO HTRS On	15
		ALR 00-035B	PORV Open	11
		BD-OFN MA-038	Rapid Plant Shutdown	21
		GEN 00-004	Power Operation	102
		GEN 00-006	Hot Standby to Cold Shutdown	106
		OFN BB-031	Shutdown LOCA	38
		OFN EJ-015	Loss of RHR Cooling	35
		OFN EJ-040	CL Recirc during Mode 3 with Accumulators Isolated Mode 4, 5, o 6	9
		OFN MA-001	Load Rejection or Turbine Trip	27
		OFN MA-038	Rapid Plant Shutdown	32A
	SYS BB-203		Pressurizer Backup Heater Operations	7

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.05		SYS EF-121	Startup of A RHR Train in Cooldown Mode	35
		SYS EJ-120	Startup of A Residual Heat Removal Train	74
	Work Orders	WO	7723, 128490, 147853, 147862, 20036382, 20039620, 22-483478-000	
71111.08P	Corrective Action Documents	Condition Report (CR)	10044951, 10045435, 10045485	
	Miscellaneous	WCRC-35	Boundary Matrix	14
	Procedures	STN KC-216E	Main Transformers Deluge Spray Testing	008
71111.08P	Corrective Action Documents		10031846, 10031864, 00136243, 10034009, 10035593, 10035594, 10038448, 10042664, 10031862, 10031957, 10031999, 10032067, 10032127, 10035080, 10035591, 10035592, 10036043, 10038960, 10039174, 10041804, 10042596, 10043072, 10043394, 10043579, 10044110, 10044886, 10044876	
	Miscellaneous	INR-WC-1R26-24-04	Radial Support Keys at 120°, 240° and 300°	04/14/2024
		PWROG letter OG-21-160R1	Justification for Continued Operation for Clevis Block Insert and Bolts Inspection	04/27/2024
		SA-2025-0204	ISI self assessment 2025	07/03/2025
	Procedures	54-IS1-604-017	Automated Ultrasonic Examination of Open Tube RPV Closure Head Penetrations	17
		54-ISI-460-009	Multi-Frequency Eddy Current Orthogonal Coil Array Probe Examination of Nozzle Welds and Regions	9
		54-ISI-494-005	Multi-Frequency Eddy Current Array ID Probe Examination of Vent Line and RVLIS Nozzle Bores	5
		54-ISI-603-013	Automated Ultrasonic Examination of RPV Closure Head Penetrations Containing Thermal Sleeves	13
		54-UT-901-004	Automated Ultrasonic Examination of Wear in RPV Closure Head Thermal Sleeves	4
		AI 16F-001	Evaluation Of Boric Acid Leakage	13
		AI 16F-002	Boric Acid Leakage Management	14
		AP 16F-001	Boric Acid Corrosion Control Program	11
		AP 16F-002	Fluid Leakage Management	1A
		AP 28D-001	Self-Assessment Process	25

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		Docket No. 50-482	Notification of Extension to the Fourth Ten-Year Interval of the Inservice Testing Program	09/03/2025
		GWS-ASME	ASME General Welding Standard	7
		LMT-08-MT-01	Magnetic Particle Examination	1
		LMT-08-PDI-UT-1	Ultrasonic Examination of Ferritic Piping Welds	1
		LMT-08-PDI-UT-11	Ultrasonic Examination of Nozzle Inner Radius Areas at Wolf Creek Nuclear Operating Corporation	1
		LMT-08-PDI-UT-2	Ultrasonic Examination of Austenitic Piping Welds	1
		LMT-08-PDI-UT-3	Ultrasonic Through Wall Sizing in Piping Welds	1
		LMT-08-UT-004	Ultrasonic Examination of Vessel Welds and Adjacent Base Metal > 2.0" in Thickness	2
		LMT-08-UT-119	Conventional Ultrasonic Instrument Linearity	2
		WCRC-30	Inservice Inspection Program Plan Wolf Creek Generating Station Interval 4	8
		WCRC-36	Containment Inservice Inspection (CISI) Program Plan Wolf Creek Generating Station Interval 3	6
		WCRC-40	Inservice Inspection Program Plan Wolf Creek Generating Station Interval 5	0
	Work Orders	WO	60740, 77098, 92648, 109004, 113426, 113460	
71111.11Q	Procedures	EMG E-0	Reactor Trip or Safety Injection	48
		EMG ES-02	Reactor Trip Response	42
		GEN 00-003	Hot Standby to Minimum Load	111, 112
		GEN 00-004	Power Operations	102
		GEN 00-005	Minimum Load to Hot Standby	97
		GEN 00-006	Hot Standby to Cold Shutdown	106
71111.12	Corrective Action Documents	Condition Report (CR)	10045385, 10042393, 10041165, 10041372, 10041703, 10041825, 10041890, 10042663, 10042690, 10043234, 10043335, 10043351, 10043395, 10043492, 10043719, 10043714, 10044201, 10044345, 10044497, 10044552, 10012536, 10012550, 10029593	
	Work Orders	WO	114936, 75563, 58731, 70762, 104553, 17194, 17201, 17212, 81637, 81641, 106888, 114106, 140169, 58995, 59001, 59009, 67436, 67672, 67708, 59001, 59009, 67436, 67672, 96036, 104553, 17205, 70762, 39977	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.13	Corrective Action Documents	Condition Report (CR)	10046923, 10046945, 10046948, 10046985	
	Miscellaneous		Breach authorization permits: PT118168, 118170, 118174, 118176, 118258	
		M-021-00086	Instruction Manual for Terry Turbine Corp.	W53
		M-021A-00011	Instruction Manual for Terry Turbine Digital Controls Replacement/Retrofit	W07
		WCRE-35	Boundary Matrix	14
	Procedures	AI 16C-006	Troubleshooting	13A
		AP 10-103	Fire Protection Impairment Control	42
		AP 10-104	Breach Authorization	42
		STS AL-211	TD AFP Comprehensive Pump Testing, Flow Path Verification and CV Testing	46
		STS PE-004	Aux Building and Control Room Pressure Test	20
	Work Orders	WO	154486	
71111.15	Corrective Action Documents	Condition Report (CR)	10045385	
71111.18	Corrective Action Documents	Condition Report (CR)	10043534, 10044228, 10044289, 10044405, 10044534, 10044544, 10045241	
	Drawings	KD-7496	One Line Diagram	83
	Engineering Changes	DCP 020384	Main Generator Step-Up Power Transformer (GSU)-XMA01A, B, C Replacement	0
		DCP 13071	AVR Replacement Project	0
	Procedures	AI 26A-003	Regulatory Evaluations (Other Than 10 CFR 50.59)	18
		AP 26A-003	10 CFR 50.59 Reviews	15
		OFN KC-016	Fire Response	54
71111.20	Corrective Action Documents	Condition Report (CR)	10003133, 10002793, 135907, 10044649, 10045142, 10045143, 10044894, 10044935, 10045173, 10045176, 10045160, 10045627, 10044797, 10045738, 10045072, 10045253, 10045701, 10045393, 10045742, 10045600, 10045484, 10045261, 10045016, 10045124	
	Corrective Action Documents Resulting from	Condition Report (CR)	10044645, 10044646, 10044647, 10044648, 10044864, 10044849, 10044954, 10044955, 10044959, 10046432, 10046463, 10046498, 10047953, 10047955	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Inspection			
	Drawings	K-12KJ04	Piping and Instrumentation Diagram Standby Diesel Generator B Cooling Water System	19
		M-12EF01	Piping Instrumentation Diagram Essential Service Water	30
		M-12FC03	Piping and Instrumentation Diagram Auxiliary Turbines SGFP Turbine A	18
		M-12KA02	Piping and Instrumentation Diagram Compressed Air System (Service AR)	42
	Miscellaneous	C27-R-KJ-B-001A	EDG B Clearance Order	0
		WCRE-34	Fourth 10-Year Interval Inservice Testing Basis Document	15
	Procedures	AP 12-004	Containment Entry and Material Control	7
		AP 21G-001	Control of Locked Component Status	78E
		AP 22C-003	On-Line Nuclear Safety and Generation Risk Assessment	30
		FHP 02-001	Refueling Operations	47A
		FHP 02-007B	Reactor Vessel Closure Head Installation	20A
		FHP 05-001	Equipment Hatch Operations	3
		GEN 00-002	Cold Shutdown to Hot Standby	110B
		GEN 00-003	Hot Standby to Minimum Load	111, 112
		GEN 00-005	Minimum Load to Hot Standby	97
		GEN 00-008	RCS Level Less Than Reactor Vessel Flange Operations	36
		OFN EJ-015	Loss of RHR Cooling	35
		OFN NB-030	Loss of All AC Power – Shutdown Conditions	35A
		STN EJ-002	Containment Inspection	27
		STS BB-011	Reactor Coolant System and Pressurizer Heatup/Cooldown Surveillance	35
		STS EJ-002	Containment Sump Inspection	17A
		SYS EJ-111	Initial RCS Fill	6
		SYS SL-139	Energizing #10 or #11 Transformer	3
	Work Orders	WO	26058, 26813, 70712, 147953, 148576, 149045	
71111.24	Calculations	BN-M-013	RWST Volume Requirements for Injection, ECCS, and Containment Spray Pumps Transfer and Time Available for Operator Actions	1

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
	Corrective Action Documents	Condition Report (CR)	33323, 38960, 39037, 39056, 39193, 39285, 39331, 46927, 10041112, 10045750, 10044967, 10044972, 10045149, 10045150, 10045074, 10045087, 10045058, 10045252, 10045287, 10045298, 10045341, 10045926, 10046056, 10046452, 10046454, 10044764, 10045125, 10045053, 10045196, 10045002, 10045192, 10045862, 10045925, 10045979, 10046231, 10046767, 10045688, 10045706, 10045707, 10045727, 10044240, 10044497, 10044775, 10044857, 10045496, 10045426, 10046474, 10044982, 10045068, 10045492, 10045613, 10045686, 10045764, 10046572, 10046575, 10045621, 10046631, 10046633, 10046686, 10046733, 10043694, 10046237, 10046246, 10046000, 10044860, 10044862, 10044863, 10043310, 10045059, 10045060, 10045481, 10045665, 10045795, 10046003, 10047248, 10046779	
	Corrective Action Documents Resulting from Inspection	Condition Report (CR)	10046236	
	Drawings	E-025-00007, Sheet 236	RHR A to SIS Hot Leg Recirc Loops 2 and 3 ISO	W14
		E-025-00007, Sheet 237	RHR B to SIS Hot Leg Recirc Loops 2 and 3 ISO	W16
		E-13BG48	Schematic Diagram Excess Letdown Line Isolation Valves	2
		M-12BG01	Piping and Instrumentation Diagram Chemical and Volume Control System	26
		M-12BG01	Piping and Instrumentation Diagram Chemical and Volume Control System	53
		M-12BN01	Piping and Instrumentation Diagram Borated Refueling Water Storage System	20
		M-12EG01	Piping and Instrumentation Diagram Component Cooling Water System	26
		M-12EG02	Piping and Instrumentation Diagram Component Cooling Water System	29

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		M-12EG03	Piping and Instrumentation Diagram Component Cooling Water System	19
		M-12EJ01	Piping and Instrumentation Diagram Residual Heat Removal System	57
		M-12EM01	Piping and Instrumentation Diagram High Pressure Coolant Injection System	49
		M-12GP01	Piping and Instrumentation Diagram Containment Integrated Leak Rate Test	6
		M-1HX001, Sheet 83	Heat Exchanger Tube Sheet Map Component Cooling Water Heat Exchanger B (South End)	95
		STS BN-206	Borated Refueling Water Storage System Inservice Testing Valve Test	29
Engineering Changes	DCP 013071		Main Generator Automatic Voltage Regulator Replacement	1
Miscellaneous	C27-R-KJ-B-001A		EDG B Clearance Order	0
	WCLR-22		10 CFR 50, Appendix J B2.1.30 NUREG-1801 Program XI.S4 Aging Management Program	2
	WCRE-34		Fourth 10-Year Interval Inservice Testing Basis Document	15
Procedures	AP 16E-002		Post Maintenance Testing Development Guide	22A
	AP 21G-001		Control of Locked Component Status	78E
	AP 23D-001		Motor Operated Valve Program	4A
	AP 29B-002		ASME Code Testing of Pumps and Valves	12A
	AP 29E-001		Program Plan for Containment Leakage Measurement	17A
	CKL EG-120		Component Cooling Water System Valve, Switch and Breaker Lineup	51
	EMG ES-12		Transfer to Cold Leg Recirculation	25A
	EMG ES-13		Transfer to Hot Leg Recirculation	14A
	MEC-009		Disassembly/Assembly of Westinghouse Style A Check Valves	4
	STN EF-022B		ESW Train B Post-Loca Flow Balance	12A
	STN EJ-202		System Valve Test	2
	STN IC-902		Cross Trip Check: Main Transformers, Main Generator/Switchyard	13A

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		STS CV-211	RHR Full Flow Test and Accumulator Safety Injection Check Valve Test	35
		STS EJ-100A	RHR System Inservice Pump A Test	57
		STS KJ-001B	Integrated D/G and Safeguards Actuation Test – Train B	71
		STS MT-027	Snubber Functional Test	19
		STS PE-007	Periodic Verification of Motor Operated Valves	5A
		STS PE-018	Containment Integrated Leakage Rate Test	12
		STS PE-122	LLRT Valve Lineup for Penetration 22	7
		STS VT-001	Verification of ASME OM Code Mandatory Appendix III, MOV Exercise Requirements	7
		SYS GP-510B	CILRT – BB System (RCPS & SEAL INJ)	1
		SYS GP-516	CILRT – EG System	3
		SYS GP-517	CILRT – EJ System	2
		SYS KJ-124	Post Maintenance Run of Emergency Diesel Generator B	78
	Work Orders	WO	23830, 32141, 71730, 71732, 71737, 80932, 104728, 112409, 112410, 118065, 120018, 21-473687-003, 23-485897-001, 92819, 23-485897-001, 23-485897-000, 20-462318-000, 146043, 23912, 23916, 23918, 104697, 104696, 104694, 71689, 146043, 120311, 152336, 149775, 104157, 151489, 75563, 114936, 75563, 58731, 70762, 104553, 17194, 17201, 17212, 81637, 81641, 106888, 114106, 140169, 58995, 59001, 59009, 67436, 67672, 67708, 59001, 59009, 67436, 67672, 96036, 104553, 17205, 70762, 39977, 10045739, 10045740, 114936, 75563, 58731, 70762, 104553, 17194, 17201, 17212, 81637, 81641, 106888, 114106, 140169, 58995, 59001, 59009, 67436, 67672, 67708, 59001, 59009, 67436, 67672, 96036, 104553, 17205, 70762, 39977, 70712, 107997, 144373, 81350, 96766, 137093, 7164, 81328, 81332, 82966, 135735, 104803, 21-473172, 9719, 81328, 81332, 71658, 71666, 71662, 19-450030-005, 153024, 153066, 152956, 152957, 152961, 152908, 152845, 152846, 152931, 10326827-000, 08-306055-002, 80979, 23-485785-000, 21-473687-000, 23-485786-000, 23-485787-000, 23-485785-005, 72894	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71152A	Corrective Action Documents	Condition Report (CR)	88519, 103802, 126718, 23157, 40756, 146699, 10002001, 10003500, 10012402, 10023022, 10025491, 10027136, 10033678, 10034501, 10034549, 10041825, 10041992, 10043396, 10044344, 10045192	
	Drawings	M-12BB01	Piping and Instrumentation Diagram Reactor Coolant System	49
		M-12BB02	Piping and Instrumentation Diagram Reactor Coolant System	39
	Miscellaneous	M-724-00448	Instruction Manual for Pressurizer Spray Valve	W10
	Procedures	ALR 00-033C	Press LO HTRS On	15
		BD-OFN MA-038	Rapid Plant Shutdown	21
		GEN 00-004	Power Operation	102
		OFN MA-001	Load Rejection or Turbine Trip	27
		OFN MA-038	Rapid Plant Shutdown	32A
		SU7-BB05	Pressurizer Continuous Spray Flow Setting	1
		SYS BB-220	Pressurizer Continuous Spray Flow Setting	5
	Work Orders	WO	76485, 11-343606-001, 12-350340-000, 13-375254-006	
71152S	Calculations	XX-M-080	Wolf Creek Generating Station Pump Room Heat-up Analysis	1
	Corrective Action Documents	Condition Report (CR)	10023022, 10041992, 10027136, 10025491, 10012402, 126718, 88519, 103802, 146699, 10002001, 10003500, 10041825, 10037556, 10036376, 10036649, 10040645, 10036368	
	Drawings	M-13EF02	Piping Isometric Essential Service Water Sys Aux BLDG "A" Train Supply	22
		M-1HX001	Heat Exchanger Tube Sheet Maps	95
	Engineering Changes	CP 014266	ESW Aboveground Pipe Replacement	15
	Procedures	AP 26C-004	Operability Determinations and Functionality Assessments	48
		STN FP-452	Fire Barrier Penetration Seals Inspection	7
	Work Orders	WO	68145, 85293, 85303, 87055, 87228, 91842, 100855, 120072, 14-383646-000, 14-387207-002, 15-404306-000, 16-412985-000, 18-436050-000, 19-456291-001, 21-474963-003	

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71153	Calculations	XX-E-006	AC System Analysis	10
	Corrective Action Documents	Condition Report (CR)	10045627, 138017, 138036, 138042, 10045630, 10045632, 10045652, 10045661, 10045689, 10045691, 10045713, 10037832, 10031456, 10037823, 10046428, 10045821, 10045853, 10045730, 10045586, 10045588, 10045592, 10045829, 10043577, 10043578, 10046405, 10046409	
	Corrective Action Documents Resulting from Inspection	Condition Report (CR)	10045950, 10043580, 10047946	
	Drawings	KD-7496	One Line Diagram	85
		M-622.1A-00007	SGK04A and SGK04B Air Conditioning Electric Schematic	7
	Engineering Changes	DECP 020350	SGK05A/B Fuse Replacement	0
	Miscellaneous	WCNOC-76	Medium Low Voltage AC and Low Voltage DC Overcurrent Protection Coordination for Wolf Creek Generating Station	8
		WCRE-08	WCGS Fuse List	35
		WCRE-35	Boundary Matrix	14
	Procedures	ALR 00-018D	NB01 UV Relay ¼	7A
		AP 10-104		
		AP 10-104	Breach Authorization	42
		STS IC-208A	4kV Loss of Voltage and Degraded Voltage TADOT NB01 Bus – Sep GRP 1	9A
		STS PE-004	Aux Building and Control Room Pressure Test	20
		SYS GK-201	Mitigating Actions for Inoperable SGK05 Train	5A
		SYS SL-138	Energizing 13.8 kV Buses SL-7 and SL-8	7
		TMP 25-003	Energizing Main Transformers from Backfeed for Post Installation Soak	0A
	Work Orders	WO	14042, 95363, 102802, 149045, 18-437840-000, 18-437840-001, 423-488488-000	