



Sequoyah Nuclear Plant, Post Office Box 2000, Soddy Daisy, Tennessee 37384

August 21, 2025

10 CFR 50.4
10 CFR 50.55a

ATTN: Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555-0001

Sequoyah Nuclear Plant, Unit 2
Renewed Facility Operating License No. DPR-79
NRC Docket No. 50-328

Subject: **Unit 2 Cycle 26 Refueling Outage - 90-Day Inservice Inspection Summary Report**

In accordance with the American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code, Section XI, 2007 Edition and the 2008 Addenda, Article IWA-6240 and Code Case N-532-5, the Tennessee Valley Authority is providing the Sequoyah Nuclear Plant (SQN), Unit 2, Inservice Inspection (ISI) Summary Report within 90 days from completion of the Unit 2 Cycle 26 Refueling Outage (U2C26). The U2C26 refueling outage ended on May 26, 2025. Accordingly, this report is required to be submitted by August 24, 2025.

The Enclosure to this letter contains the Owner's Activity Report, a list of items with flaws or relevant conditions that required evaluation for continued service (Table 1), an abstract of repair and replacement activities required for continued service due to a flaw or relevant condition (Table 2), and the evaluation of acceptability of inaccessible areas of the steel containment vessel in accordance with 10 CFR 50.55a(b)(2)(ix)(A).

There are no new regulatory commitments associated with this submittal. Should you have any questions, please contact Mr. Rick Medina, Site Regulatory Compliance Manager at (423) 843-8129 or rmedina4@tva.gov.

Respectfully,

A handwritten signature in black ink, appearing to read 'Kevin M. Michael', with a stylized flourish at the end.

Kevin M. Michael
Site Vice President
Sequoyah Nuclear Plant

U.S. Nuclear Regulatory Commission
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Enclosure: ASME Section XI Inservice Inspection Summary Report,
Unit 2 Cycle 26 Refueling Outage

cc (Enclosure):
NRC Regional Administrator – Region II
NRC Senior Resident Inspector – Sequoyah Nuclear Plant
NRC Project Manager – Sequoyah Nuclear Plant

ENCLOSURE

**TENNESSEE VALLEY AUTHORITY
SEQUOYAH NUCLEAR PLANT, UNIT 2**

**ASME SECTION XI
INSERVICE INSPECTION SUMMARY REPORT
UNIT 2 CYCLE 26 REFUELING OUTAGE**

FORM OAR-1 OWNER'S ACTIVITY REPORT

Report Number U2R26

Plant SEQUOYAH NUCLEAR PLANT, P.O. Box 2000, Soddy-Daisy, TN 37384-2000

Unit No 2 Commercial Service Date June 1, 1982 Refueling Outage No 26
(if applicable)

Current Inspection Interval 4th (ISI) and 3rd (CISI)
(1st, 2nd, 3rd, 4th, other)

Current Inspection Period 3rd (ISI) and 2nd (CISI)
(1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the Inspection Plans 2007 Edition, 2008 Addenda - Inservice Inspection (ISI)
 2013 Edition - Containment Inservice Inspection (CISI)


Date / Revision of Inspection Plans November 20, 2023: 0-TPP-DXI-114.4 Rev 2 (ISI)
 April 17, 2024: 0-TPP-DXI-114.4 Rev. 3 (ISI)
 July 22, 2024: 0-TPP-DXI-000-114.4 Rev. 4 (ISI)
 February 17, 2022: 0-PI-DXI-000-116.3 Rev. 1 (CISI)

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans 2007 Edition, 2008 Addenda

Code Cases used: N-508-4, N-532-5, N-586-1, N-613-2, N-648-1, N-716-1, N-722-1, N-729-6, N-731, N-747, N-751, N-770-5, N-798, N-854, N-864, N-885

CERTIFICATE OF CONFORMANCE

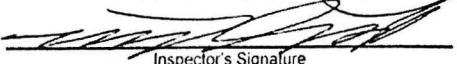
I certify that (a) the statements made in this report are correct, (b) the examinations and tests, meet the Inspection Plan as required by the ASME Code, Section XI, and (c) the repair/replacement activities and evaluations supporting the completion of U2R26 conform to the requirements of Section XI (refueling outage number)

Signed  Date 8-12-25
Owner or Owner's Designee, Title

CERTIFICATE OF INSERVICE INSPECTION

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and employed by The Hartford Steam Boiler Inspection and Insurance Company of Hartford, Connecticut have inspected the items described in this Owner's Activity Report, and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of Section XI.

By signing this certificate neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/ replacement activities and evaluation described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection

 Commissions N14739 INR
Inspector's Signature (National Board Number and Endorsement)

Date 12 August 2025

TABLE 1**Items with Flaws or Relevant Conditions That Required Evaluation for Continued Service**

Examination Category and Item Number	Item Description	Evaluation Description
F-A, F1.20C	Variable Spring Support 2-AFDH-316 setting found to be out of tolerance and found to contain a bent rod.	CR 1953133 Evaluation found support would continue to perform its design function. Based on the Civil Design Evaluation, the variable spring support, BE-401 Type B Size 4 catalog, was well within the working load range, and the minor bend in the rod support would not prevent the support from performing it's intended function. All the support components had significant margin, and the increased load identified during inspection was insignificant per calculation 2-AFDH-0316 and Catalog 193 Basic-PSA, INC. Accepted as is, and WO 124757011 was electively generated to restore the support to the original design compliant condition.

TABLE 2
Abstract of Repair/replacement Activities Required for Continued Service

Code Class	Item Description	Description of Work	Date Completed	Repair/Replacement Plan Number
3	SQN-2-CLR-030-0195	Replace the 2B-B 714 Pen Room Cooler due to through wall tube leak	5/24/2023	122564426
3	SQN-0-STN-067-1065	Replace 1" strainer blowdown flange due to through wall leak	6/28/2023	123802272
3	SQN-2-AHU-313-0367	Replace piping/fittings with new on 2B Shutdown Board Room cooler due to through wall leak	5/19/2025	125302018

REPORTING REQUIRED BY 10 CFR 50.55a(b)(2)(ix)(A)(2)
ASME Section XI, Subsection IWE
Steel Containment Vessel Inspection Program

10 CFR 50.55a(b)(2)(ix)(A)(2) requires licensee to provide the following in the ISI Summary Report as required by IWA-6000:

- (i) A description of the type and estimated extent of degradation, and the conditions that led to the degradation;
- (ii) An evaluation of each area, and the result of the evaluation; and
- (iii) A description of necessary corrective actions.

There were no conditions identified during the Unit 2 Cycle 26 refueling outage on the items examined that prompted an evaluation of the inaccessible area of the SCV, as required.