UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION III
2056 WESTINGS AVENUE, SUITE 400
NAPERVILLE, IL 60563-2657

August 7, 2025

Terry Brown
Site Vice President
Vistra Operations Company, LLC
Perry Nuclear Power Plant
10 Center Road
Perry, OH 44081

SUBJECT: PERRY NUCLEAR POWER PLANT – INTEGRATED INSPECTION REPORT 05000440/2025002

Dear Terry Brown:

On June 30, 2025, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at Perry Nuclear Power Plant. On July 16, 2025, the NRC inspectors discussed the results of this inspection with D. Benyak, Plant Manager, and other members of your staff. The results of this inspection are documented in the enclosed report.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at http://www.nrc.gov/reading-rm/adams.html and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

Elba M. Sanchez Santiago Signed by Sanchez Santiago, Elba on 08/07/25

Elba M. Sanchez Santiago, Chief
Reactor Projects Branch 2
Division of Operating Reactor Safety

Docket No. 05000440
License No. NPF–58

Enclosure:
As stated

cc: Distribution via LISTSERV®

Letter to T. Brown from E. Sanchez Santiago dated August 7, 2025.

SUBJECT: PERRY NUCLEAR POWER PLANT – INTEGRATED INSPECTION REPORT 05000440/2025002

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U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Number:	05000440
License Number:	NPF–58
Report Number:	05000440/2025002
Enterprise Identifier:	I-2025-002-0050
Licensee:	Vistra Operations Company, LLC
Facility:	Perry Nuclear Power Plant
Location:	Perry, OH
Inspection Dates:	April 01, 2025 to June 30, 2025
Inspectors:	J. Beavers, Senior Resident Inspector N. Day, Senior Resident Inspector T. Ospino, Senior Resident Inspector
Approved By:	Elba M. Sanchez Santiago, Chief Reactor Projects Branch 2 Division of Operating Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting an integrated inspection at Perry Nuclear Power Plant, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to https://www.nrc.gov/reactors/operating/oversight.html for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

Type	Issue Number	Title	Report Section	Status
LER	05000440/2025-001-00	LER 2025-001-00 for Perry Nuclear Power Plant, Automatic Actuation of Division 3 Diesel Generator Resulting from Startup Transformer Testing	71153	Closed

PLANT STATUS

Unit 1 began the inspection period shut down in a refueling outage. On April 14, 2025, the outage was completed and power restored to rated thermal power. On June 28, 2025, the unit was reduced to 65 percent to adjust rod line. On June 30, 2025, the unit was restored to rated thermal power. The plant operated at or near rated thermal power for the remainder of the inspection period with occasional power derates related to heat and humidity.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors performed activities described in IMC 2515, Appendix D, "Plant Status," observed risk significant activities, and completed onsite portions of IPs. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71111.01 - Adverse Weather Protection

Seasonal Extreme Weather Sample (IP Section 03.01) (1 Sample)

(1) The inspectors evaluated readiness for seasonal extreme weather conditions prior to the onset of seasonal hot temperatures for the main steam tunnel ventilation, turbine building chilled water system, and auxiliary building ventilation systems on June 20, 2025.

71111.04 - Equipment Alignment

Partial Walkdown Sample (IP Section 03.01) (3 Samples)

The inspectors evaluated system configurations during partial walkdowns of the following systems/trains:

(1) Control rod drive hydraulic train 'B' before startup operations on April 19, 2025
(2) Reactor core isolation cooling system after maintenance on April 19, 2025
(3) Fuel pool cooling and cleanup system after fuel offload on June 26, 2025

Complete Walkdown Sample (IP Section 03.02) (1 Sample)

(1) High pressure core spray during the week of April 7, 2025

71111.05 - Fire Protection

Fire Area Walkdown and Inspection Sample (IP Section 03.01) (2 Samples)

The inspectors evaluated the implementation of the fire protection program by conducting a walkdown and performing a review to verify program compliance, equipment functionality, material condition, and operational readiness of the following fire areas:

(1) Unit 1 turbine building on April 2, 2025
(2) Auxiliary building steam tunnel on April 9, 2025

Fire Brigade Drill Performance Sample (IP Section 03.02) (1 Sample)

(1) The inspectors evaluated the onsite fire brigade training and performance during an announced fire drill on May 14, 2025

71111.11Q - Licensed Operator Requalification Program and Licensed Operator Performance

Licensed Operator Performance in the Actual Plant/Main Control Room (IP Section 03.01) (1 Sample)

(1) The inspectors observed and evaluated licensed operator performance in the control room during reactor plant startup on April 20 and 21, 2025

Licensed Operator Requalification Training/Examinations (IP Section 03.02) (1 Sample)

(1) The inspectors observed and evaluated operator requalification simulator performance on May 5, 2025

71111.12 - Maintenance Effectiveness

Maintenance Effectiveness (IP Section 03.01) (3 Samples)

The inspectors evaluated the effectiveness of maintenance to ensure the following structures, systems, and components (SSCs) remain capable of performing their intended function:

(1) Outboard main steam isolation valve 'D' failure on April 6, 2025
(2) Reactor recirculation pump 'A' flow control valve failure on April 6, 2025
(3) Reactor recirculation pump 'B' flow control valve failure on April 7, 2025

71111.13 - Maintenance Risk Assessments and Emergent Work Control

Risk Assessment and Management Sample (IP Section 03.01) (5 Samples)

The inspectors evaluated the accuracy and completeness of risk assessments for the following planned and emergent work activities to ensure configuration changes and appropriate work controls were addressed:

(1) Risk associated with the turbine overspeed test malfunction on May 3, 2025

(2) Risk associated with the main transformer 'C' cooling groups 1 and 4 issues on May 8, 2025
(3) Division 1 work week planned activities during the week ending May 16, 2025
(4) Division 2 work week planned activities to include division 1 heating and ventilation outage during the week ending May 23, 2025
(5) Emergency service water system 'A' medium risk activities and associated 72-hour technical specifical limiting conditions of operation during the week ending June 20, 2025

71111.15 - Operability Determinations and Functionality Assessments

Operability Determination or Functionality Assessment (IP Section 03.01) (5 Samples)

The inspectors evaluated the licensee's justifications and actions associated with the following operability determinations and functionality assessments:

(1) Operability determination verification of mode restraint list and readiness for entry into Mode 4 on April 10, 2025
(2) Operability determination of intermediate range instruments 'C' and 'E' due to failed linearity checks during reactor plant startup on April 20, 2025
(3) Operability determination of offgas pretreatment radiation monitor on April 22, 2025
(4) Operability determination of division 2 emergency diesel generator during the week ending on May 23, 2025
(5) Operability determination of the offgas pretreat system during the 1R20 outage on June 9, 2025

71111.18 - Plant Modifications

Temporary Modifications and/or Permanent Modifications (IP Section 03.01 and/or 03.02) (1 Sample)

The inspectors evaluated the following temporary or permanent modifications:

(1) Permanent modification of the reactor protection system power selector switch

71111.20 - Refueling and Other Outage Activities

Refueling/Other Outage Sample (IP Section 03.01) (1 Sample)

(1) The inspectors evaluated refueling outage activities from April 1, 2025, to April 21, 2025

71111.24 - Testing and Maintenance of Equipment Important to Risk

The inspectors evaluated the following testing and maintenance activities to verify system operability and/or functionality:

Post-Maintenance Testing (PMT) (IP Section 03.01) (13 Samples)

(1) Replacement of hydramotor 1D17F0071B as a result of a failed stroke time close on March 12. 2025

(2) Replacement of the high voltage fuse and the challenges returning transformer LH2A to service on March 27, 2025
(3) Low pressure core spray injection valve work on March 29, 2025
(4) Low pressure core spray and 'A' residual heat removal (RHR) waterleg pump test on March 29, 2025
(5) Drywell airlock test during the 1R20 outage
(6) RHR 'A' to feedwater injection valve leak rate test on April 9,2025
(7) Reactor core isolation cooling steam supply inboard and outboard valves following maintenance on April 10, 2025
(8) Main steam isolation outboard 'A' air drop test after test failure on April 13, 2025
(9) Main steam isolation inboard valve 'A' VT-2 during reactor pressure vessel test on April 16, 2025
(10) Reactor recirculating pump suction valve VT2 during reactor pressure vessel test on April 16, 2025
(11) Main steam isolation inboard drain valve VT-2 during reactor pressure vessel test on April 16, 2025
(12) Reactor recirculating pump 'B' flow control valve VT-2 during reactor pressure vessel test on April 16, 2025
(13) Outboard main steam isolation valve 'A' local leak rate testing (LLRT) and other tests after issues with it on June 11, 2025

Surveillance Testing (IP Section 03.01) (7 Samples)

(1) Division 3 diesel generator start and load on March 28, 2025
(2) Division 1 retest of the loss of offsite power/ loss of coolant accident (LOOP/LOCA) testing on April 1, 2025
(3) Fire suppression valve cycle on April 3, 2025
(4) Leak rate testing for 'A' RHR to feedwater injection valve on April 8, 2025
(5) Unit 2 transformer insulation resistance checks on April 10, 2025
(6) Intermediate range monitoring voltage calibration on April 20, 2025
(7) Unit 2 main transformer double testing on April 23, 2025

Inservice Testing (IST) (IP Section 03.01) (3 Samples)

(1) Shutdown cooling suction isolation valve operability test on March 28, 2025
(2) Low pressure core spray isolation valve leak rate test on March 30, 2025
(3) Reactor coolant system leakage pressure test on April 16, 2025

71114.06 - Drill Evaluation

Required Emergency Preparedness Drill (1 Sample)

(1) Full scale emergency response organization drill on June 11, 2025

OTHER ACTIVITIES – BASELINE

71151 - Performance Indicator Verification

The inspectors verified licensee performance indicators submittals listed below:

MS05: Safety System Functional Failures (SSFFs) Sample (IP Section 02.04) (1 Sample)

(1) Unit 1 (April 1, 2024 through March 31, 2025)

MS06: Emergency AC Power Systems (IP Section 02.05) (1 Sample)

(1) Unit 1 (April 1, 2024 through March 31, 2025)

MS07: High Pressure Injection Systems (IP Section 02.06) (1 Sample)

(1) Unit 1 (April 1, 2024 through March 31, 2025)

71152A - Annual Follow-up Problem Identification and Resolution

Annual Follow-up of Selected Issues (Section 03.03) (1 Sample)

The inspectors reviewed the licensee's implementation of its corrective action program related to the following issues:

(1) Onsite storage container radioactive leak

71152S - Semiannual Trend Problem Identification and Resolution

Semiannual Trend Review (Section 03.02) (1 Sample)

(1) The inspectors reviewed the licensee's corrective action program to identify potential trends in equipment reliability that might be indicative of a more significant safety issue.

71153 - Follow-Up of Events and Notices of Enforcement Discretion

Event Follow-up (IP Section 03.01) (1 Sample)

(1) On June 11, 2025, an individual found smoke in warehouse 6, which is located inside the licensee's owner-controlled area but outside the protected area. The control room was immediately notified and Off Normal Instruction ONI-P54, "Fire," was entered and 911 was called to initiate offsite firefighting response. The control room directed the starting of the motor fire pump and immediate notification of site personnel including the resident inspectors. The site fire brigade remained on standby as per agreements with local firefighting agencies as this building was outside of the protected area fence and required no special access to mitigate the fire. Licensee personnel remained in contact with the local firefighting agency efforts and were available to deenergize the building as required by the incident commander, which was not requested. Inside the warehouse, a limited area fire was identified and mitigation commenced. Damaged and burned materials were removed to the parking lot and thoroughly wetted. The fire was extinguished within an hour of offsite response.

No personnel were injured during this event, and no ambulance response was required. The same day the off normal instruction was exited and the fire system restored to standby. The affected warehouse contained materials to support plant operation and maintenance activities. No hazardous materials were identified as impacted, and the building contained no radioactive material.

The inspectors determined the event did not meet any emergency action level entry criteria due to the fire's location being outside of the protected area, and the fact that the fire did not impact the safety and security of the power plant or its operations. Additionally, no nuclear regulatory reporting actions were required. The inspectors did not identify any issues associated with the licensee's response.

Event Report (IP Section 03.02) (1 Sample)

The inspectors evaluated the following licensee event reports (LERs):

(1) LER 2025-001, "Automatic Actuation of Division 3 Diesel Generator Resulting from Startup Transformer Testing." (ADAMS Accession No. ML25141A212)

The inspection conclusions associated with this LER are documented in this report under Inspection Results Section 71153. This LER is Closed.

INSPECTION RESULTS

Observation: Onsite Storage Container Radioactive Leak	71152A
The inspectors performed an indepth review of the October 14, 2024, onsite storage container radioactive leak event cause and corrective action progress as of June 30, 2025. This sample was selected as a timely review of the event with potential public impact and interest previously dispositioned in the Perry Integrated Inspection Report 05000440/2024004 (ML25042A066). The licensee's analysis determined site personnel made assumptions that the shield container was suitable for outdoor storage without verification in contrast to the requirements of site procedures. This conclusion was supported by: the site not procuring the necessary equipment or plan for how to handle the liner after it was depleted to ensure safe, timely handling, storage, and disposal; inadequate information regarding the suitability of the shield container for outdoor storage in the work order, procedures, or other instructions; and failure to stop in the face of uncertainty when trying the set the shield plug into place. These issues were dispositioned in the aforementioned inspection report. The licensee's corrective actions included the completed immediate actions of securing the leak source and radioactive mitigation of the affected area. The plan to track and clean up the onsite storage container yard has been approved and largely completed with the majority of the contaminated soil shipped offsite and the remainder of the soil and sealed source liner prepared for imminent transport. The documentation and process changes have been largely completed, including the update of 10CFR50.75(g) for use in eventual plant decommissioning activities. The remaining items include the identification and fabrication of a new location for onsite storage of radiological waste awaiting transport. The inspectors evaluated the licensee's onsite storage container radioactive leak cause and corrective action effort and identified no more than minor findings or violations.	

Observation: Trend in Equipment Reliability	71152S

The inspectors performed a semiannual review of a potential adverse trend in the licensee's equipment reliability issues that might be indicative of a more significant safety issue. The following corrective action documents were associated with this review:

1. CR 2025-01258, "Unplanned Reactor Shutdown"
2. CR 2025-01429, "Equipment Reliability Challenge Board: Perry is projected to have incurred INPO Noteworthy Consequential Events in each of the last four months"
3. CR 2025-03352, "Perry Escalation - Equipment Reliability"
4. CR 2024-03861, "Equipment Reliability Challenge Board: Corrective Action Program"

During the inspection period, the inspectors evaluated corrective action program entries related to equipment reliability. They were chosen for inspection sample due to their relation to an increased number of forced outages and down powers. The primary events evaluated were turbine trip on loss of vacuum, forced outage due to circulating water system leak, significant down power to stator cooling water system outage, and recirculating water system flow issues on non-safety systems. These events resulted in significant power transients and initiating event cornerstone challenges, driving abnormal plant configurations and increased risk. The licensee also performed a review of this trend and identified a concern of "not leveraging the corrective action program for issue identification may lead to the lack of cross functional team knowledge, understanding, challenge, and correction of equipment reliability issues." The licensee further stated that "Relative to the number consequential events that have occurred at Perry, promotion and reinforcement of a lower corrective action program issue initiation threshold may be required." The licensee documented these conclusions in CR-2025-03861.

The inspectors performed their review in accordance with the inspection procedure and trended the behavior to consider potential safety culture weaknesses. Based on their review they noted a trend associated with CO.4 Expectations: "Leaders frequently communicate and reinforce the expectation that nuclear safety is the organization's overriding priority. Specifically, executives and senior managers reinforce the importance of nuclear safety by clearly communicating its relationship to strategic issues including budget, workforce planning, equipment reliability, and business plans," as defined in NUREG 2165, "Safety Culture Common Language." The inspectors provided this feedback to the licensee to ensure they were taking the necessary actions to address the identified trend. This review focused on the corrective action program aspects of the issues. No performance deficiencies were identified as a result of this review.

Minor Performance Deficiency	71153

LER 2025-001-00 for Perry Nuclear Power Plant, Automatic Actuation of Division 3 Diesel Generator Resulting from Startup Transformer Testing

Minor Performance Deficiency: On March 29, 2025, Unit 1 was in Mode 5 during a refueling outage with a single offsite power source available through the Unit 1 startup transformer. During testing of the Unit 2 startup transformer, power was disrupted to the available but inoperable division 3 shutdown safety electrical bus. This caused an actuation of the division 3 emergency diesel generator (3DG) system and entries into off normal operating procedures for the following systems: electrical power, non-safety service water, and the plant air system. Both the licensee and the NRC resident inspectors became aware of the issue immediately.

This event was reported at 1616 on March 29, 2025, as a 10 CFR 50.72(b)(3)(iv)(A) event resulting in a valid actuation of the emergency diesel generator system.

During the investigation process, the licensee identified relay testing associated with a startup transformer tripped a switch interlock on the breaker that had been energizing the 13.8 kV bus feeding the division 3 shutdown safety electrical bus. They also identified they performed insufficient impact and work order reviews of transformer relay testing. Additionally, engineering provided erroneous technical information regarding breaker control schematics and engineering did not perform a review of a work order revision with the test's steps. Corrective actions included training on electrical interlocks and relay schemes, installing a caution placard on a breaker door, issuing of a night order to operators regarding lessons learned, and incorporating this operating experience into a work order for similar work in the next refueling outage.

The failure to properly review the transformer relay testing impact and work order sufficiently during the planning phase of the work management process was contrary to the licensee's Work Management Process procedure, NOP-WM-0001. The inspectors determined this failure was a performance deficiency. No violation of NRC requirements was identified during this review.

Screening: The inspectors determined the performance deficiency was minor. Though the performance deficiency was associated with the mitigating systems cornerstone attribute of human performance, it did not adversely affect the associated cornerstone objective. Specifically, the 3DG and its supported high pressure core spray pump system were already inoperable to support maintenance and not needed to meet technical specification (TS) requirements. Additionally, both division 1 and division 2 emergency diesel generators and residual heat removal 'B' and 'C' emergency core cooling systems (ECCS) were operable to satisfy ECCS TS 3.5.2 requirements for Modes 5.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On July 16, 2025, the inspectors presented the integrated inspection results to D. Benyak, Plant Manager, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.01	Procedures	IOI-0015	Seasonal Variations	36
71111.04	Procedures	VLI-C11 (CRDH)	Control Rod Drive Hydraulic System (CRDH)	21
		VLI-E22A	High Pressure Core Spray	11
		VLI-E51	Reactor Core Isolation Cooling System	11
		VLI-G41 (FPCC)	Fuel Pool Cooling and Cleanup System	06/26/2025
71111.05	Fire Plans	FPI-1ST	Steam Tunnel	04/09/2025
		FPI-1TB	Turbine Building Unit 1	0
	Procedures	FD 1450-051425	MB100 2nd Floor Classroom	05/14/2025
71111.11Q	Procedures	IOI-0001	Cold Startup	63
71111.12	Corrective Action Documents	CR-2025-02066	Recirc Pump 'A' Discharge Valve Handwheel Broken	03/24/2025
		CR-2025-02528	MSL 'A' OTBD MSIV Leakage Exceeds Allowable Limit	04/06/2025
		CR-2025-02551	FCV 'B' Will Not Stroke Open	04/07/2025
		CR-2025-02766	Mounting Screw Snapped During Reinstallation on FCV 'A' RVDT	04/13/2025
		CR-2025-03227	Abnormal Movement of Flow Control Valve 'A'	04/29/2025
	Engineering Changes	25-1067-001	Reactor Recirculation System Discharge Block Valve (1B33F0067A) Actuator Motor Replacement	0
	Procedures	SVI-B33-T1158B	Reactor Recirculation Flow Control Valve 1B33-F060B Functional Test	0
71111.13	Corrective Action Documents	CR-2025-03401	Turbine Mechanical Overspeed Indicated Test Malfunction	05/03/2025
		CR-2025-03449	Main Transformer 'C' Cooler 1 and 4 Trouble	05/06/2025
	Drawings	2008-0220-00006	TDUN 25-1094-001-001	0
		208-0151-00399	Turbine Cntrl Emergency Trip Sys	1
		208-0220-00006	Power Transformers	R
		209-0220-00020	Temporary Drawing Update Notice 25-1094-001-003	0
	Engineering Changes	ECP 25-0194	Temporary Power Set up for the F1A12 Transformer	05/08/2025
	Procedures	ARI-S11-S001C-0001-A5	Cooler No. 1 Trouble	10
		ARI-S11-S001C-0001-A7	Cooler No. 4 Trouble	10

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		PAP-1925	Shutdown Defense in Depth Assessment and Management	28
	Work Orders	200517736	Cable Diagnostic Withstand Tests	06/16/2025
		200872933	Deenergized/Energized Motor Analysis-LR	06/16/2025
		200933745	Inspect/Tighten ESW 'A' Pump Packing	06/16/2025
71111.15	Corrective Action Documents	CR-2023-05197	Div. 2 DG Exhaust Louver is Dusty and Needs Cleaned	06/29/2023
		CR-2024-06674	Div. 2 Diesel exhaust Damper Screen Dirty	08/19/2024
		CR-2025-03056	Offgas Pretreat Sample Selector Panel Doesn't Have Power	06/09/2025
		CR-2025-03056	Offgas Pretreat Sample Selector Panel Doesn't Have Power	04/22/2025
		CR-2025-03793	Div 2 Diesel Generator Exhaust Screen Need Cleaned	05/19/2025
	Miscellaneous		Perry Open Mode Hold and PCR Forms Mode Restraint List	04/08/2025
	Procedures	ICI-C-C51-3	Intermediate Range Monitoring (IRM) Voltage Preamplifier Calibration	4
71111.18	Work Orders	200922062	RPS Power Transfer Switch Replacement	04/17/2025
71111.24	Corrective Action Documents	CR-2024-09654	Low Pressure Core Spray Waterleg Pump Oil Seals Leaking	12/18/2024
		CR-2025-01189	Drywell Radiation Monitor Inboard Suction Isolation Failed Stroke Time Close Per SVI-D17-T2001	02/23/2025
		CR-2025-01917	Ability to Complete Cable Testing on LH2A is Challenged Due to Inprocessing and Accessing Issues	03/19/2025
		CR-2025-02009	Outboard Main Steam Isolation Valve 'A' Stroke Time Was Fast for SVI-B21-T2001	03/22/2025
		CR-2025-02094	Pre Fab Does Not Match As-Found Condition for LPCS Waterleg Pump to RHR A Discharge Check Valve	03/25/2025
		CR-2025-02192	Phasing Checks Failed on LH-2-A	03/27/2025
		CR-2025-02212	RHR Shutdown Cooling Suction Line Min Path Leakage Limit Exceeded	03/27/2025
		CR-2025-02265	Loss of L20 Bus During Breaker Trip Check Testing	03/29/2025
		CR-2025-02338	Plant Computer Failover During Div 1 Diesel Test	03/31/2025
		CR-2025-02339	1R20 SVI-R43-T7000A Division 1 Integrated Test Data	04/01/2025

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		CR-2025-02655	Main Steam Line 'A' Outboard MSIV As-Found Inspection Results	04/10/2025
	Corrective Action Documents Resulting from Inspection	CR-2025-01985	NRC Identified: Methods for Crediting Systems for Shutdown Defense in Depth Not Clear	03/21/2025
	Engineering Changes	ECP 21-0194-002	PDUN 21-0194-001-002	0
	Engineering Evaluations	601486502	MSIV Spring Cap, Guide Lug Interference	04/11/2025
	Procedures	FTI-F-0038	Hydrostatic Pump Testing Instruction	2
		GEI-0001	Performing Insulation Resistance Checks	17
		ICI-B12-0001	ITT NH90 Series Milliampere Proportional/On-Off Hydramotor Actuator Calibration	17
		ICI-C-51-3	IRM Voltage Preamplifier Calibration	4
		ISI-B21-T1300-1	Reactor Coolant System Leakage Pressure Test	23
		PMI-0061	Overhaul of NH95 Hydramotors	17
		PTI-P54-P3200	Fire Suppression Valve Cycle and Position Switch Functional Test	03/25/2025
		SOI-G33	Reactor Water Cleanup System	59
		SVI-B21-T2001	MSIV Full Stroke Operability Test	16
		SVI-B21-T9000	Type C Local Rate Test of 1B21 MSL Penetration (P122, P124, P415 and P416)	15
		SVI-B21-T9000	Type 'C' Local leak Rate Test of 1B21 MSL Penetrations (P122, P124, P415, and P416)	15
		SVI-C71-T0038E	MSIV 1B21-F028A Closure Channel Calibration for 1C71-N702A and 1C71-N703A	13
		SVI-E12-T2200	RHR Shutdown Cooling Suction Isolation Valve Operability Test	14
		SVI-E12-T2210	Leak Rate Test for RHR to FDW Injection Valves 1E12-F050A and 1E12-F053A	15
		SVI-E21-T2004	LPCS and RHR 'A' Waterleg Pump and Associated Valve Cold Shutdown Operability Test	03/29/2025

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		SVI-E21-T2200	Low Pressure Core Spray Isolation Valve Leak Rate Test	14
		SVI-E22-T1319	Diesel Generator Start and Load Division 3	31
		SVI-E51-T9422	Type C Local Leak Rate Test of 1E51 Penetration P422	13
		SVI-E51-T9422	Type C Local Rate Test of 1E51 Penetration P422	13
		SVI-P53-T9024	Drywell Airlock in Between Seals Test	6
		SVI-R43-T7000-A	Division 1 ECCS Integrated Test	04/01/2025
		SVI-T23-T1022	Drywell Airlock Interlock Verification	5
	Work Orders	200242803	MSIV-CPLO Rebuild OB MSIV 'A'	04/13/2025
		200242803	MSIV-CPLO Rebuild OB MSIV 'A'	04/25/2025
		200679637	1E12-F0084A Dresser Valve Replacement	04/05/2025
		200859641	Perry S-2 Bus Backup Bus Diff-U2S/U - ECP 21-0194-002	04/10/2025
		200903818	Replace High Voltage XFMR Fuse	03/27/2025
		200906321	Static MOV Test (ASME OM Code) & PM Ins	03/29/2025
		200906342	Gen. Insp. / Double Complete Test 1-PUY-T XFMR A0, B0, C0	04/23/2025
		200909389	SVI-B21T2201 MSIV Full Strike OPER Pi-Cal	03/22/2025
		200910660	SVI-C71T0038E OTBD MSIV Closure CH 'A'	04/16/2025
		200911995	SVI-B21T900 1B21 MSL Penetration LLRT	03/24/2025
		200946729	Blast Shield Door to be Opened Closed in the Normal Course. Defeat/Restore DW Interlocks on DW Airlock Hatch. Of IOI-17, Drywell Entry	04/18/2025
		200967423	Replace Hydramotor 1D17F0071B	04/12/2025
		SVI-D17-T9201	Type C Local Leak Rate of 1D17 Penetration P201	11
71114.06	Miscellaneous		Perry ERO Integrated Drill Scenario Guide	06/11/2025
	Procedures	EMA 0083 7/22	State of Ohio Supplemental Action Default Actions - PNPP	06/11/2025
		NOP-LP-5503-09	Perry Emergency Notification Cover Sheet	00
		NOP-LP-5503-12	Industry Event Notification	00
71151	Miscellaneous	MS05 Performance Indicator Data	Safety System Functional Failures	03/31/2025
		MS06 Performance Indicator Data	Emergency AC Power Systems	03/31/2025
		MS07 Performance Indicator Data	High Pressure Injection Systems	03/31/2025

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71152A	Corrective Action Documents	CR-2024-08053	Container in the Outside Radiological Controlled Area Leaked Radioactive Water	10/14/2024
71153	Corrective Action Documents	CR-2025-04315	Fire in Warehouse #6	06/11/2025