

UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

August 12, 2025

Ms. Jamie M. Coleman Regulatory Affairs Director Southern Nuclear Operating Co., Inc. 3535 Colonnade Parkway Birmingham, AL 35243

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2; EDWIN I. HATCH

NUCLEAR PLANT, UNITS 1 AND 2; AND VOGTLE ELECTRIC GENERATING PLANT, UNITS 1, 2, 3, AND 4, ISSUANCE OF AMENDMENTS TO RELOCATE

STAFF QUALIFICATION REQUIREMENTS (EPID L-2025-LLA-0010)

Dear Ms. Coleman:

The Nuclear Regulatory Commission has issued the enclosed Amendment No. 257 to Renewed Facility Operating License No. NPF-2 and Amendment No. 254 to Renewed Facility Operating License No. NPF-8 for the Joseph M. Farley Nuclear Plant (Farley), Units 1 and 2, respectively; Amendment No. 328 to Renewed Facility Operating License No. DPR-57 and Amendment No. 273 to Renewed Facility Operating License No. NPF-5 for the Edwin I. Hatch Nuclear Plant (Hatch), Unit Nos 1 and 2, respectively; Amendment No. 228 to Renewed Facility Operating License NPF-68 and Amendment No. 210 to Renewed Facility Operating License NPF-81 for the Vogtle Electric Generating Plant (Vogtle), Units 1 and 2, respectively; and Amendment No. 204 to Combined License No. NPF-91 and Amendment No. 201 to Combined License No. NPF-92, respectively. The amendments consist of changes to the License and Technical Specifications (TSs) in response to your application dated January 23, 2025, as supplemented by letter dated June 19, 2025.

The amendments revise the respective technical specifications (TSs) regarding "Unit Staff Qualifications," by removing the licensee's staff qualification requirements from the TSs and relocating the control of unit staff qualification requirements to the licensee controlled Southern Nuclear Operating Company Quality Assurance Topical Report (QATR) for Farley, Units 1 and 2, Hatch, Units 1 and 2, and Vogtle, Units 1 and 2, and the Nuclear Development Quality Assurance Manual (NDQAM) for Vogtle, Units 3 and 4, consistent with the guidance in NRC Administrative Letter (AL) 95-06, "Relocation of Technical Specification Administrative Controls Related to Quality Assurance" (Agencywide Documents Access and Management System (ADAMS) Accession No. ML031110271).

A copy of the related safety evaluation is also enclosed. A Notice of Issuance will be included in the Commission's monthly *Federal Register* notice.

J. Coleman - 2 -

If you have questions, you can contact me at 301-415-3100 or John.Lamb@nrc.gov.

Sincerely,

/RA

John G. Lamb, Senior Project Manager Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Docket Nos. 50-348, 50-364, 50-321, 50-366, 50-424, 50-425, 52-025, and 52-026

Enclosures:

- 1. Amendment No. 257 to NPF-2
- 2. Amendment No. 254 to NPF-8
- 3. Amendment No. 328 to DPR-57
- 4. Amendment No. 273 to NPF-5
- 5. Amendment No. 228 to NPF-68
- 6. Amendment No. 210 to NPF-81
- 7. Amendment No. 204 to NPF-91
- 8. Amendment No. 201 to NPF-92
- 9. Safety Evaluation

cc: Listserv



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-348

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 257 Renewed License No. NPF-2

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated January 23, 2025, as supplemented by letter dated June 19, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-2 is hereby amended to read as follows:

(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 257, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 365 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael Markley, Chief Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 12, 2025



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

ALABAMA POWER COMPANY

DOCKET NO. 50-364

JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 254 Renewed License No. NPF-8

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern Nuclear Operating Company, Inc. (Southern Nuclear), dated January 23, 2025, as supplemented by letter dated June 19, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this license amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Technical Specifications, as indicated in the attachment to this license amendment; and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-8 is hereby amended to read as follows:

(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 254, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of its date of issuance and shall be implemented within 365 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael Markley, Chief Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment:
Changes to the Technical
Specifications

Date of Issuance: August 12, 2025

ATTACHMENT TO LICENSE AMENDMENT NO. 257 JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 1

TO RENEWED FACILITY OPERATING LICENSE NO. NPF-2

DOCKET NO. 50-348

<u>AND</u>

ATTACHMENT TO LICENSE AMENDMENT NO. 254 JOSEPH M. FARLEY NUCLEAR PLANT, UNIT 2 TO RENEWED FACILITY OPERATING LICENSE NO. NPF-8

DOCKET NO. 50-364

Replace the following pages of the License and Appendix A Technical Specifications (TSs) with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Pages	Insert Pages	
<u>License</u>	<u>License</u>	
License No. NPF-2, page 4 License No. NPF-8, page 3	License No. NPF-2, page 4 License No. NPF-8, page 3	
<u>TSs</u>	<u>TSs</u>	
5.3-1	5.3-1	

(2) <u>Technical Specifications</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 257, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the Issuance of the renewed license or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the renewed license supported by a favorable evaluation by the Commission.

- a. Southern Nuclear shall not operate the reactor in Operational Modes 1 and 2 with less than three reactor coolant pumps in operation.
- b. Deleted per Amendment 13
- c. Deleted per Amendment 2
- d. Deleted per Amendment 2
- e. Deleted per Amendment 152
 - Deleted per Amendment 2
- f. Deleted per Amendment 158
- g. Southern Nuclear shall maintain a secondary water chemistry monitoring program to inhibit steam generator tube degradation. This program shall include:
 - 1) Identification of a sampling schedule for the critical parameters and control points for these parameters;
 - 2) Identification of the procedures used to quantify parameters that are critical to control points;
 - 3) Identification of process sampling points;
 - 4) A procedure for the recording and management of data;
 - 5) Procedures defining corrective actions for off control point chemistry conditions; and

- (2) Alabama Power Company, pursuant to Section 103 of the Act and 10 CFR Part 50, "Licensing of Production and Utilization Facilities," to possess but not operate the facility at the designated location in Houston County, Alabama in accordance with the procedures and limitations set forth in this renewed license.
- (3) Southern Nuclear, pursuant to the Act and 10 CFR Part 70, to receive, possess and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
- (4) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to receive, possess, and use in amounts as required any byproducts, source or special nuclear material without restriction to chemical or physical form for sample analysis or instrument calibration or associated with radioactive apparatus or components; and
- (6) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- C. This renewed license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter I and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect; and is subject to the additional conditions specified or incorporate below:
 - (1) <u>Maximum Power Level</u>

Southern Nuclear is authorized to operate the facility at reactor core power levels not in excess of 2821 megawatts thermal.

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 254, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications.

- (3) Delete per Amendment 144
- (4) Delete Per Amendment 149
- (5) Delete per Amend 144

5.0 ADMINISTRATIVE CONTROLS

5.3 Unit Staff Qualifications

5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications specified in the SNC Quality Assurance Topical Report.



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-321

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 328 Renewed License No. DPR-57

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit No. 1 (the facility) Renewed Facility Operating License No. DPR-57 filed by Southern Nuclear Operating Company, Inc. (the licensee), acting for itself, Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the owners), dated January 23, 2025, as supplemented by letter dated June 19, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and the first paragraph of paragraph 2.C.(2) of Renewed Facility Operating License No. DPR-57 is hereby amended to read as follows:

(2) <u>Technical Specifications</u>

The Technical Specifications (Appendix A) and the Environmental Protection Plan (Appendix B), as revised through Amendment No. 328, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 365 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael Markley, Chief Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment:
Changes to Renewed Facility
Operating License No. DPR-57
and Technical Specifications

Date of Issuance: August 12, 2025

ATTACHMENT TO LICENSE AMENDMENT NO. 328

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 1

RENEWED FACILITY OPERATING LICENSE NO. DPR-57

DOCKET NO. 50-321

Replace the following pages of the License and the Appendix A Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages	<u>Insert Pages</u>	
<u>License</u>	<u>License</u>	
License No. DPR-57, page 4	License No. DPR-57, page	
<u>TSs</u>	<u>TSs</u>	
5.0-5	5.0-5	

4

for sample analysis or instrument calibration, or associated with radioactive apparatus or components

- (6) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- (C) This renewed license shall be deemed to contain, and is subject to, the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 of Part 50, and Section 70.32 of Part 70; all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and the additional conditions specified or incorporated below:

(1) Maximum Power Level

Southern Nuclear is authorized to operate the facility at steady-state reactor core power levels not in excess of 2,804 megawatts thermal.

(2) <u>Technical Specifications</u>

The Technical Specifications (Appendix A) and the Environmental Protection Plan (Appendix B), as revised through Amendment No. 328, are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

The Surveillance Requirement (SR) contained in the Technical Specifications and listed below, is not required to be performed immediately upon implementation of Amendment No. 195. The SR listed below shall be successfully demonstrated before the time and condition specified:

SR 3.8.1.18 shall be successfully demonstrated at its next regularly scheduled performance.

(3) <u>Fire Protection</u>

Southern Nuclear Operating Company shall implement and maintain in effect all provisions of the approved fire protection program that comply with 10 CFR 50.48(a) and 10 CFR 50.48(c), as specified in the licensee amendment request dated April 4, 2018, supplemented by letters dated May 28, August 9, October 7, and December 13, 2019, and February 5, and March 13, 2020, and as approved in the NRC safety evaluation (SE) dated June 11, 2020. Except where NRC approval for changes or deviations is required by 10 CFR 50.48(c), and provided no other regulation, technical specification, license condition or requirement would require prior NRC approval, the licensee may make changes to the fire protection program without prior approval of the Commission if those changes satisfy the provisions set forth in 10 CFR 50.48(a) and 10 CFR 50.48(c), the change does not require a change to a technical specification or a license condition, and the criteria listed below are satisfied.

- 5.0 ADMINISTRATIVE CONTROLS
- 5.3 Unit Staff Qualifications
- 5.3.1 Each member of the unit staff shall either meet or exceed the minimum qualifications specified in the SNC Quality Assurance Topical Report.



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-366

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 273 Renewed License No. NPF-5

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Edwin I. Hatch Nuclear Plant, Unit No. 2 (the facility) Renewed Facility Operating License No. NPF-5 filed by Southern Nuclear Operating Company, Inc. (the licensee), acting for itself, Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the owners), dated January 23, 2025, as supplemented by letter dated June 19, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-5 is hereby amended to read as follows:

(2) <u>Technical Specifications</u>

The Technical Specifications (Appendix A) and the Environmental Protection Plan (Appendix B), as revised through Amendment No. 273 are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 365 days from the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael Markley, Chief Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment:
Changes to Renewed Facility
Operating License No. NPF-5
and Technical Specifications

Date of Issuance: August 12, 2025

ATTACHMENT TO LICENSE AMENDMENT NO. 273

EDWIN I. HATCH NUCLEAR PLANT, UNIT NO. 2

RENEWED FACILITY OPERATING LICENSE NO. NPF-5

DOCKET NO. 50-366

Replace the following pages of the License and the Appendix A Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Insert Pages

<u>License</u>

License No. NPF-5, page 4

License No. NPF-5, page 4

TSs TSs

Remove Pages

5.0-5

- (6) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30 and 70, to possess, but not separate, such byproduct and special nuclear materials as may be produced by the operation of the facility.
- (C) This renewed license shall be deemed to contain, and is subject to, the conditions specified in the following Commission regulations in 10 CFR Chapter I: Part 20, Section 30.34 of Part 30, Section 40.41 of Part 40, Section 50.54 of Part 50, and Section 70.32 of Part 70; all applicable provisions of the Act and the rules, regulations, and orders of the Commission now or hereafter in effect; and the additional conditions² specified or incorporated below:

(1) <u>Maximum Power Level</u>

Southern Nuclear is authorized to operate the facility at steady sate reactor core power levels not in excess of 2,804 megawatts thermal, in accordance with the conditions specified herein.

(2) <u>Technical Specifications</u>

The Technical Specifications (Appendix A) and the Environmental Protection Plan (Appendix B); as revised through Amendment No. 273 are hereby incorporated in the renewed license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

(3) Additional Conditions

The matters specified in the following conditions shall be completed to the satisfaction of the Commission within the stated time periods following the issuance of the renewed license or within the operational restrictions indicated. The removal of these conditions shall be made by an amendment to the license supported by a favorable evaluation by the Commission.

(a) Fire Protection

Southern Nuclear Operating Company shall implement and maintain in effect all provisions of the approved fire protection program that comply with 10 CFR 50.48(a) and 10 CFR 50.48(c), as specified in the licensee amendment request dated April 4, 2018, supplemented by letters dated May 28, August 9, October 7, and December 13, 2019, and February 5, and March 13, 2020, and as approved in the NRC safety evaluation (SE) dated June 11, 2020. Except where NRC approval for changes or deviations is required by 10 CFR 50.48(c), and provided no other regulation, technical specification, license condition or requirement would

The original licensee authorized to possess, use, and operate the facility with Georgia Power Company (GPC). Consequently, certain historical references to GPC remain in certain license conditions.

- 5.0 ADMINISTRATIVE CONTROLS
- 5.3 Unit Staff Qualifications
- 5.3.1 Each member of the unit staff shall either meet or exceed the minimum qualifications specified in the SNC Quality Assurance Topical Report.



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-424

VOGTLE ELECTRIC GENERATING PLANT, UNIT 1

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 228 Renewed License No. NPF-68

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 1 (the facility) Renewed Facility Operating License No. NPF-68 filed by the Southern Nuclear Operating Company, Inc. (the licensee), acting for itself, Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the owners), dated January 23, 2025, as supplemented by letter dated June 19, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

- E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- 2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-68 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 228, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 365 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael Markley, Chief Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment: Changes to License No. NPF-68 and the Technical Specifications

Date of Issuance: August 12, 2025



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MUNICIPAL ELECTRIC AUTHORITY OF GEORGIA

CITY OF DALTON, GEORGIA

DOCKET NO. 50-425

VOGTLE ELECTRIC GENERATING PLANT, UNIT 2

AMENDMENT TO RENEWED FACILITY OPERATING LICENSE

Amendment No. 210 Renewed License No. NPF-81

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment to the Vogtle Electric Generating Plant, Unit 2 (the facility) Renewed Facility Operating License No. NPF-81 filed by the Southern Nuclear Operating Company, Inc. (the licensee), acting for itself, Georgia Power Company Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia (the owners), dated January 23, 2025, as supplemented by letter dated June 19, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachment to this license amendment, and the first paragraph of paragraph 2.C.(2) of Renewed Facility Operating License No. NPF-81 is hereby amended to read as follows:

Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 210, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance and shall be implemented within 365 days of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael Markley, Chief Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment: Changes to License No. NPF-81 and the Technical Specifications

Date of Issuance: August 12, 2025

<u>ATTACHMENT</u>

VOGTLE ELECTRIC GENERATING PLANT, UNITS 1 AND 2

TO LICENSE AMENDMENT NO. 228

RENEWED FACILITY OPERATING LICENSE NO. NPF-68

DOCKET NO. 50-424

<u>AND</u>

TO LICENSE AMENDMENT NO. 210

RENEWED FACILITY OPERATING LICENSE NO. NPF-81

DOCKET NO. 50-425

Replace the following pages of the Licenses and the Appendix A Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages	<u>Insert Pages</u>		
<u>License</u> License No. NPF-68, page 4 License No. NPF-81, page 3	<u>License</u> License No. NPF-68, page 4 License No. NPF-81, page 3		
TSs	<u>TSs</u>		
5.3-1	5.3-1		

(1) Maximum Power Level

Southern Nuclear is authorized to operate the facility at reactor core power levels not in excess of 3625.6 megawatts thermal (100 percent power) in accordance with the conditions specified herein.

(2) <u>Technical Specifications and Environmental Protection Plan</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 228, and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

- (3) Southern Nuclear Operating Company shall be capable of establishing containment hydrogen monitoring within 90 minutes of initiating safety injection following a loss of coolant accident.
- (4) Deleted
- (5) Deleted
- (6) Deleted
- (7) Deleted
- (8) Deleted
- (9) Deleted

(10) Mitigation Strategy License Condition

The licensee shall develop and maintain strategies for addressing large fires and explosions and that include the following key areas:

- (a) Fire fighting response strategy with the following elements:
 - 1. Pre-defined coordinated fire response strategy and guidance
 - 2. Assessment of mutual aid fire fighting assets
 - 3. Designated staging areas for equipment and materials
 - Command and control
 - 5. Training and response personnel
- (b) Operations to mitigate fuel damage considering the following:
 - 1. Protection and use of personnel assets
 - 2. Communications
 - 3. Minimizing fire spread
 - 4. Procedures for Implementing integrated fire response strategy
 - 5. Identification of readily-available pre-staged equipment
 - 6. Training on integrated fire response strategy

- (2) Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, and City of Dalton, Georgia, pursuant to the Act and 10 CFR Part 50, to possess but not operate the facility at the designated location in Burke County, Georgia, in accordance with the procedures and limitations set forth in this license;
- (3) Southern Nuclear, pursuant to the Act and 10 CFR Part 70, to receive, possess, and use at any time special nuclear material as reactor fuel, in accordance with the limitations for storage and amounts required for reactor operation, as described in the Final Safety Analysis Report, as supplemented and amended;
- (4) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40, and 70 to receive, possess, and use at any time any byproduct, source and special nuclear material as sealed neutron sources for reactor startup, sealed sources for reactor instrumentation and radiation monitoring equipment calibration, and as fission detectors in amounts as required;
- (5) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40, and 70, to receive, possess, and use in amounts as required any byproduct, source or special nuclear material without restriction to chemical or physical form, for sample analysis or instrument calibration or associated with radioactive apparatus or components;
- (6) Southern Nuclear, pursuant to the Act and 10 CFR Parts 30, 40 and 70, to possess, but not separate, such byproduct and special nuclear materials as my be produced by the operation of the facility authorized herein.
- C. This license shall be deemed to contain and is subject to the conditions specified in the Commission's regulations set forth in 10 CFR Chapter 1 and is subject to all applicable provisions of the Act and to the rules, regulations, and orders of the Commission now or hereafter in effect, and is subject to the additional conditions specified or incorporated below.

(1) Maximum Power Level

Southern Nuclear is authorized to operate the facility at reactor core power levels not in excess of 3625.6 megawatts thermal (100 percent power) in accordance with the conditions specified herein.

(2) <u>Technical Specifications and Environmental Protection Plan</u>

The Technical Specifications contained in Appendix A, as revised through Amendment No. 210 and the Environmental Protection Plan contained in Appendix B, both of which are attached hereto, are hereby incorporated into this license. Southern Nuclear shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

The Surveillance requirements (SRs) contained in the Appendix A Technical Specifications and listed below are not required to be performed immediately upon implementation of Amendment No. 74. The SRs listed below shall be

5.0 ADMINISTRATIVE CONTROLS

5.3 Unit Staff Qualifications

5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications specified in the SNC Quality Assurance Topical Report.



UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MEAG POWER SPVM, LLC

MEAG POWER SPVJ, LLC

MEAG POWER SPVP, LLC

CITY OF DALTON, GEORGIA

VOGTLE ELECTRIC GENERATING PLANT, UNIT 3

DOCKET NO. 52-025

AMENDMENT TO FACILITY COMBINED LICENSE

Amendment No. 204 License No. NPF-91

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern Nuclear Operating Company, Inc., (SNC), dated January 23, 2025, as supplemented by letter dated June 19, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will be constructed and will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Combined License Condition 2.D.(8) as well as Appendix A of the facility Combined License as indicated in the attachment to this license amendment. Paragraph 2.D.(8) of facility Combined License No. NPF-91 is hereby amended to read as follows:

(8) <u>Incorporation</u>

The Technical Specifications and Environmental Protection Plan in Appendices A and B, respectively, of this license, as revised through Amendment No. 204, are hereby incorporated into this license.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 365 days following the first Unit 4 refueling outage or within 365 days following issuance of the amendments, whichever is later.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael Markley, Chief Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment(s):
Page 4 of the facility Combined License and affected pages of Appendix A of the facility Combined License

Date of Issuance: August 12, 2025



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

GEORGIA POWER COMPANY

OGLETHORPE POWER CORPORATION

MEAG POWER SPVM, LLC

MEAG POWER SPVJ, LLC

MEAG POWER SPVP, LLC

CITY OF DALTON, GEORGIA

VOGTLE ELECTRIC GENERATING PLANT, UNIT 4

DOCKET NO. 52-026

AMENDMENT TO FACILITY COMBINED LICENSE

Amendment No. 201 License No. NPF-92

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Southern Nuclear Operating Company, Inc., (SNC), dated January 23, 2025, as supplemented by letter dated June 19, 2025, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will be constructed and will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations, and all applicable requirements have been satisfied.

2. Accordingly, the license is amended by changes to the Combined License Condition 2.D.(8) as well as Appendix A of the facility Combined License as indicated in the attachment to this license amendment. Paragraph 2.D(8) of facility Combined License No. NPF-92 is hereby amended to read as follows:

(8) <u>Incorporation</u>

The Technical Specifications and Environmental Protection Plan in Appendices A, and B, respectively of this license, as revised through Amendment No. 201, are hereby incorporated into this license.

3. This license amendment is effective as of the date of its issuance and shall be implemented within 365 days of issuance or 365 days after the first refueling outage (fall 2025), whichever is greater.

FOR THE NUCLEAR REGULATORY COMMISSION

Michael Markley, Chief Plant Licensing Branch II-1 Division of Operating Reactor Licensing Office of Nuclear Reactor Regulation

Attachment(s):
Page 4 of the facility Combined License and affected pages of Appendix A of the facility Combined License

Date of Issuance: August 12, 2025

<u>ATTACHMENT</u>

VOGTLE ELECTRIC GENERATING PLANT, UNITS 3 AND 4

TO LICENSE AMENDMENT NO. 204

FACILITY COMBINED LICENSE NO. NPF-91

DOCKET NO. 52-025

<u>AND</u>

TO LICENSE AMENDMENT NO. 201

FACILITY COMBINED LICENSE NO. NPF-92

DOCKET NO. 52-026

Replace the following pages of the Licenses and the Appendix A Technical Specifications (TSs) with the attached revised pages. The revised pages are identified by amendment number and contain marginal lines indicating the areas of change.

Remove Pages	<u>Insert Pages</u>		
<u>License</u> License No. NPF-91, page 4 License No. NPF-92, page 4	<u>License</u> License No. NPF-91, page 4 License No. NPF-92, page 4		
<u>TSs</u>	<u>TSs</u>		
5.3-1	5.3-1		

- D. The license is subject to, and SNC shall comply with, the conditions specified and incorporated below:
 - (1) <u>Changes during Construction</u> Removed by Amendment No. 202
 - (2) <u>Pre-operational Testing</u> Removed by Amendment Nos. 192 and 202
 - (3) <u>Nuclear Fuel Loading and Pre-critical Testing</u> Removed by Amendment Nos. 192 and 202
 - (4) <u>Initial Criticality and Low-Power Testing</u> Removed by Amendment No. 202
 - (5) <u>Power Ascension Testing</u> Removed by Amendment No. 202
 - (6) Maximum Power Level

SNC is authorized to operate the facility at steady state reactor core power levels not to exceed 3400 MW thermal (100-percent thermal power), as described in the UFSAR, in accordance with the conditions specified herein.

- (7) Reporting Requirements Removed by Amendment No. 202
- (8) <u>Incorporation</u>

The Technical Specifications and Environmental Protection Plan in Appendices A and B, respectively, of this license, as revised through Amendment No. 204, are hereby incorporated into this license.

- (9) <u>Technical Specifications</u> Removed by Amendment No. 202
- (10) Operational Program Implementation Removed by Amendment No. 202
- (11) <u>Operational Program Implementation Schedule</u> Removed by Amendment No. 202
- (12) <u>Site- and Unit-specific Conditions</u> Removed by Amendment No. 202

[Blank Pages 5 through 14 removed by Amendment No. 202.]

- D. The license is subject to, and SNC shall comply with, the conditions specified and incorporated below:
 - (1) Changes during Construction Removed by Amendment No. 199
 - (2) Pre-operational Testing Removed by Amendment Nos. 194 and 199
 - (3) <u>Nuclear Fuel Loading and Pre-critical Testing</u> Removed by Amendment Nos. 194 and 199
 - (4) <u>Initial Criticality and Low-Power Testing</u> Removed by Amendment No. 199
 - (5) <u>Power Ascension Testing</u> Removed by Amendment No. 199
 - (6) Maximum Power Level

SNC is authorized to operate the facility at steady state reactor core power levels not to exceed 3400 MW thermal (100-percent thermal power), as described in the UFSAR, in accordance with the conditions specified herein.

- (7) Reporting Requirements Removed by Amendment No. 199
- (8) <u>Incorporation</u>

The Technical Specifications and Environmental Protection Plan in Appendices A and B, respectively, of this license, as revised through Amendment No. 201, are hereby incorporated into this license.

- (9) Technical Specifications Removed by Amendment No. 199
- (10) Operational Program Implementation Removed by Amendment No. 199
- (11) <u>Operational Program Implementation Schedule</u> Removed by Amendment No. 199
- (12) Site- and Unit-specific Conditions Removed by Amendment No. 199

[Blank Pages 5 through 14 removed by Amendment No. 199.]

5.0 ADMINISTRATIVE CONTROLS

5.3 Unit Staff Qualifications

- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications specified in either the SNC Nuclear Development Quality Assurance Manual or the SNC Quality Assurance Topical Report.
- 5.3.2 For the purpose of 10 CFR 55.4, a licensed Senior Reactor Operator (SRO) and a licensed reactor operator (RO) are those individuals who, in addition to meeting the requirements of TS 5.3.1, perform the functions described in 10 CFR 50.54(m).



UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D.C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION RELATED TO

AMENDMENT NO. 257 TO RENEWED FACILITY OPERATING LICENSE NPF-2

AMENDMENT NO. 254 TO RENEWED FACILITY OPERATING LICENSE NPF-8

AMENDMENT NO. 328 TO RENEWED FACILITY OPERATING LICENSE DPR-57

AMENDMENT NO. 273 TO RENEWED FACILITY OPERATING LICENSE NPF-5

AMENDMENT NO. 228 TO RENEWED FACILITY OPERATING LICENSE NPF-68

AMENDMENT NO. 210 TO RENEWED FACILITY OPERATING LICENSE NPF-81

AMENDMENT NO. 204 TO FACILITY COMBINED LICENSE NPF-91

AMENDMENT NO. 201 TO FACILITY COMBINED LICENSE NPF-92

FOR

JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2

EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2

VOGTLE ELECTRIC GENERATING PLANT, UNITS 1, 2, 3, AND 4

SOUTHERN NUCLEAR OPERATING COMPANY, INC.

DOCKET NOS. 50-348, 50-364, 50-321, 50-366, 50-424 50-425, 52-025, AND 52-026

1.0 INTRODUCTION

By application dated January 23, 2025 (Agencywide Documents Access and Management System (ADAMS) Accession No. ML25023A218), as supplemented by letter dated June 19, 2025 (ML25170A016), Southern Nuclear Operating Company (SNC, the licensee) submitted a license amendment request (LAR) for the Joseph M. Farley Nuclear Plant (Farley), Units 1 and 2; Edwin I. Hatch Nuclear Plant (Hatch), Units 1 and 2; and Vogtle Electric Generating Plant (Vogtle), Units 1, 2, 3, and 4. The proposed amendments would revise technical specifications (TS) to relocate control of the requirements related to staff qualifications to the SNC Quality Assurance Topical Report (QATR) and the SNC Nuclear Development Quality Assurance

Manual (NDQAM), as applicable, consistent with the guidance in NRC Administrative Letter (AL) 95-06, "Relocation of Technical Specification Administrative Controls Related to Quality Assurance" (ADAMS Accession No. ML031110271).

The licensee's submittal dated January 23, 2025, as supplemented by letter dated June 19, 2025, included two enclosures: Enclosure 1, "Description and Justification of Quality Assurance Program Changes," and Enclosure 2, "Description and Assessment of the Proposed Technical Specification Changes." The NRC review and this safety evaluation (SE) focused on the TS changes described in Enclosure 2. This SE does reference information contained in Enclosure 1, however, the NRC staffs review, and approval of information contained in Enclosure 1 was the subject of a separate review and approval (ML25198A201).

The supplement, dated June 19, 2025, provided additional information that clarified the application, did not expand the scope of the application as originally noticed, and did not change the NRC staff's original proposed no significant hazards consideration determination as published in the *Federal Register*.

2.0 <u>REGULATORY EVALUATION</u>

The NRC staff considered the following regulatory requirements in its review of the licensee's application:

- Title 10 of the Code of Federal Regulations (10 CFR) 50.36, "Technical specifications," provides requirements for the content of the TS in the category "Administrative Controls." Paragraph (c)(5) of 10 CFR 50.36 requires, in part, that technical specifications include administrative controls, which are "provisions relating to organization and management, procedures, recordkeeping, review and audit, and reporting necessary to assure operation of the facility in a safe manner."
- 10 CFR 50.120, "Training and qualification of nuclear power plant personnel," requires training and qualification for personnel to operate and maintain the facility in a safe manner in all modes of operation.
- Appendix B, "Quality Assurance Criteria for Nuclear Power Plants and Fuel Reprocessing Plants," to the Title 10 of the Code of Federal Regulations (10 CFR) Part 50, "Domestic Licensing of Production and Utilization Facilities," which provides requirements for design, fabrication, construction, and testing of structures, systems, and components, and the managerial and administrative controls used to ensure safe operations.
- 10 CFR 50.54(a)(4), which provides that a licensee may make a change to a previously accepted quality assurance program description included or referenced in the Safety Analysis Report that reduce commitments, provided they receive prior NRC approval.

The NRC staff considered the following guidance in its review of the licensee's application:

 NUREG-0800, "Standard Review Plan for the Review of Safety Analysis Reports for Nuclear Power Plants: LWR Edition," Chapter 13, "Conduct of Operations," Section 13.2.2, "Non-Licensed Plant Staff Training," Revision 4 (ADAMS Accession No. ML15006A129) August 2016, provides guidance for the NRC review of training programs for non-licensed plant staff.

- Regulatory Guide (RG) 1.8, Revision 4, "Qualification and Training of Personnel for Nuclear Power Plants," June 2019 (ADAMS Accession No. ML19101A395), describes a method acceptable for complying with those portions of the Commission's regulations with regard to the training and qualifications of nuclear power plant personnel.
- NRC AL 95-06, "Relocation of Technical Specification Administrative Controls Related to Quality Assurance," dated December 12, 1995 (ADAMS Accession No. ML031110271), provides guidance for relocating requirements from the administrative controls section of TSs in cases where adequate controls are provided by other methods. AL 95-06 notes that the quality assurance program is a logical candidate for such relocations due to the controls imposed on the quality assurance program by regulations (e.g., 10 CFR 50, Appendix B, and 10 CFR 50.54(a)), and the existence of NRC-approved quality assurance plans and commitments to industry quality assurance standards.

3.0 TECHNICAL EVALUATION

- 3.1 Farley, Units 1 and 2
- 3.1.1 Proposed Technical Specification Changes

Current Farley, Units 1 and 2, TS 5.3.1 states:

5.3.1 Each member of the unit staff, including plant manager, shall meet or exceed the minimum qualifications of ANSI N18.1-1971 for comparable positions and the supplemental requirements specified in 10 CFR 55, except for (1) the senior individual in charge of radiation protection who shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975. Personnel who complete an accredited program which has been endorsed by the NRC shall meet the requirements of the accredited program in lieu of the above. The operations manager shall meet or exceed the above requirements except that Technical Specification 5.2.2.f shall specify the requirements regarding the holding of an SRO license.

Revised Farley, Units 1 and 2, TS 5.3.1 would state:

5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications specified in the SNC Quality Assurance Topical Report.

3.2 Hatch, Units 1 and 2

3.2.1 Proposed Technical Specification Changes

Current Hatch, Unit 1, TS 5.3.1 states:

5.3.1 Each member of the unit staff, including plant manager, shall either meet or exceed the minimum qualifications of ANSI N18.1-1971, or shall meet or exceed the minimum qualifications of the accredited program

requirements for those positions stipulated in Enclosure 1 to letter NL-07-1925. The operations manager shall meet or exceed the above requirements except that Technical Specification 5.2.2.f shall specify the requirements regarding the holding of an SRO license. The senior individual in charge of radiation protection shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

Revised Hatch, Unit 1, TS 5.3.1 would state:

5.3.1 Each member of the unit staff shall either meet or exceed the minimum qualifications specified in the SNC Quality Assurance Topical Report.

Current Hatch, Unit 2 TS 5.3.1 states:

5.3.1 Each member of the unit staff, including plant manager, shall either meet or exceed the minimum qualifications of ANSI N18.1-1971, or shall meet or exceed the minimum qualifications of the accredited program requirements for those positions stipulated in Enclosure 1 to letter NL-07-1925. The operations manager shall meet or exceed the above requirements except that Technical Specification 5.2.2.f shall specify the requirements regarding the holding of an SRO license. The senior individual in charge of radiation protection shall meet or exceed the qualifications of Regulatory Guide 1.8, September 1975.

Revised Hatch, Unit 2, TS 5.3.1 to state:

- 5.3.1 Each member of the unit staff shall either meet or exceed the minimum qualifications specified in the SNC Quality Assurance Topical Report.
- 3.3 Vogtle, Units 1 and 2
- 3.3.1 Proposed Technical Specification Changes

Current Vogtle, Units 1 and 2, TS 5.3.1 states:

5.3.1 Each member of the unit staff, including plant manager, shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, Revision 2, 1987, and, for licensed staff, 10 CFR 55.59. Prior to meeting the recommendations of Regulatory Guide 1.8, Revision 2, personnel may be trained to perform specific tasks and will be qualified to perform those tasks independently. Personnel who complete an accredited program which has been endorsed by the NRC shall meet the requirements of the accredited program in lieu of the above. The operations manager shall meet or exceed the above requirements except that Technical Specification 5.2.2.f shall specify the requirements regarding the holding of an SRO license.

Revised Vogtle, Units 1 and 2, TS 5.3.1 would state:

5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications specified in the SNC Quality Assurance Topical Report.

3.4 Vogtle, Units 3 and 4

3.4.1 Proposed Technical Specification Changes

Current Vogtle, Units 3 and 4, TS 5.3.1 states:

- 5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications of Regulatory Guide 1.8, Revision 3, 2000, with the following exception:
 - a. During cold license operator training through the first refueling outage, the Regulatory Position C.1.b of Regulatory Guide 1.8, Revision 2, 1987, applies: cold license operator candidates meet the training elements defined in ANSI/ANS 3.1-1993 but are exempt from the experience requirements defined in ANSI/ANS 3.1-1993.

Revised Vogtle, Units 3 and 4, TS 5.3.1 would state:

5.3.1 Each member of the unit staff shall meet or exceed the minimum qualifications specified in either the SNC Nuclear Development Quality Assurance Manual or the SNC Quality Assurance Topical Report.

3.5 NRC Staff Evaluation

In deciding whether an amendment to a license will be issued, the NRC staff is guided by the considerations that govern the issuance of initial licenses to the extent applicable and appropriate. The NRC staff evaluated the licensee's request to decide whether the proposed change is consistent with the regulations, licensing basis, and guidance discussed above in this safety evaluation. The NRC staff reviewed the proposed changes to decide whether relocating the unit staff qualification requirements in TS 5.3.1 to the quality assurance program documents for the SNC Fleet will continue to adequately define the unit staff qualification requirements sufficient to meet the requirements of 10 CFR 50.36(c)(5) and assure that the referenced pant staff qualification requirements relocated to the QATR and NDQAM will adequately maintain the training and qualification programs in accordance with 10 CFR 50.120.

In its request, SNC stated that the proposed amendment is consistent with NRC AL 95-06. AL 95-06 informed licensees about recent experiences involving the relocation of technical specification administrative controls related to quality assurance. Consistent with AL 95-06, the proposed amendment will relocate the control of plant staff qualification requirements currently located in TS Sections 5.3.1, as described above, to the SNC QATR or the SNC NDQAM as applicable.

The SNC Fleet unit staff qualifications in TS currently follow prior versions of RG 1.8 or its endorsed standards. In a separate review activity (Enclosure 1 to the LAR), in accordance with 10 CFR 50.54(a)(4), SNC requested approval of a proposed change to the QATR and NDQAM. In the proposed change, SNC would revise its QATR and NDQAM to satisfy Regulatory Guide (RG) 1.8 Revision 4, rather than prior versions of RG 1.8 or its endorsed standards, per the applicable TS.

The NRC staff reviewed this proposal in conjunction with this TS 5.3.1 update, and approved adoption of program commitment to RG 1.8, Revision 4, without exception in the associated 10 CFR 50.54(a)(4) program update review (ML25198A201).

Based on the above, the NRC staff concludes that the proposed change to relocate control of the SNC Fleet unit staff qualifications from the TS to the quality assurance program documents is acceptable. The proposed change is consistent with guidance contained in AL 95-06 and the training and qualification requirements will continue to be acceptably mandated in the SNC QATR or the SNC NDQAM, as applicable. In addition, future changes to these requirements in the SNC QATR or the SNC NDQAM would require prior NRC approval, in accordance with 10 CFR 50.54(a)(4), unless the changes did not reduce the commitments in the program description as accepted by the NRC. Therefore, the NRC staff finds that the proposed change will continue to meet 10 CFR 50.36(c)(5) because the TS changes continue to assure operation of the facility in a safe manner. The NRC staff also finds the proposed change will continue to meet 10 CFR 50.120 because the changes to the quality assurance program documents continue to ensure unit staff training and qualification requirements are sufficient.

4.0 STATE CONSULTATION

In accordance with the Commission's regulations, the Alabama State and Georgia State officials were notified on July 21, 2025. On July 21, 2025, the State official of Alabama stated that they had no comments. On July 25, 2025, the State official of Georgia stated that they had no comments.

5.0 ENVIRONMENTAL CONSIDERATION

The amendment relates to changes in recordkeeping, reporting, or administrative procedures or requirements. The Commission has previously issued a proposed finding that the amendments involve no significant hazards consideration, as published in the *Federal Register* on March 18, 2025 (90 FR 12572), and there has been no public comment on such finding. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared in connection with the issuance of the amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that: (1) there is reasonable assurance that the health and safety of the public will not be endangered by operation in the proposed manner, (2) there is reasonable assurance that such activities will be conducted in compliance with the Commission's regulations, and (3) the issuance of the amendments will not be inimical to the common defense and security or to the health and safety of the public.

Principal Contributors: A. Keim, NRR

J. Davis, NRR C. Ashley, NRR

Date: August 12, 2025

J. Coleman - 3 -

SUBJECT: JOSEPH M. FARLEY NUCLEAR PLANT, UNITS 1 AND 2; EDWIN I. HATCH

NUCLEAR PLANT, UNITS 1 AND 2; AND VOGTLE ELECTRIC GENERATING PLANT, UNITS 1, 2, 3, AND 4, ISSUANCE OF AMENDMENTS TO RELOCATE

STAFF QUALIFICATION REQUIREMENTS (EPID L-2025-LLA-0010)

DATED AUGUST 12, 2025

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NRR-058

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NAME	JLamb	KZeleznock (KEntz for)	SMehta	KKavanagh
DATE	06/19/2025	7/29/2025	07/25/2025	07/27/2025
OFFICE	NRR/DRO/IOLB/BC	NRR/DRA/ARCB/BC	NRR/DORL/LPL2-1/BC	NRR/DORL/LPL2-1/PM
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DATE	07/25/2025	07/25/2025	08/12/2025	08/12/2025

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