UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
245 PEACHTREE CENTER AVENUE N.E., SUITE 1200
ATLANTA, GEORGIA 30303-1257

July 9, 2025

Delson Erb
Vice President, OPS Support
Tennessee Valley Authority
1101 Market Street, LP 4A-C
Chattanooga, TN 37402-2801

SUBJECT: SEQUOYAH NUCLEAR PLANT – BIENNIAL PROBLEM IDENTIFICATION AND RESOLUTION INSPECTION REPORT 05000327/2025011 AND 05000328/2025011

Dear Delson Erb:

On June 26, 2025, the U.S. Nuclear Regulatory Commission (NRC) completed a problem identification and resolution inspection at your Sequoyah Nuclear Plant and discussed the results of this inspection with Kevin M. Michael, Site Vice President, and other members of your staff. The results of this inspection are documented in the enclosed report.

The NRC inspection team reviewed the station's problem identification and resolution program to confirm that the station was complying with NRC regulations and licensee standards. Based on the samples reviewed, the team determined that your program complies with NRC regulations and applicable industry standards such that the Reactor Oversight process can continue to be implemented.

The team also evaluated the station's effectiveness in identifying, prioritizing, evaluating, and correcting problems, reviewed licensee audits and self-assessments, and its use of industry and NRC operating experience information. The results of these evaluations are in the enclosure.

Finally, the team reviewed the station's programs to establish and maintain a safety-conscious work environment and interviewed station personnel to evaluate the effectiveness of these programs. Based on the team's observations and the results of these interviews, the team found no evidence of challenges to your organization's safety-conscious work environment. Your employees appeared willing to raise nuclear safety concerns through at least one of the several means available.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at http://www.nrc.gov/reading-rm/adams.html and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

Signed by Taylor, Ryan
on 07/09/25

Ryan C. Taylor, Chief
Reactor Projects Branch 5
Division of Operating Reactor Safety

Docket Nos. 05000327 and 05000328
License Nos. DPR-77 and DPR-79

Enclosure:
As stated

cc w/ encl: Distribution via LISTSERV

SUBJECT: SEQUOYAH NUCLEAR PLANT – BIENNIAL PROBLEM IDENTIFICATION AND RESOLUTION INSPECTION REPORT 05000327/2025011 AND 05000328/2025011 DATED JULY 09, 2025

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NAME	C. Curran	R. Taylor			
DATE	07/08/25	07/09/25			

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U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Numbers:	05000327 and 05000328
License Numbers:	DPR-77 and DPR-79
Report Numbers:	05000327/2025011 and 05000328/2025011
Enterprise Identifier:	I-2025-011-0027
Licensee:	Tennessee Valley Authority
Facility:	Sequoyah Nuclear Plant
Location:	Soddy Daisy, TN 37379
Inspection Dates:	June 9, 2025 to June 26, 2025
Inspectors:	C. Curran, Resident Inspector V. Furr, Operations Engineer T. Griffin, Project Engineer D. Hardage, Senior Resident Inspector
Approved By:	Ryan C. Taylor, Chief Reactor Projects Branch 5 Division of Operating Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a biennial problem identification and resolution inspection at Sequoyah Nuclear Plant, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to https://www.nrc.gov/reactors/operating/oversight.html for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

OTHER ACTIVITIES – BASELINE

71152B - Problem Identification and Resolution

Biennial Team Inspection (IP Section 03.04) (1 Sample)

(1) The inspectors performed a biennial assessment of the effectiveness of the licensee's Problem Identification and Resolution program, use of operating experience, self-assessments and audits, and safety conscious work environment.

- Problem Identification and Resolution Effectiveness: The inspectors assessed the effectiveness of the licensee's Problem Identification and Resolution program in identifying, prioritizing, evaluating, and correcting problems. The inspectors also conducted a five-year review of the Emergency Raw Cooling Water System. The corrective actions for the following non-cited violations, minor violations, and findings were evaluated as part of the assessment: NCV 2023002-01, 2023002-02, 2023002-03, 2023403-01, 2023403-02, 2023420-01, 2023404-01, 2024002-01, 2024002-02, 2024003-01, 2025010-01, FIN 2023001-01.

- Operating Experience: The inspectors assessed the effectiveness of the licensee's processes for use of operating experience.

- Self-Assessments and Audits: The inspectors assessed the effectiveness of the licensee's identification and correction of problems identified through audits and self-assessments.

- Safety Conscious Work Environment: The inspectors assessed the effectiveness of the station's programs to establish and maintain a safety-conscious work environment.

INSPECTION RESULTS

Assessment	71152B

Sequoyah Biennial PI&R Assessment

1) Corrective Action Program Effectiveness

Problem Identification: Based on a review of the requirements for initiating condition reports as described in licensee procedure NPG-SPP-22.300 Rev. 26, "Corrective Action Program," the inspectors determined that the licensee was effective in identifying problems and entering them into the corrective action program (which includes the work management system), and there was a low threshold for entering issues into the corrective action program. Additionally, site management was actively involved in the corrective action program and focused appropriate attention on significant plant issues.

Problem Prioritization and Evaluation: Based on the review of condition reports, work orders, and work requests, the inspectors determined that problems were prioritized and evaluated in accordance with licensee guidance. The inspectors determined that adequate consideration was given to system or component operability and associated plant risk. The inspectors determined that in general, plant personnel had conducted cause evaluations in compliance with the licensee's corrective action program procedures and cause determinations were appropriate, and considered the significance of the issues being evaluated.

Corrective Actions: Based on a review of corrective action documents, interviews with licensee staff, and verification of completed corrective actions, the inspectors determined that generally, corrective actions were timely, commensurate with the safety significance of the issues, and effective, in that conditions adverse to quality were corrected. The team determined that the licensee was generally effective in developing corrective actions that were appropriately focused to also address the root and contributing causes for significant conditions adverse to quality to preclude repetition. Effectiveness reviews for corrective actions to preclude repetition were sufficient to ensure corrective actions were properly implemented and were effective.

Based on the samples reviewed, the team concluded that the licensee's corrective action program complied with regulatory requirements and self-imposed standards. The licensee's implementation of the corrective action program adequately supported nuclear safety.

2) Operating Experience

The team determined that the station's processes for the use of industry and NRC operating experience information were effective and complied with regulatory requirements and licensee standards. The implementation of these programs adequately supported nuclear safety. The team concluded that operating experience was adequately evaluated for applicability and that appropriate actions were implemented in accordance with applicable procedures.

3) Self-Assessments and Audits

The inspectors determined that the licensee was effective at performing self-assessments and audits to identify issues at a low level, properly evaluated those issues, and resolved them commensurate with their safety significance. The self-assessments and audits were

adequately self-critical and performance-related issues were being appropriately identified. The inspectors verified that action requests were created to document areas for improvement and findings and verified that actions had been completed consistent with those recommendations.

4) Safety-Conscious Work Environment

Based on interviews with plant staff and reviews of the latest safety culture survey results, the team found no evidence of challenges to a safety conscious work environment. Employees interviewed appeared willing to raise nuclear safety concerns through at least one of the several means available. Generally, interviewees were familiar with the employee concerns program (ECP), specifically how to raise issues through the ECP, or who to contact on site.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On June 26, 2025, the inspectors presented the biennial problem identification and resolution inspection results to Kevin M. Michael, Site Vice President, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71152B	Corrective Action Documents	1965821, 1973910, 1915127, 1950819, 1950821, 1968027, 1928476, 1964645, 1969318, 1973921, 1943726, 1855352, 1876057, 1872806, 1915300, 1930877, 1930874, 1897818, 1897921, 1853806, 1864040, 1976695, 1893012, 1893009, 1930877, 1930874, 1881708, 1896718, 1872171, 1966779, 1858347, 1881328, 1855179, 1854992, 1922292, 1906679, 1900849, 1937554, 1925615, 1859724, 1873451, 1924343, 1951074, 1988736, 2003719, 1953128, 1962283, 1821490, 1872165, 1888213, 1896585, 1892443, 1902333, 1947208, 1873471, 1873495, 1872426, 1849888, 1707916, 1931267, 1926614, 1926802, 1926579, 1972326, 1985447, 1837725, 1836250, 1837794, 1839830, 1842083, 1843346, 1843958, 1847451, 1847596, 1847713, 1841555, 1844755, 1848532, 1855352, 1863699, 1863972, 1870646, 1878482, 1853784, 1930672, 1866767, 1884366, 1875450, 1919433, 1922488, 1848840, 1851824, 1852964, 1854211, 1856330, 1866577, 1938885, 1943083, 1944214, 1881632, 1881328, 1883125, 1884181, 1885447, 1885449, 1885521, 1922292, 1930338, 1931818, 1934831, 1949398, 1959662, 1983244, 1990870, 1946974, 1947905, 1985303, 1994839, 1994768, 1839384, 1843763, 1848022, 1852514, 1854565, 1856317, 1854008, 1838110, 1845109, 1845447, 1845599, 1845124, 1845126, 1858734, 1891089, 1898780, 1900954, 1907804, 1933246, 1940102, 1960206, 1961291, 1845294, 1849562, 1913250, 1922242, 1946906, 1947801, 1969022, 1993223, 2003162, 1836250, 1843478, 1843431, 1846695, 1851365, 1923313, 1924343, 1926579, 1854559, 1854992, 1856729, 1873790, 1884366, 1897684, 1927514, 1945062, 1962284, 1973640, 1985440, 1988156, 1987338		
71152B	Engineering Changes	SQN-23-054	Allow Installation of Larger Actuator Stem Nut Washer	Rev. 0003

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
			for the 4" Motor Driven Auxiliary Feedwater - Steam Generator Level Control Valves	
71152B	Miscellaneous	VTD-M120-0220	Vendor Manual for AFW Level Control Valve	
71152B	Procedures	0-TI-OPS-000-911.1	Risk Management Actions for Risk Informed Completion Times	Rev. 0001
71152B	Procedures	NEDP-27	Past Operability Evaluations	Rev. 0008
71152B	Procedures	NPG-SPP-09.11.4	Risk-Managed Technical Specifications Program	Rev. 0002
71152B	Procedures	NPG-SPP-09.11.5	Risk Managed Tech Specs Program Cumulative Risk	Rev. 0000
71152B	Procedures	NPG-SPP-22.300	Corrective Action Program	Rev. 0026
71152B	Procedures	NPG-SPP-22.302	Condition Report Screening	Rev. 0015
71152B	Self-Assessments	1855681, 1858441, 1859853, 1860691, 1861229, 1862418, 1865029, 1867075, 1868781, 1871902, 1871903, 1871906, 1871907, 1871908, 1912037, 1932289, 1947457, 1989047, 1963351, 1672591		
71152B	Work Orders	124024347, 124168298, 123751950, 123609194, 124757624, 124900215, 123904583, 123560734, 124176500, 123199304, 124567710, 124576850,		

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		123816383, 124993206, 124133190, 124133186, 124491725, 124061515, 124522828, 124808850, 123604322, 122746400, 124660618		