

NMP1L3647
June 25, 2025

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555-0001

Nine Mile Point Nuclear Station, Unit 1
Renewed Facility Operating License No. DPR-63
Docket No. 50-220

Subject: Fifth Inservice Inspection Interval, First Inservice Inspection Period
2025 Owner's Activity Report for RFO-28 Inservice Examinations

Pursuant to 10 CFR 50.55a, Nine Mile Point Nuclear Station, LLC (NMPNS) hereby submits the attached Fifth Inservice Inspection Interval, First Inservice Inspection Period, 2025 Owner's Activity Report for RFO-28 Inservice Examinations, for Nine Mile Point Unit 1 (NMP1).

This activity report meets the requirements of the American Society of Mechanical Engineers (ASME), Code Case N-532-5, "Alternative Requirements to Repair and Replacement Documentation Requirements and Inservice Summary Report Preparation and Submission as Required by IWA-4000 and IWA-6000, Section XI, Division 1," as approved for use by Regulatory Guide 1.147, "Inservice Inspection Code Case Acceptability, ASME Section XI, Division 1."

There are no regulatory commitments contained in this letter.

Should you have any questions regarding the information in this submittal, please contact Ryan Gigliotti, Prin Regulatory Engineer at (315) 349-1398.

Sincerely,

Mattessich, Kevin A Digitally signed by Mattessich,
Kevin A
Date: 2025.06.25 14:03:16 -04'00'

Kevin A. Mattessich
Engineering Director, Nine Mile Point Nuclear Station

KM/RG

Attachment: Fifth Inservice Inspection Interval, First Inservice Inspection Period
2025 Owner's Activity Report for RFO-28 Inservice Examinations

cc: Regional Administrator, Region I, USNRC
Project Manager, USNRC
Resident Inspector, USNRC

Attachment

Fifth Inservice Inspection Interval, First Inservice Inspection Period
2025 Owner's Activity Report for RFO-28 Inservice Examinations

Report Number: N1R28

Unit No. _____ Nine Mile Point Nuclear Station, P. O. Box 63, Lycoming, New York 13093
(Name and Address of Plant)

Plant Unit 1 Commercial service date December 26, 1969 Refueling outage no. N1R28

Applicable inspection interval 5th Inservice Inspection Interval August 23, 2019 to August 22, 2029
(1st, 2nd, 3rd, 4th, Other)

Applicable inspection period Second In-service Inspection Period August 23, 2013 to August 22, 2027
(1st, 2nd, 3rd)

Edition and Addenda of Section XI applicable to the inspection plans 2013 Edition

Date and revision of inspection plans ER-NM-330-1001, Rev 5 (January 5, 2024), Rev 4 (February 28, 2023), Rev 3 (September 16, 2022)

Edition and Addenda of Section XI applicable to repair/replacement activities, if different than the inspection plans
N/A (same Code Edition/Addenda)

Code Cases used for inspection and evaluation: N-532-5, N-613-2, N-638-10, N-648-2, N-653-2, N-702, N-716-1, N-740-2
(if applicable)

Remarks None

I certify that (a) the statements made in this report are correct; (b) the examinations and tests meet the Inspection Plan as required by the ASME Code, Section XI; and (c) the repair/replacement activities and evaluations supporting the completion of N1R28 conform to the requirements of the ASME Section XI.
(refueling outage number)

Signed _____ Date _____
(Owner's Designee)

I, the undersigned, holding a valid commission issued by the National Board of Boiler and Pressure Vessel Inspectors and employed by The Hartford Steam Boiler Inspection and Insurance Company of Hartford, CT have inspected the items described in this Owner's Activity Report and state that, to the best of my knowledge and belief, the Owner has performed all activities represented by this report in accordance with the requirements of the ASME Code, Section XI.

By signing this certificate, neither the Inspector nor his employer makes any warranty, expressed or implied, concerning the repair/replacement activities and evaluations described in this report. Furthermore, neither the Inspector nor his employer shall be liable in any manner for any personal injury or property damage or a loss of any kind arising from or connected with this inspection.

(Inspector's Signature) Commissions ANIR 16158 NY 5570

(National Board Number and Endorsement)

Date _____

ITEMS WITH FLAWS OR RELEVANT CONDITIONS THAT REQUIRED EVALUATION FOR CONTINUED SERVICE

EXAMINATION CATEGORY	ITEM NUMBER	ITEM DESCRIPTION	EVALUATION DESCRIPTION
F-A	F1.10D	Jam Nut Missing on 01-HS-01	<p>While performing a VT-3 visual exam of Pipe Support 01-HS-01, it was found that a jam nut was missing. The previous inspection in N1R23 identified loose bolting. Engineering review determined that the functionality of the snubber is not affected as the jam nut is intended as an enhancement to prevent the load nut from loosening, which the inspection confirmed has not happened. It was also determined that the missing nut did not impact other systems or pose an FME risk.</p> <p>IR 04847185</p>
F-A	F1.40C	Reactor Recirc Support 32-15-H1 Springs Out of Tolerance	<p>While performing a VT-3 visual exam of Pipe Support Spring 32-15-H1, three of the four constant support indicators were found to be outside of the 10% cold load setting tolerance. 32-15-H1A was missing a cold load setting mark. It was determined that the springs still had room to travel under varying hot and cold loads and would not be restricted in their movement. The conditions were found to be acceptable per engineering evaluation.</p> <p>IR 04848306</p>
F-A	F1.10C	Support 31-H6 Out of Tolerance	<p>While performing a VT-3 visual exam of Pipe Support Stanchion 31-H6, it was found that the cold load setting was out of tolerance. This condition was previously evaluated under IR 02471275 and since the condition was found to be unchanged, this evaluation was determined to still be valid. This evaluation determined that there is still room for the support to move due to varying temperatures and is therefore acceptable as is.</p> <p>IR 04847454</p>

TABLE 2
ABSTRACT OF REPAIR/REPLACEMENT ACTIVITIES REQUIRED FOR CONTINUED SERVICE

CODE CLASS	ITEM DESCRIPTION	DESCRIPTION OF WORK	DATE COMPLETED	REPAIR/REPLACEMENT PLAN NUMBER
1	32-WD-208 N2E	Full Structural Weld Overlay repair of 32-WD-208 N2E. IR 04565150	4/3/2025	C93904254
3	ESW Pump 12 Test Line	A pin hole leak was found in the discharge of ESW Pump 12. A weld repair was performed to remove the repair the leak. IR 04713705	12/19/2023	C93947457
1	01-HS-01	A Jam Nut was replaced on Pipe Support 01-HS-01 after it was found to be missing during visual examination. IR 04847092, 04847185	3/24/2025	C94012552
MC	Drywell Swing Bolting	8 Drywell Swing Bolts were replaced after they were found to be damaged during their removal. IR 04848821	4/10/2025	C93941208
1	PSV-01-102C	ERV 121 Tailpipe Temperature was observed to be elevated compared to other ERVs. Valve PSV-01-102C was replaced with a refurbished valve, and the Class 1 inlet flange nuts and studs as well as the Class 2 enclosure nuts and studs were replaced. IR 04758340	4/08/2025	C93910173
1	PSV-01-102F	Visual signs of steam cutting were found on ERV 121 and ERV 123 indicating that a steam leak had occurred. Valve PSV-01-102F was replaced with a refurbished valve, and the Class 1 inlet flange nuts and studs as well as the Class 2 enclosure nuts and studs were replaced. IR 04847772	4/8/2025	C93909205
1	RPV-NR02	During reactor disassembly, the bolting for the reactor head vent was cut and required replacing prior to reassembly.	4/21/2025	C93943057
1	Reactor Head "B" Flange on Line 37-2-B	Reactor Head "B" Flange was replaced following observed indications that exceeded the criteria in CC-AA-407 "MAINTENANCE SPECIFICATION: EVALUATION AND REPAIR OF PIPING AND EQUIPMENT FLANGES". IR 04563939	4/17/2025	C93903481
1	Reactor Head Level Instrument "D" Flange	Reactor Head Instrument Level Piping "D" Flange was replaced following the identification of rolled edge indications. IR 04563961	4/16/2025	C93903482

END OF INSERVICE INSPECTION
OWNER'S ACTIVITY REPORT
FOR
REFUELING OUTAGE TWENTY EIGHT (RFO-28)