



**UNITED STATES
NUCLEAR REGULATORY COMMISSION**

REGION I
475 ALLENDALE RD, STE 102
KING OF PRUSSIA, PENNSYLVANIA 19406-1415

June 17, 2025

David P. Rhoades
Senior Vice President
Constellation Energy Generation, LLC
President and Chief Nuclear Officer (CNO)
Constellation Nuclear
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT – DESIGN BASIS
ASSURANCE INSPECTION (PROGRAMS) – AGE-RELATED DEGRADATION
INSPECTION REPORT 05000333/2025040

Dear David Rhoades:

On May 15, 2025, the U.S. Nuclear Regulatory Commission (NRC) completed an inspection at James A. Fitzpatrick Nuclear Power Plant and discussed the results of this inspection with Garrick Olson, Plant General Manager, and other members of your staff.

No findings or violations of more than minor significance were identified during this inspection.

This letter, its enclosure, and your response (if any) will be made available for public inspection and copying at <http://www.nrc.gov/reading-rm/adams.html> and at the NRC Public Document Room in accordance with Title 10 of the *Code of Federal Regulations* 2.390, "Public Inspections, Exemptions, Requests for Withholding."

Sincerely,

Erin E. Carfang, Chief
Engineering Branch 1
Division of Operating Reactor Safety

Docket No. 05000333
License No. DPR-59

Enclosure:
As stated

cc w/ encl: Distribution via LISTSERV

SUBJECT: JAMES A. FITZPATRICK NUCLEAR POWER PLANT – DESIGN BASIS
 ASSURANCE INSPECTION (PROGRAMS) – AGE-RELATED DEGRADATION
 INSPECTION REPORT 05000333/2025040 DATED JUNE 17, 2025

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U.S. NUCLEAR REGULATORY COMMISSION
Inspection Report

Docket Number: 05000333

License Number: DPR-59

Report Number: 05000333/2025040

Enterprise Identifier: I-2025-040-0000

Licensee: Constellation Nuclear

Facility: James A. Fitzpatrick Nuclear Power Plant

Location: Oswego, NY

Inspection Dates: April 28, 2025 to May 15, 2025

Inspectors: P. Cataldo, Senior Reactor Inspector
A. Patel, Senior Reactor Inspector
G. Stock, Reactor Operations Engineer

Approved By: Erin E. Carfang, Chief
Engineering Branch 1
Division of Operating Reactor Safety

Enclosure

SUMMARY

The U.S. Nuclear Regulatory Commission (NRC) continued monitoring the licensee's performance by conducting a design basis assurance inspection (programs) inspection at James A. Fitzpatrick Nuclear Power Plant, in accordance with the Reactor Oversight Process. The Reactor Oversight Process is the NRC's program for overseeing the safe operation of commercial nuclear power reactors. Refer to <https://www.nrc.gov/reactors/operating/oversight.html> for more information.

List of Findings and Violations

No findings or violations of more than minor significance were identified.

Additional Tracking Items

None.

INSPECTION SCOPES

Inspections were conducted using the appropriate portions of the inspection procedures (IPs) in effect at the beginning of the inspection unless otherwise noted. Currently approved IPs with their attached revision histories are located on the public website at <http://www.nrc.gov/reading-rm/doc-collections/insp-manual/inspection-procedure/index.html>. Samples were declared complete when the IP requirements most appropriate to the inspection activity were met consistent with Inspection Manual Chapter (IMC) 2515, "Light-Water Reactor Inspection Program - Operations Phase." The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel to assess licensee performance and compliance with Commission rules and regulations, license conditions, site procedures, and standards.

REACTOR SAFETY

71111.21N.04 - Age-Related Degradation

Age-Related Degradation (12 Samples)

The inspectors selected a sample of components and structures to verify that engineering performance and maintenance activities addressed age-related degradation and that problems were appropriately identified, addressed, and corrected.

Samples were selected considering risk insights and the potential for degradation. Specifically, the inspectors selected samples that were risk-significant and either had indications of age-related degradation or were exposed to environments that could cause age-related degradation. The inspectors considered site-specific and industry-wide operating experience to identify samples and included both active and passive sub-components within each component or structure.

For each selected sample listed below, the inspectors reviewed the licensee's engineering and maintenance activities credited to address age-related degradation to assess whether they were being performed using applicable standards and procedures, evaluated against appropriate acceptance criteria, and that maintenance and condition monitoring was completed at an appropriate interval. The inspectors also assessed whether issues identified during licensee activities were being entered into and addressed by the applicable station processes including the corrective action program, and whether periodic assessments of maintenance effectiveness and operating experience were being completed, including feedback and process adjustments.

- (1) Emergency diesel generator fuel oil storage tank 93TK-6A
- (2) Condensate drain cooler 33DC-7A
- (3) Fire protection piping
- (4) Reactor core isolation cooling pump 13P-1
- (5) Low pressure coolant injection pump power supplies
- (6) 'C' Residual heat removal pump motor and cables
- (7) Low pressure coolant injection valves (10MOV-25A&B) and associated logic relays
- (8) 'B' Residual heat removal pump and motor (10P-3B)
- (9) 'D' Emergency diesel generator 93EDG-D
- (10) 'B' Core spray pump 14P-1B
- (11) Automatic depressurization system valve 02RV-5 (RV-71E)
- (12) 'A' Residual heat removal service water pump 10P-1A

INSPECTION RESULTS

No findings were identified.

EXIT MEETINGS AND DEBRIEFS

The inspectors verified no proprietary information was retained or documented in this report.

- On May 15, 2025, the inspectors presented the design basis assurance inspection (programs) results to Garrick Olson, Plant General Manager, and other members of the licensee staff.

DOCUMENTS REVIEWED

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
71111.21N.04	Corrective Action Documents	AR 04802093	J1R26 FAC/EPC Non-Compliance	09/16/2024
	Corrective Action Documents Resulting from Inspection	IR 4861447	Oil level below standstill line	
		IR 4861509	ARDI walkdown inspection identified issues	
		IR 4861529	Fire door 76FDR-DG-272-3 bracket loose	
		IR 4861529	Fire door 76FDR-DG-272-3 bracket loose	1
		IR 4861541	93TK-6A remaining service life evaluation needed	
		IR 4864389	Crack in reactor building east crescent	
		IR 4865806	ARDI generate WO to paint 4-WF-151-133	
		IR 4865807	Structures monitoring report lacking detail	
		IR 4865828	Model WO for cable testing not comprehensive	
		IR 4866029	ST-76KA performance	
	Drawings	ESK-6MP	Elementary Diagram 600V CKTS-MOV RHR Inboard VV's 10MOV-25A&B	23
		FE-32C	Manhole and Duct Line Detail	16
	Engineering Changes	EC 633292	76-4"-WF-151-133 Minimum Wall Thickness Evaluation	000
		EC 635673	Install Enhanced Safety Motor Guards to Fully Enclose Rotating Equipment on Several SR Components	0
		EC 638245	Emergency Diesel Generator A, B, C, D Circulating Lube Oil Pump Mount Reconfiguration	001
		EC 641494	93TK-6A Bulge Tech Eval (Ref. IR 04772624)	000
	Miscellaneous		Constellation Preventative Maintenance Control Relay Template	
	Procedures	ER-AA-200-1001	Equipment Classification	8
		ISP-8-2B	Core Spray Pump Low Pressure and Low Flow Bypass Valve Instrument Calibration	007
		PMID 00344088	PM - Calibrate EDG Relay Tachometer Loop	1
		ST-22C	ADS Logic System Functional Test	50
		ST-76KB	Integrity Inspection of FP CO2 System and SGT Filter Tran B FP System	09/14/2023
	Work Orders	05116995-01	PM – Perform Eddy Current Inspection on 33DC-7A	09/08/2024

Inspection Procedure	Type	Designation	Description or Title	Revision or Date
		05417087	LR - Review Service Water Piping per ER-AA-5400	