NRC FORM 374	U.S. NUCLEAR REGULATORY COMMISSION	Page 1 of 17

MATERIALS LICENSE

Pursuant to the Atomic Energy Act of 1954, as amended, the Energy Reorganization Act of 1974 (Public Law 93-438), and Title 10, Code of Federal Regulations, Chapter I, Parts 30, 31, 32, 33, 34, 35, 36, 39, 40, and 70, and in reliance on statements and representations heretofore made by the licensee, a license is hereby issued authorizing the licensee to receive, acquire, possess, and transfer byproduct, source, and special nuclear material designated below; to use such material for the purpose(s) and at the place(s) designated below; to deliver or transfer such material to persons authorized to receive it in accordance with the regulations of the applicable Part(s). This license shall be deemed to contain the conditions specified in Section 183 of the Atomic Energy Act of 1954, as amended, and is subject to all applicable rules, regulations, and orders of the U.S. Nuclear Regulatory Commission now or hereafter in effect and to any conditions specified below.

Licensee	
1. Louisiana Energy Services, LLC	3. License Number: SNM-2010, Amendment 107
2. 275 Highway 176	4. Expiration Date: June 9, 2040
P.O. Box 1789	5. Docket No. 70-3103
Eunice, New Mexico 88231	

6. Source and/or Special Nuclear Material and/or Byproduct Material	7. Chemical and/or Physical Form	8. Maximum amount that licensee may possess at any one time under this license
A. Uranium (natural and depleted) and daughter products	A.1 Physical: Solid, Liquid A.2 Chemical: UF_6, UF_4, UO_2F_2, oxides and other compounds	A. 251,000,000 kg
B. Uranium enriched in isotope U-235 less than 10 percent by weight (wt. percent) and uranium daughters, subject to the following additional constraints below:	B.1 Physical: Solid, Liquid, and Gas B.2 Chemical: UF_6, UF_4, UO_2F_2, oxides, metal and other compounds	B. [SEE SENSITIVE CONDITIONS]
B.1 Grams of U-235 contained in uranium enriched to 10 wt. percent or more but less than or equal to 10.8 wt. percent in the U-235 isotope subject to the following additional constraints below:	B.1 Physical: Solid, Liquid, and Gas B.2 Chemical: UF_6, UF_4, UO_2F_2, oxides, metal and other compounds	B.1 Less than or equal to 10,000 grams U-235 (to be included in License Condition 8B total)

This license contains **SENSITIVE SECURITY - RELATED INFORMATION**. Upon removal of the Sensitive Conditions on Pages 14 - 17, this license is **DECONTROLLED**.

Enclosure 3

NRC FORM 374A U.S. NUCLEAR REGULATORY COMMISSION	Page 2 of 17
MATERIALS LICENSE **SUPPLEMENTARY SHEET**	License Number SNM-2010
	Docket or Reference Number 70-3103
	Amendment 107

Constraints for License Conditions 6B and 6B.1

URENCO USA facility (UUSA) shall not input parameters into the plant control system (PCS) to produce material for the assay above 5.0 wt. percent limit until the U.S. Nuclear Regulatory Commission (NRC) has completed an operational readiness review (ORR) to verify the necessary changes have been implemented and the facility will be operated safely and in accordance with the requirements of the license. UUSA shall notify the NRC for scheduling the ORR at least 60 days prior to the planned production. UUSA shall not produce product material at or above 10 wt. percent U-235 other than in the course of cascade performance adjustments, thus providing the operational flexibility to generate material to satisfactorily fulfill customer orders up to the less than 10 wt. percent U-235 limit. UUSA shall not at any time input parameters into the PCS to produce material for the assay at or above 10 wt. percent U-235.

C. Tc-99, transuranic isotopes and other contamination	C. Any	C. Amount that exists as contamination as a consequence of the historical feed of recycled Uranium at other facilities
D. Deleted	D. Deleted	D. Deleted
E. Deleted	E. Deleted	E. Deleted
F. Deleted	F. Deleted	F. Deleted
G. Co-60	G. Sealed per §30.32(g)(1)	G. [SEE SENSITIVE CONDITIONS]
H. Deleted	H. Deleted	H. Delete
I. Deleted	I. Deleted	I. Deleted
J. Deleted	J. Deleted	J. Deleted
K. Sr-90	K. Sealed per §30.32(g)(1)	K. [SEE SENSITIVE CONDITIONS]
L. Deleted	L. Deleted	L. Deleted
M. Deleted	M. Deleted	M. Deleted
N. Deleted	N. Deleted	N. Deleted
O. Deleted	O. Deleted	O. Deleted
P. Deleted	P. Deleted	P. Deleted
Q. Cs-137	Q. Sealed per §30.32(g)(1)	Q. [SEE SENSITIVE CONDITIONS]

NRC FORM 374A U.S. NUCLEAR REGULATORY COMMISSION	Page 3 of 17
MATERIALS LICENSE **SUPPLEMENTARY SHEET**	License Number SNM-2010
	Docket or Reference Number 70-3103
	Amendment 107

R. Deleted	R. Deleted	R. Deleted
S. Po-210	S. Sealed per §30.32(g)(1)	S. [SEE SENSITIVE CONDITIONS]
T. Th-230	T. Sealed per §30.32(g)(1)	T. [SEE SENSITIVE CONDITIONS]
U. U-232	U. Sealed per §30.32(g)(1)	U. [SEE SENSITIVE CONDITIONS]
V. U-233	V. Sealed per §30.32(g)(1)	V. [SEE SENSITIVE CONDITIONS]
W. U-234	W. Sealed per §30.32(g)(1)	W. [SEE SENSITIVE CONDITIONS]
X. U-235	X. Sealed per §30.32(g)(1)	X. [SEE SENSITIVE CONDITIONS]
Y. U-236	Y. Sealed per §30.32(g)(1)	Y. [SEE SENSITIVE CONDITIONS]
Z. U-238	Z. Sealed per §30.32(g)(1)	Z. [SEE SENSITIVE CONDITIONS]
AA. Am-241	AA. Sealed per §30.32(g)(1)	AA. [SEE SENSITIVE CONDITIONS]
BB. Cf-252	BB. Sealed per §30.32(g)(1)	BB. [SEE SENSITIVE CONDITIONS]
CC. Cs-139	CC. Sealed per §30.32(g)(1)	CC. [SEE SENSITIVE CONDITIONS]
DD. Co-60	DD. Unsealed per §30.32(i)(1)(ii)	DD. [SEE SENSITIVE CONDITIONS]
EE. Sr-90	EE. Unsealed per §30.32(i)(1)(ii)	EE. [SEE SENSITIVE CONDITIONS]
FF. Cs-137	FF. Unsealed per §30.32(i)(1)(ii)	FF. [SEE SENSITIVE CONDITIONS]
GG. Po-210	GG. Unsealed per §30.32(i)(1)(ii)	GG. [SEE SENSITIVE CONDITIONS]
HH. Th-230	HH. Unsealed per §30.32(i)(1)(ii)	HH. [SEE SENSITIVE CONDITIONS]
II. U-232	II. Unsealed per §30.32(i)(1)(ii)	II. [SEE SENSITIVE CONDITIONS]
JJ. U-233	JJ. Unsealed per §30.32(i)(1)(ii)	JJ. [SEE SENSITIVE CONDITIONS]
KK. U-234	KK. Unsealed per §30.32(i)(1)(ii)	KK. [SEE SENSITIVE CONDITIONS]
LL. U-235	LL. Unsealed per §30.32(i)(1)(ii)	LL. [SEE SENSITIVE CONDITIONS]
MM. U-236	MM. Unsealed per §30.32(i)(1)(ii)	MM. [SEE SENSITIVE CONDITIONS]
NN. U-238	NN. Unsealed per §30.32(i)(1)(ii)	NN. [SEE SENSITIVE CONDITIONS]

NRC FORM 374A U.S. NUCLEAR REGULATORY COMMISSION	Page 4 of 17
MATERIALS LICENSE **SUPPLEMENTARY SHEET**	License Number SNM-2010
	Docket or Reference Number 70-3103
	Amendment 107

OO. Am-241	OO. Unsealed per §30.32(i)(1)(ii)	OO. [SEE SENSITIVE CONDITIONS]
PP. Cs-139	PP. Unsealed per §30.32(i)(1)(ii)	PP. [SEE SENSITIVE CONDITIONS]
QQ. Eu-152	QQ. Sealed per §30.32(g)(1)	QQ. [SEE SENSITIVE CONDITIONS]
RR. Ba-133	RR. Sealed per §30.32(g)(1)	RR. [SEE SENSITIVE CONDITIONS]
SS. TC-99	SS. Sealed per §30.32(g)(1)	SS. [SEE SENSITIVE CONDITIONS]

9.A Authorized place of use: UUSA, located 5 miles east of Eunice, New Mexico, on Highway 176 in Lea County, New Mexico.

9.B This license shall be deemed to contain two Sections: Domestic Safety and Safeguards Conditions and International Safeguards Conditions. These sections are part of the license, and the licensee is subject to compliance with all listed conditions in each section.

FOR THE U.S. NUCLEAR REGULATORY COMMISSION

Date: See digital signature

By: Signed by Lav, Samantha on 04/22/25

Samantha Lav, Chief
Fuel Facility Licensing Branch
Division of Fuel Management
Office of Nuclear Material Safety
and Safeguards

NRC FORM 374A U.S. NUCLEAR REGULATORY COMMISSION	Page 5 of 17
MATERIALS LICENSE **SUPPLEMENTARY SHEET**	License Number SNM-2010
	Docket or Reference Number 70-3103
	Amendment 107

SECTION 1.0 – DOMESTIC SAFETY AND SAFEGUARDS

10. The licensee shall conduct authorized activities in accordance with the statements, representations, and conditions in the documents listed below, or as revised in accordance with Section 19 of the Quality Assurance Program Description, Title 10 of the *Code of Federal Regulations* (10 CFR) Paragraph 40.35(f), 10 CFR Section 51.22, 10 CFR Section 70.32, 10 CFR Section 70.72, or 10 CFR Section 95.19 in:

    a. Application for Material License, NRC Form 313 dated December 12, 2003.
    b. Safety Analysis Report (SAR) dated December 12, 2003, as revised by letters dated February 27, 2004; July 30, 2004; September 30, 2004; April 22, 2005; April 29, 2005; May 25, 2005; June 10, 2005; February 16, 2006; February 28, 2006; March 16, 2006; March 24, 2006; January 29, 2007; April 10, 2007; July 30, 2007; October 12, 2007; October 19, 2007; November 2, 2007; November 12, 2007; November 30, 2007; February 28, 2008; November 19, 2008; January 23, 2009; March 5, 2009; September 24, 2009; November 25, 2009; January 29, 2010; March 31, 2010; May 2, 2010; May 16, 2010; May 23, 2010; May 25, 2010; May 26, 2010; June 2, 2010; June 3, 2010; June 23, 2010; July 16, 2010; March 22, 2011; March 29, 2011; April 11, 2011; September 19, 2011; September 22, 2011; June 25, 2019; September 8, 2020 (Agencywide Documents Access and Management System (ADAMS) ML20262H070); August 3, 2021 (ML21217A141); December 10, 2021 (ML21349A142); April 27, 2022 (ML22122A059), and August 3, 2023 (ML23220A389).
    c. Deleted per Amendment 70.
    d. Physical Security Plan dated December 12, 2003, as revised by letters dated May 12, 2004; July 30, 2004; December 10, 2004; January 12, 2005; February 12, 2008; August 11, 2008; May 1, 2009; July 16, 2009; February 5, 2010; September 20, 2010; October 2, 2015; August 10, 2021 (ML21236A084), and July 12, 2023 (ML23220A047).
    e. Fundamental Nuclear Material Control Plan dated December 12, 2003, as revised by letters dated February 27, 2004; July 30, 2004; October 7, 2004; December 7, 2004; April 22, 2005; October 23, 2006; October 19, 2007; November 30, 2007; September 4, 2009; September 24, 2009; January 13, 2010; January 14, 2010; June 30, 2010; November 12, 2010; April 7, 2011; May 3, 2011; June 1, 2011; June 28, 2011, March 13, 2012, July 9, 2012 (ML12194A047), November 15, 2012 (ML12325A068), and December 5, 2012 (ML12346A249).
    f. Quality Assurance Program Description dated April 9, 2004, as revised by letter dated April 22, 2005; October 23, 2006; November 12, 2007; July 30, 2007; October 12, 2007; October 19, 2007; November 12, 2007; July 31, 2008; January 21, 2009; March 2, 2009; March 5, 2009; September 24, 2009; November 25, 2009; January 29, 2010; March 31, 2010; June 23, 2010; July 16, 2010; October 1, 2010; December 10, 2010; December 16, 2010; June 21, 2011; August 17, 2011; September 19, 2011; October 15, 2013; November 28, 2018; June 4, 2020; September 8, 2020 (ML20262H070); March 9, 2021; June 7, 2021; October 28, 2021 (ML21307A319); November 2, 2021 (ML21309A475); and November 11, 2021 (ML21335A226). Exception to this license condition is granted for the Cylinder Receipt and Dispatch Building superstructure/footers, as amended by correspondence dated October 14, 2011.
    g. Emergency Plan dated December 12, 2003, as revised by letters dated July 30, 2004; September 30, 2004; April 22, 2005; October 23, 2006; July 30, 2007; October 19, 2007; November 2, 2007; March 10, 2008; September 4, 2008; September 30, 2008; February 19, 2009;

NRC FORM 374A U.S. NUCLEAR REGULATORY COMMISSION	Page 6 of 17
MATERIALS LICENSE **SUPPLEMENTARY SHEET**	License Number SNM-2010
	Docket or Reference Number 70-3103
	Amendment 107

March 5, 2009; April 16, 2009; September 24, 2009; November 25, 2009; January 29, 2010; March 31, 2010; June 21, 2011; August 17, 2011; and September 19, 2011.

h. Standard Practice Procedure Plan (SPPP) for the Protection of Classified Matter dated December 12, 2003, as revised by correspondence dated July 30, 2004; March 16, 2006; November 21, 2006; November 22, 2006; March 20, 2007; April 27, 2007; July 19, 2007; October 12, 2007; November 30, 2007; February 4, 2008; May 1, 2008; May 7, 2008; June 26, 2008; July 7, 2008; August 4, 2008; September 4, 2008; September 5, 2008; October 6, 2008; October 16, 2008; November 20, 2008; November 25, 2008; February 12, 2009; March 2, 2009; December 29, 2009; May 25, 2010; July 15, 2010; October 22, 2010; February 11, 2011; March 8, 2011; April 19, 2011; May 20, 2011; May 25, 2011; June 29, 2011; July 18, 2011; December 21, 2011; October 7, 2015; February 29, 2016; November 7, 2018; May 12, 2023 (ML23132A331); December 17, 2024 (ML24352A488); and February 7, 2025 (ML25040A002).
   Constraint to License Condition 10.h
   Prior to implementation of the updated classified boundary as detailed by the December 17, 2024, and February 7, 2025 submittals, NRC staff must complete an ORR and UUSA must receive a final inspection report from Region II.

i. Deleted per Amendment 83.

j. Deleted per Amendment 64.

k. Movement Plan for Transportation of Classified Centrifuge Components/Materials between Tripartite Countries and the U.S. dated February 26, 2008, as revised by letter dated December 8, 2017; June 24, 2023 (ML23177A054); and June 12, 2024 (ML24165A275).

l. Deleted per Amendment 80.

m. Deleted.

n. Deleted Per Amendment 75.

o. Deleted per Amendment 64.

p. Deleted per Amendment 64.

q. Deleted per Amendment 102.

r. Deleted per Amendment 81.

s. Deleted per Amendment 64.

t. Deleted per Amendment 64.

u. Deleted per Amendment 64.

v. Deleted per Amendment 64.

w. Deleted per Amendment 64.

x. Deleted per Amendment 102.

y. The licensee is granted a waiver to the U.S. Department of Energy (DOE) National Security System Manual, paragraph EN-10 of DOE M 205.1-4, for external labeling of junction boxes for Louisiana Energy Services (LES) Information System Security Plan (ISSP) 1.0. The conditions to which this waiver applies are covered under the SPPP for Protection of Classified Matter.

z. Deleted per Amendment 82.

aa. Deleted per Amendment 82.

ab. Deleted per Amendment 103.

ac. Deleted per Amendment 89.

ad. Deleted per Amendment 103.

ae. Deleted per Amendment 103.

af. Decommissioning Financial Assurance submitted by letters dated December 12, 2019 (ML19351C772); October 12, 2020 (ML20296A196); April 1, 2021 (ML21109A186); December 15, 2021 (ML22063A481); December 13, 2022 (ML22354A126); and December 14, 2023 (ML23348A271).

ag. Standard Practice Procedure Plan for the Protection of Classified Matter at the ETC USA Location at the site of Urenco USA, Revision 0a, dated May 12, 2023 (ML23132A335).

ah. ETC USA Information System Security Plan (ISSP) for the Centrifuge Assembly Building Classified Network dated August 7, 2023 (ML23220A008), as revised by letter dated December 18, 2023 (ML23353A061).

ai. Urenco USA Facility, Classified Systems Information System Security Plan (UUSA-ISSP-Classified Systems), dated December 26, 2023 (ML23360A771).

11. Introduction of UF_6 into any module of the National Enrichment Facility (NEF) shall not occur until the Commission completes an operational readiness and management measures verification review to verify that management measures that ensure compliance with the performance requirements of 10 CFR Section 70.61 have been implemented and confirms that the facility has been constructed and will be operated safely and in accordance with the requirements of the license. The licensee shall provide the Commission with 120 days advance notice of its plan to introduce UF_6 in any module of the NEF.

12. The licensee is hereby granted the special authorizations and exemptions identified in Section 1.2.5 of the UUSA Safety Evaluation Report, dated June 2013.

13. This license was originally issued for a period of 30 years from the date of license issuance. Based on an amendment request dated May 6, 2011, the license expiration date has been extended to June 9, 2040.

14. For the disposition of depleted UF_6 (DUF_6) the licensee shall not use a DUF_6 de-conversion facility that employs a process that results in the production of anhydrous hydrofluoric acid.

15. The licensee shall maintain a minimum of $200 million in liability insurance coverage.

16. a & b Deleted

c. The licensee shall provide an updated Decommissioning Funding Plan, including a decommissioning cost estimate update and final copies of the proposed financial assurance instruments, to the NRC for review at least 6 months prior to initiating operations in the next cascade in an Assay Unit within any subsequent Separation Building Module (SBM) or licensed material into CRDB-2. Final executed copies of the reviewed financial assurance instruments, along with a revised certification of financial assurance reflecting the updated amount of the financial assurance instruments, shall be provided to NRC at least 21 days prior to initiating operations in the next cascade. For the first cascade in a new SBM, the Decommissioning Funding Plan update must provide for full funding for decontamination and decommissioning of the SBM structure, first cascade, and the common equipment of the new Assay Unit. The amount of the

financial assurance instrument shall be updated to current year dollars and include any applicable changes to the decommissioning cost estimate.

d. Updated decommissioning cost estimates for depleted uranium disposition shall be provided triennially on a forward-looking basis to reflect annual projections of depleted uranium byproduct generation. The depleted uranium disposition cost estimate shall include an update to the DOE depleted uranium disposition cost estimate. Funding instruments shall be provided annually, and the total amount funded for depleted uranium disposition shall be no less than the updated DOE cost estimate.

17. Deleted

18. Deleted

19. To define the boundaries of each item relied on for safety (IROFS), the licensee shall utilize its procedure, "IROFS Boundary Definitions." Completed IROFS boundaries for all IROFS shall be available for inspection at the time of the ORR.

20. [SEE SENSITIVE CONDITIONS]

21. Onsite storage of DUF_6 generated at the NEF shall be limited to a maximum of 25,000 48Y cylinders (or the equivalent amount of uranium stored in other NRC accepted and Department of Transportation ["DOT"] certified cylinder types) of DUF_6. The generation of any additional DUF_6 to be stored onsite by the licensee beyond this limit shall constitute noncompliance with the license. The licensee shall suspend production of any additional DUF_6 for onsite storage until this noncompliance is remedied. In no event shall the licensee store DUF_6 generated at the NEF in New Mexico other than at the NEF.

22. Onsite storage of any one cylinder of DUF_6 generated at the NEF shall be limited to a maximum of 25 years, beginning from the date that each cylinder is filled in accordance with the licensee's standard procedures. The storage of any one DUF_6 cylinder beyond this limit by the licensee shall constitute noncompliance with the license. The licensee shall suspend production of any additional DUF_6 for onsite storage until this noncompliance is remedied. In no event shall the licensee store DUF_6 generated at the NEF in New Mexico other than at the NEF.

23. The licensee shall provide financial assurance for the offsite disposal of DUF_6 from the NEF using a minimum contingency factor of 25 percent.

    Upon reaching 24,000 cylinders of DUF_6 in 48Y cylinders (or the equivalent amount of uranium stored in other NRC accepted and DOT certified cylinder types) in onsite storage, the licensee shall immediately increase the financial assurance to provide a 50 percent contingency factor for disposition of DUF_6 stored at the NEF unless: (a) an application to construct and operate a de-conversion facility outside of New Mexico that is specifically designated to de-convert the DUF_6 stored onsite at the NEF has been docketed by the agency responsible for reviewing the application; (b) an application for such a facility

has been approved by the agency responsible for reviewing the application; or (c) the licensee is using another alternate method for removing the DUF_6 stored onsite.

In addition, upon reaching the limit of 25,000 cylinders of DUF_6 in 48Y cylinders (or the equivalent amount of uranium stored in other NRC accepted and DOT certified cylinder types) in onsite storage, the licensee shall immediately increase the financial assurance to provide 50 percent contingency factor for disposition of DUF_6 stored at the NEF if the contingency factor has not already been increased to 50 percent. The contingency factor shall remain at 50 percent until the number of cylinders stored onsite is reduced to 98 percent of the 25,000 cylinder limit and either: (a) an application to construct and operate a de-conversion facility outside of New Mexico that is specifically designated to de-convert the DUF_6 stored onsite at the NEF has been docketed by the agency responsible for reviewing the application; (b) an application for such a facility has been approved by the agency responsible for reviewing the application; or (c) the licensee is using another alternate method for removing the DUF_6 from New Mexico. Nothing herein shall release the licensee from other financial assurance obligations set forth in applicable laws and regulations.

24. The licensee shall maintain and follow the Fundamental Nuclear Material Control Program for control and accounting and measurement control of uranium source material and special nuclear material at the NEF pursuant to 10 CFR Paragraph 74.33(b). The licensee shall make no change to material control procedures essential for the safeguarding of uranium source material or special nuclear material that would decrease the effectiveness of the material control and accounting program implemented pursuant to 10 CFR Paragraph 74.33(b) without prior approval of the Commission. If the licensee desires to make changes that would decrease the effectiveness of its material control and accounting program or its measurement control program, the licensee shall submit an application for amendment to its license pursuant to 10 CFR Section 70.34.

    The licensee shall maintain records of changes to the material control and accounting program made without prior Commission approval a period of five years from the date of the change. The licensee shall furnish to the Director, Division of Nuclear Security, Office Nuclear Security and Incident Response, using an appropriate method listed in 10 CFR 70.5(a), a report containing a description of each change within six months of the change if it pertains to uranium enriched less than 20 percent in the uranium-235 isotope.

25. If there are any revisions to the nuclear criticality safety validation report, then the licensee shall provide a letter to the NRC describing the changes and shall provide the revised validation report upon request. The licensee may not implement the changes in the revised validation report until the NRC approves the changes.

26. Deleted per Amendment 102

27. [SEE SENSITIVE CONDITIONS]

NRC FORM 374A U.S. NUCLEAR REGULATORY COMMISSION	Page 10 of 17
MATERIALS LICENSE **SUPPLEMENTARY SHEET**	License Number SNM-2010
	Docket or Reference Number 70-3103
	Amendment 107

28. The Licensee is exempted from the definitions of a commercial grade item, "basic component," "critical characteristics," "dedicating entity," and "dedication" in 10 CFR Section 21.3, as replaced by the following:

Commercial grade item: A commercial grade item means a structure, system, or component, or part thereof that affects its IROFS function that was not designed and manufactured as a basic component. Commercial grade items do not include items where the design and manufacturing process require in-process inspections and verifications to ensure that defects or failures to comply are identified and corrected (i.e., one or more critical characteristics of the item cannot be verified).

Basic component: A basic component means a structure, system, or component, or part thereof that affects their IROFS function, that is directly procured by the licensee or activity subject to the regulations in Part 70 and in which a defect or failure to comply with any applicable regulation in this chapter, order, or license issued by the Commission would create a substantial safety hazard (i.e., exceed performance requirements of 10 CFR Section 70.61). In all cases, basic components include IROFS-related design, analysis, inspection, testing, fabrication, replacement parts, or consulting services that are associated with the component hardware whether these services are performed by the component supplier or others.

When applied to fire protection systems procured for facilities and other activities licensed under 10 CFR Part 70 of the chapter, basic component means a structure, system, or component, or part thereof, that affects their safety function, in which a defect or failure to comply with any applicable regulation in this chapter, order, or license issued by the Commission could create a substantial safety hazard. For fire protection systems designated as items relied on for safety, a basic component may be directly procured from a commercial entity by a Part 70 licensee if: (1) the system, structure or component is manufactured to an established, acceptable national code or standard that includes some independent product endorsements based on qualification testing or periodic testing of selected characteristics of the component; and (2) the acceptability of the item's manufacture, testing, and/or certification has been reviewed and verified by the licensee prior to use as a basic component. Once the acceptability of the item has been verified by LES and the item has been designated for use as a basic component, the licensee accepts responsibility for Part 21 reporting.

Critical characteristics: Critical characteristics are those important design, material, and performance characteristics of a commercial grade item that, once verified, will provide reasonable assurance that the item will perform its intended IROFS function.

Dedication: Dedication is an acceptance process undertaken to provide reasonable assurance that a commercial grade item to be used as a basic component will perform its intended IROFS function and, in this respect, is deemed equivalent to an item designed and manufactured under a 10 CFR Part 50, Appendix B, Quality Assurance Program. This assurance is achieved by identifying the critical characteristics of the item and verifying their acceptability by inspections, tests, or analyses performed by the purchaser or third-party dedicating entity after delivery, supplemented as necessary by one or more of the following: commercial grade surveys, product inspections or witness at holdpoints at the manufacturer's facility, and analysis of historical records for acceptable performance. In all cases, the

dedication process must be conducted in accordance with the applicable provisions of 10 CFR Part 50, Appendix B. The process is considered complete when the item is designated for use as a basic component.

Dedicating entity: Dedicating entity means the organization that performs the dedication process. Dedication may be performed by the manufacturer of the item, a third-party dedicating entity, or the licensee itself. The dedicating entity, pursuant to Section 21.21(c) of this part, is responsible for identifying and evaluating deviations, reporting defects and failure to comply for the dedicated item, and maintaining auditable records of the dedication process. In cases where the licensee applies the commercial grade item procurement strategy and performs the dedication process, the licensee would assume full responsibility as the dedicating entity.

Prior to implementing the above commercial grade procurement strategy and dedication process, the licensee shall submit a license amendment request to the NRC for approval amending its Quality Assurance Program Description to include its commitments described in its exemption request submittals dated November 21, 2008, and November 24, 2008.

29. Deleted.

30. The licensee is granted the special authorization as requested in correspondence dated May 24, 2012, (LES-12-00074-NRC) (ML12150A372). Specifically:

    a) The licensee shall not make changes to the SAR, without prior NRC approval unless the criteria in paragraph b are satisfied. For changes requiring prior NRC approval, the licensee shall submit to the NRC, for review and approval, an application to amend the license. Such changes shall not be implemented until approval is granted unless prior written Authorization is provided by the NRC.

    b) Upon documented completion of a change request for a facility or process, the licensee may make changes in the facility or process as presented in the SAR, or conduct tests or activities not presented in the SAR that would normally be described therein, without prior NRC approval, subject to the following conditions:

        1. There is no decrease in the level of effectiveness of the design basis for safety functions as described in the SAR, and;
        2. The change does not result in a departure from a method of evaluation described in the SAR used in establishing the design bases for safety functions, and;
        3. The change does not result in a decrease in effectiveness of safety commitments as described in the SAR, and;
        4. The change does not affect compliance with applicable regulatory safety requirements, and;
        5. The change does not conflict with any condition specifically stated in LES Materials License SNM-2010.

    Changes to the SAR shall be evaluated, documented and reported in accordance with the commitments in Enclosure 1 of correspondence dated May 24, 2012 (LES-12-00074-NRC) (ML12150A372). Records

of such changes shall be maintained, including technical justification and management approval, in dedicated records to enable NRC inspection upon request at the facility. A periodic report containing a description of each such change, and appropriate revised sections to the license application, shall be submitted to the NRC every six months.

31. Deleted per Amendment 102

32. Notwithstanding the requirements for labeling of containers in 10 CFR Paragraph 20.1904(a), UF_6 feed, tails, and product cylinders (48Y, 30B, and 30B-10) do not require labeling. Waste containers and contaminated equipment staged in Restricted Areas may be simply labeled as “caution - radioactive material” or the equivalent.

33. The licensee is exempted from the specific requirement in 10 CFR Paragraph 70.24(a) requiring gamma or neutron sensitive radiation detectors in the existing Cascade Halls and proposed future Cascade Halls that are designed consistent with the halls analyzed in LAR 13-01. This exemption is also granted for Outdoor 30B Cylinder Storage Areas as discussed in LAR 17-03; however, this exemption is limited to only filled 30B product cylinders containing less than or equal to 5 wt. percent U-235 for the Outdoor Cylinder Storage Areas.

34. Deleted per Amendment 102

35. Deleted per Amendment 102

36 The Redundant IROFS Class, as defined in the SAR, will only be applicable to and implemented on equipment used to support Administrative Control IROFS.

37 All IROFS are required to adhere to Section 16 of the latest version of the Quality Assurance Program Description identified by License Condition 10(f).

38 If equipment is required for IROFS applied to multiple accidents scenarios that include quality assurance level QL-2R and QL-2AC, the higher category of quality assurance level QL-2AC will apply.

39 LES shall submit an amendment request to the NRC prior to making changes to the elements of the quality assurance level QL-2R, that would further reduce or alter the quality assurance level.

40 Notwithstanding the requirements of 10 CFR 70.50(b)(1), the licensee is exempted from the requirement to report unplanned contamination events when the following conditions are met:

1. The event occurs in a restricted area in a building which is maintained inaccessible to the public by multiple access controls,
2. The area was controlled as a Radioactive Materials Area within a building before the event occurred, the release of radioactive material is contained within the Radioactive Materials Area, and no contamination has spread outside the Radioactive Materials Area,

3. Radiation Protection Task Qualified Individuals/Operators trained in contamination control are readily available,
4. Equipment and facilities that may be needed for contamination control are readily available, and
5. The otherwise reportable unplanned contamination event is documented in the licensee's Corrective Action Program.

41. 2 months prior to possession of special nuclear material of moderate strategic significance, 2. will establish, maintain, and implement a maintenance, testing and calibration program to ensure that security systems and equipment are tested for operability and performance at predetermined intervals, maintained in operable condition, and are capable of performing their intended functions.

42. With the exception of the Gaseous Effluent Vent System, there shall be no processing of U-235 enriched to less than 10 wt. percent or components exposed to U-235 enriched to less than 10 wt. percent in any installed recycling systems or support systems, including the Ventilated Room, Decontamination Workshop, Liquid Effluent Collection and Transfer System Room, or Solid Waste Collection Room. Systems and components that have been exposed to U-235 enriched to less than 10 wt. percent shall be segregated from systems and components that have not been exposed to U-235 enriched to less than 10 wt. percent.

SECTION 2.0 – INTERNATIONAL SAFEGUARDS

SG-2.1 [SEE SENSITIVE CONDITIONS]

SG-2.2 [SEE SENSITIVE CONDITIONS]

SG-2.3 [SEE SENSITIVE CONDITIONS]

SG-2.4 [SEE SENSITIVE CONDITIONS]

SG-2.5 Deleted per Amendment 107

SG-2.6 UUSA is authorized to exchange foreign obligations between similar materials as needed for routine licensed activities as described in letter dated August 20, 2024 (ML24233A310).