From: Matthews, Shaun
To: Hoc, HOO X

Subject: [External\_Sender] Attn: Bill Nytko EN FORM 361 Seabrook GE Hitachi Relay.pdf

Date:Thursday, November 14, 2024 11:14:27 AMAttachments:EN FORM 361 Seabrook GE Hitachi Relay.pdf

Attn: Bill Nytko

## PART 21 - CONTROL RELAY DEFECT

NextEra Energy Seabrook LLC. makes the following notification under 10 CFR 21.21(d)(3)(i) of a defect found in a GE - Hitachi Relay, CR120B (Model #DD945E118P0060) during pre-installation bench testing. During bench testing, the relay failed to energize and transfer all associated contacts. The relay was purchased from GE - Hitachi (GEH) as QL-1, safety-related, GE CR-120B relays. All GE CR-120B relays that were purchased in the same batch as the failed relay were located and quarantined in order to be returned to GEH for forensic testing. NextEra Energy Seabrook, LLC has concluded that this defect constitutes a Substantial Safety Hazard (SSH). An SSH exists because the nature of the defect was such that, if installed in certain safety-related applications and failed, it would have prevented the fulfillment of a safety function. On November 12, 2024, the Seabrook site Vice President was notified of the requirement to report this event under 10 CFR 21.21. This is a non-emergency notification required by 10 CFR 21.21(d)(3)(i), a written notification will be provided in accordance with 10 CFR 21.21(d)(3)(ii).

Because the defect was discovered prior to installation, there was no impact to safety-related equipment.

The NRC SRI has been informed.

Page 1 of 1
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NRC FORM 361 (08-15-2024)

## U.S. NUCLEAR REGULATORY COMMISSION **OPERATIONS CENTER**



## **REACTOR PLANT EVENT NOTIFICATION WORKSHEET**

## APPROVED BY OMB: NO. 3150-0238

EXPIRES: 07/31/2026

Estimated burden per response to comply with this voluntary collection request: 30 minutes. The information provided will be used for evaluation of licensee event description, facility status and for input to the public website. Send comments regarding burden estimate to the FOIA, Library, and Information Collections Branch (T-6 A10M), U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001, or by email to Infocollects.Resource@nrc.gov, and and the OMB reviewer at: OMB Office of Information and Regulatory Affairs, (3150-0238), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street NW, Washington, DC 20503. The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.

″**** <sup>*</sup>								EN#	5	7424	
	[3rd] 301-415-0553	. ‡Licensee			5100 or 800-532-3469‡, B n their own ETS are provid					49-3694‡,	
Notification Time	Facility or Organia	zation	Unit	it Name of Caller/Title					Call Back #	ŧ	
1058 EST	NEXTERA ENE SEABROOK, L	ERGY	1	1 Shaun Matthews/Eng			ng An	alyst	305	-394-7454	
Event Time & Zone	Event Date		Power/	Mod	e (At Time of Event)		Powe	er/Mode (At	Time of Noti	fication)	
0738 EDT	10/01/20	)24			99/1				100/1		
EVENT C	LASSIFICATION		1-HR. N	10N-	EMERGENCY 10 CFR 50.	.72(b)(1)		v)(A) Safe	S/D Capability		AINA
GENERAL EMERGEN	CY	GEN/AAE	С	T	S Deviation	ADEV		(v)(B) RHR	Capability		AINB
SITE AREA EMERGEN	NCY	SIT/AAE(	C 4-HR. N	ION-	EMERGENCY 10 CFR 50	.72(b)(2)		(v)(C) Contr	ol of Rad Rele	ase	AINC
ALERT		ALE/AAE	C [ (i)	T	S Required S/D	ASHU		(v)(D) Accid	ent Mitigation		AIND
UNUSUAL EVENT UNU/AAEC				)(A) E	ECCS Discharge to RCS	ACCS		(xii) Offsite	e Medical		AMED
50.72 NON-EMERGENCY (see next columns)				)(B) F	RPS Actuation (scram)	ARPS		(xiii) Loss	Comm/Asmt/R	lesponse	ACOM
10 CFR 73 D???				) C	Offsite Notification	APRE	6	0-DAY OP	FIONAL 10 C	CFR 50.73(a)	(1)
MATERIAL/EXPOSURE B???				ION-	EMERGENCY 10 CFR 50	.72(b)(3)		Invalid Spe	ecified System	Actuation	AINV
FITNESS FOR DUTY		HFI <sup>-</sup>	T (ii)	(A) [	Degraded Condition	ADEG	отн	ER UNSPEC	IFIED REQUIF	REMENT (IDEN	ITIFY)
✓ OTHER UNSPECIFIED	D REQMT. (se	ee last column	1) [ii)(	(B) l	Unanalyzed Condition	AUNA	<b>√</b>	10 CFR 21.2	1(d)(3)(i)		NONR
INFORMATION ONLY		NNF	F (iv)	)(A) {	Specified System Actuation	AESF					NONR
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NOTIFICATIONS	YES NO	WILL BE					· - //		\ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \ \		
NRC RESIDENT	<b>V</b>		Anything	) Unu	isual or not understood?		Yes (	Explain abo	ve) [✓] No	· 	
STATE(s)			Did all ev	ıstan	ns function as required?	<b>V</b>	Yes		□ N/	<b>o</b> (Explain abo	ove)
LOCAL			Did all sy	rsten	ns function as required:		162			у (Ехріаін авс	ove)
OTHER GOV AGENCIES			Mode of	opera	ations until corrected (if a	applicable	e)		1		
MEDIA/PRESS RELEASE			Addition	al Inf	ormation continued on n	ext page	?	Yes	✓ No		

							Pa	ge _	of	
NRC FORM 361 (08-15-2024) REACTOR PLANT EVENT NOTIFICATION WORKSHEET (Continued)  U.S. NUCLEAR REGULATORY COMMISSION OPERATIONS CENTER EN # 57424										
RADIOLOGICAL RELEASES: CHECK OR FILL IN APPLICABLE ITEMS (specific details/explanations should be covered in event description)										
Liquid Release	Gaseous Releas	se Unplanned	d Release	P	Planned Release		Ongoing		Terminated	
Monitored	Unmonitored	Offsite Rel	lease	П	.S. Exceeded	[	RM Alarms		Areas Evacuated	
Personnel Expo	sed or Contaminated	Offsite Pro	tection Act	tions Recomn	nended	*5	State release path	in de	escription	
	Release Rate (Ci/sec)	% T.S. Limit	ноо	) Guide	Total Activity	(Ci)	) % T.S. Limit		HOO Guide	
Noble Gas			0.1	Ci/sec					1000 Ci	
lodine			10 μ	ıCi/sec					0.01 Ci	
Particulate			1 μ(	Ci/sec					1 mCi	
<b>Liquid</b> (excluding tritium and dissolved noble gas)			10 μCi/min						0.1 Ci	
Liquid (tritium)			0.2	Ci/min					5 Ci	
TOTAL										
	Plant Stack	Plant Stack Condenser/Air Ejec		tor Main Steam Line		SG Blowdown			Other	
RAD Monitor Readings										
Alarm Setpoints										
% T.S. Limit (If applicable)										
RCS OR SG T	UBE LEAKS: CHECK O	R FILL IN APPLICABL	E ITEMS:	: (specific de	tails/explanation	s sho	uld be covered in	even	t description)	
Location of the Leak	(e.g., SG #, valve, pipe,									
Leak Rate	Units: gpm/gpd	: gpm/gpd T. S. Limits		Sudden or Long-Term Development						
Leak Start Date	Time	Primary	ary Secondary							
List of Safety Relate	d Equipment not Opera	tional								
Event Description (Incl	ude: Systems affected, actua	tions and their initiating sig	nals, cause	effect of eve	ent on plant, action	s take	n or planned, etc.) ((	Contin	nued from Page 1)	