

# Concrete harvesting activities in the Netherlands

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# Introduction

The presentation covers **tests** (mainly mechanical) performed on **concrete cores** taken from structures of the **High Flux Reactor (HFR)** at Petten, Netherlands.

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HFR

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Reactor pool structure

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Concrete cores

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Determination of mechanical properties

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Ageing degradation due to chloride ingress

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Way forward

- Long-
- Term
- Operation



### IRRADIATION OF MATERIALS

Structural materials age during reactor operation due to neutron irradiation at elevated temperature and interaction with the environment. The change of materials properties due to ageing is measured by testing of irradiated specimens.



### COMPONENT INTEGRITY

The integrity of reactor components during the operating period of the reactor is assessed by simulations. Key inputs to the simulations are irradiated material properties and reactor conditions under normal and accident conditions.



### LTO ASSESSMENT

Ageing of structures and components is managed by analyses and plant programmes, demonstrating safe long-term operation of nuclear power plants.

## MODELLING OF COMPONENTS AND AGEING MECHANISMS



### CHARACTERISATION OF IRRADIATED MATERIALS

- Irradiation of materials in High Flux Reactor
- Transport of irradiated specimens from nuclear reactors
- Irradiated materials testing
- Data analysis & creation of databases



### LTO ASSESSMENT

- Review of plant programmes for LTO
- Ageing management review for LTO
- Revalidation of time limited ageing analyses



# About the High Flux Reactor (HFR) in Petten

- Licensed ability of **50 MW**.
- **Light water** cooled and moderated.
- **Tank-in-pool** type.
- Constructed during **1950s**
- Operational since **1961**.
- Present age of the reactor is **63 years**

Nuclear medicines  
(30000 patients per day)

Beam tube  
research

Irradiation  
services

Material  
testing





# Site of the HFR



# Site characteristics

## Coastal

- Industrial site
- North Holland dunes

## Seismic

- PGA 0.1g
- MRI 10000 years

## Wind

- 29.8 m/s (1 hour average)
- 40.6 m/s (gust)

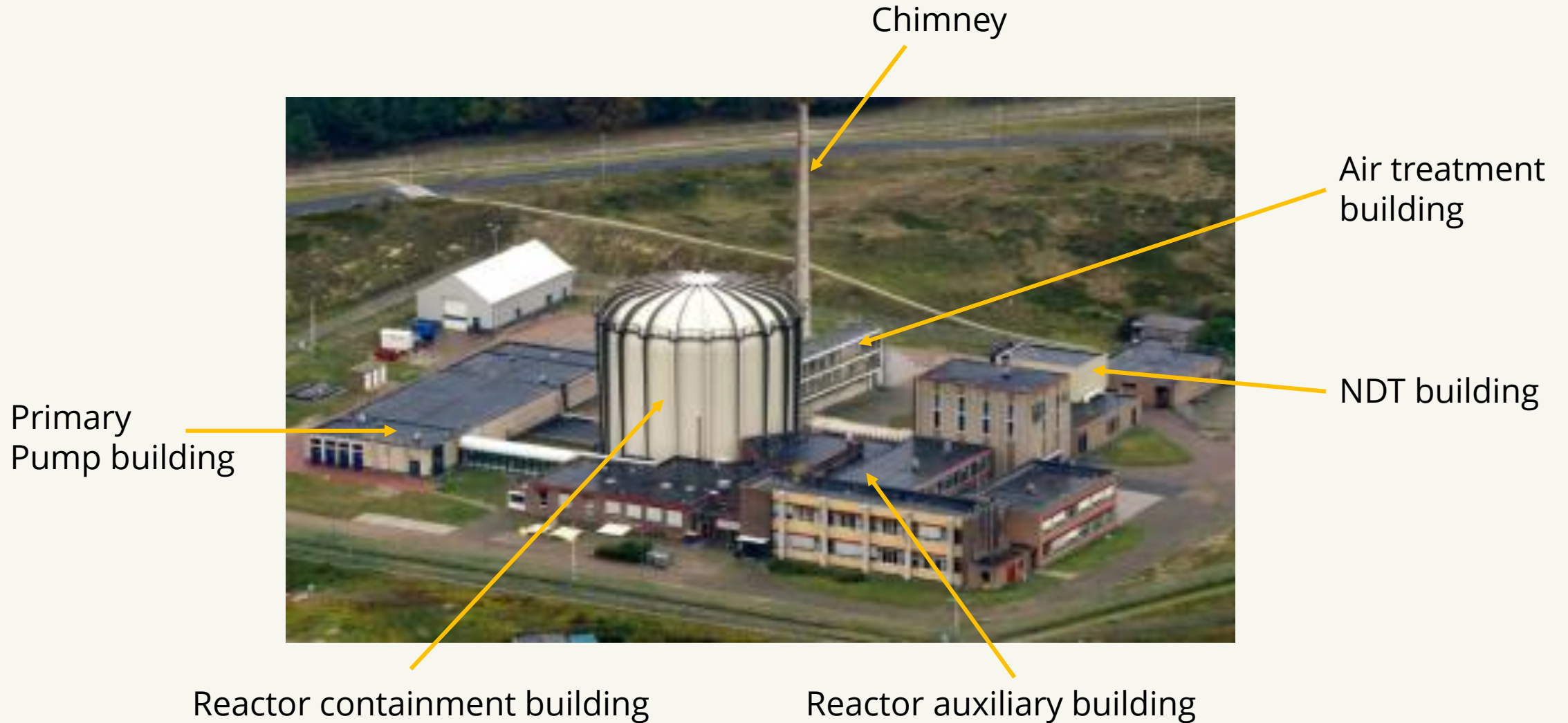
## Precipitation

- 43.5 mm (1 hour)
- 90.2 mm (24 hours)

## Temperature

- 34.8 C highest
- -20.0 C lowest

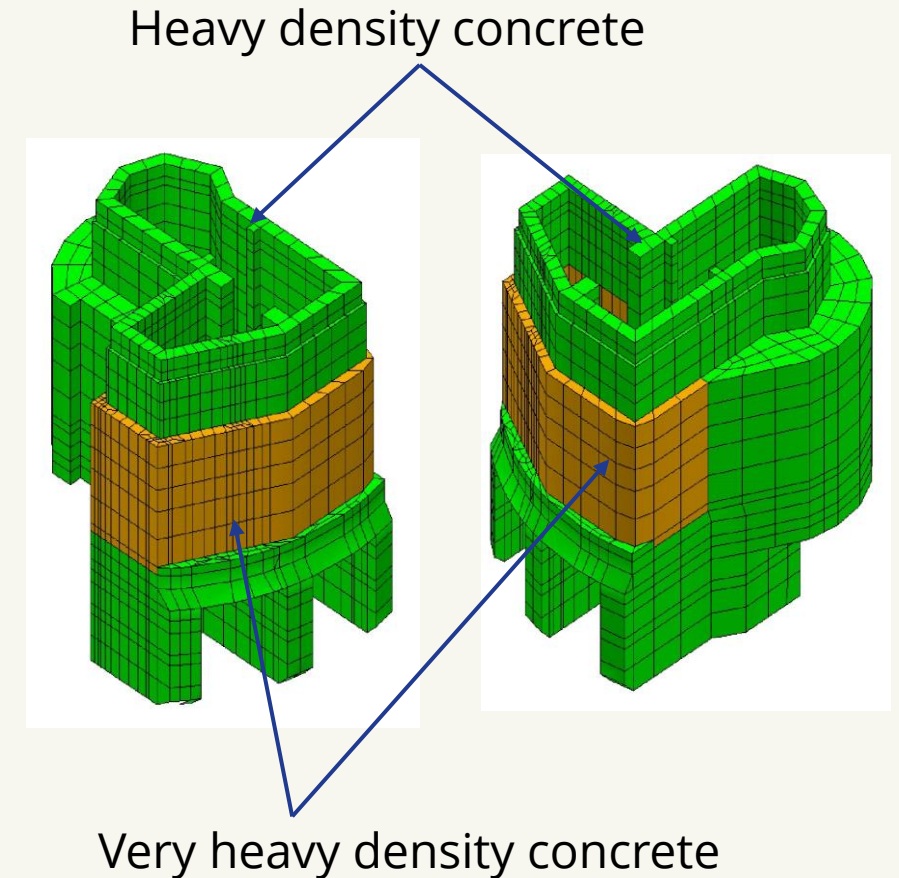
# Main plant layout of HFR





# Reactor pool structure

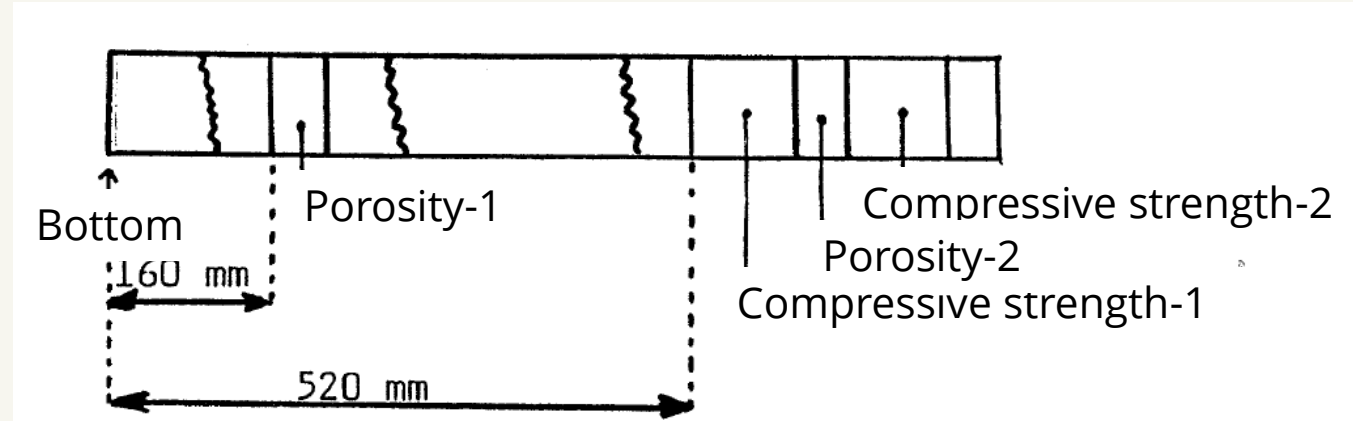
- **Reactor pool** and **storage pools-1 & 2** are housed inside the reactor containment building.
- **Heavy density** concrete ( $3500 \text{ kg/m}^3$ ) and **very heavy density** concrete ( $4200 \text{ kg/m}^3$ ) is used for the basins.
- **Aluminium liner** used for leak tightness.
- Outside the basins, **normal density concrete** was used.
- Concrete class is **C20/25** ( $20.7 \text{ N/mm}^2$ ).





# 1985: Concrete core

- **1985:** A concrete core was drilled out from the underside of the basin.



Material property	Test No.	Value	Unit
Cylinder compressive strength	1	35.7	MPa
	2	34.0	
Density	1	3840	kg/m <sup>3</sup>
	2	3930	
Porosity	1	14.4	%
	2	12.3	

# 2009: Concrete cores

- **2009:** Two concrete cores were drilled out from the ceiling of subpile room.

Material property	Test No.	Value	Unit
Cylinder compressive strength	1	32.3	MPa
	2	36.5	
Density	1	3490	kg/m <sup>3</sup>
	2	3740	
Porosity	1	12.3	%
	2	14.3	

- Based on its mass, appearance and magnetism, it was suspected that the aggregate used is **Greigite** and not **Barite, Magnetite and Hematite** were was previously assumed.



# 2010: Concrete cores

- **2010:** A vertical drilling was carried out over approximately 1.9 m of the total 2.235 m thickness of reactor floor.
- The floor was drilled from the bottom in **4 steps (cores)**.
- **Ten cylinders** were sawn off from the four drilled cores from the reactor floor.
- **Six** cylinders were used to determine the **tensile** strength.
- **Four** cylinders were used to determine the **compressive** strength and volumetric weight.
- The large range of compressive strength is given by the **voids present in the concrete**.



# 2010: Concrete cores

Property	Value	Unit
Split tensile strength	1.99	N/mm <sup>2</sup>
	2.73	
	3.22	
	3.72	
	2.90	
	3.48	
Average Split Tensile Strength	3.01	N/mm <sup>2</sup>
Design value concrete tensile strength	0.99	N/mm <sup>2</sup>
Volumetric weight	3709	kg/m <sup>3</sup>
	3825	
	3548	
	3607	
Cylinder compressive strength	27.3	N/mm <sup>2</sup>
	34.2	
	45.8	
	44.6	



# 2017: Concrete cores

- 2017: Six cores were used from pipe corridor (two from walls and four from ceilings).

Property	Unit	Value (walls)	Value (ceiling)
Volumetric weight	kg/m <sup>3</sup>	3620 3560	3320
			3640
			3390
			3540
Cylinder compressive strength	N/mm <sup>2</sup>	24.9 20.7	17.5
			50.5
			54.6
			62.6



# Statistics of measured mechanical properties

Material property	Parameter	Value	Unit	Measurements
Density	$\mu$	3630	kg/m <sup>3</sup>	12
	$\sigma$	190	kg/m <sup>3</sup>	
Porosity	$\mu$	13.33	%	4
	$\sigma$	1.18	%	
Cylinder compressive strength	$\mu$	39.63	MPa	12
	$\sigma$	12.28	MPa	
Splitting tensile strength	$\mu$	3.01	MPa	6
	$\sigma$	0.62	MPa	
Modulus of elasticity		32310	MPa	1

# Equivalent strength class

- Equivalent **strength class** for each material property:

Material property	Value	Unit	Strength class
Cylinder compressive strength	20	MPa	C20/25
Splitting tensile strength	2.06	MPa	C18/22.5
Modulus of elasticity	32.8	GPa	C30/37

- There is a **visible difference** between the corresponding strength class of each material property.
- In order to generalize the concrete material properties of low-density heavy concrete and ordinary reinforced concrete civil structures with **one unique class strength**, C20/25 was assumed for design.
- The choice of the strength class is **very conservative**, and the same strength class is taken into consideration for the high-density heavy concrete as well.

# Ageing degradation due to chloride ingress

- High concentration of chlorides in concrete lead to corrosion of rebars.
- A **chloride ingress model** is being developed based on the in-situ properties of concrete.
- The current model has been **calibrated** using data from the **chimney** using:
  - Density of cores
  - Chloride concentration with depth
  - w/c ratio (using compressive strength)
- This model can be incorporated in the **FE analysis** to simulate aged concrete with active corrosion.



# Chloride ingress model for chimney

- **Ingress of chloride ions** was evaluated based on **Fick's law** of chloride diffusion where critical time ( $t_{crit}$ ) or the time when the chloride reaches the depth of the reinforcement was estimated using:

$$t_{crit} = \frac{x_{rebar}^2}{4 \cdot D_a \cdot (erfc^{-1}(C_{crit}/C_s))^2} = 21.14 \text{ years}$$

- **Corrosion rate** was determined based on model developed by **Vu and Stewart** (2000),

$$i_{corr(t_p)} = \left( \frac{[3.78(1 - w/c)^{-1.64}]}{cover} \right) \times 0.85 \times t_p^{-0.29} = 0.426 \mu A/cm^2$$

- **Rust production** was determined in the form of :
  - Reduced rebar diameter:  $D_{rb} = 24.59 \text{ mm}$
  - Volume of steel consumed per unit length of anodic steel:  $\Delta V_s = 16.12 \text{ mm}^3/mm$
  - Internal pressure developed due to corrosion:  $P = 2.96 \text{ MPa}$
- Due to the **heterogeneous nature** of concrete, it is difficult to determine a deterministic value of the internal pressure that will lead to cover cracking

# Future works

- In-situ material properties of the **very high-density concrete** needs to be determined.
- Further investigations required to determine the real nature of the **aggregates** (magnetite / barite/ greigite).
- **Chloride ingress model** needs to be updated based on more data from the reactor pool structure.
- **Reinforcement coupons** may be extracted to determine the properties of rebars.
- Impact of **Alkali silica reaction (ASR)** needs to be investigated on the reactor pool structure.

# References

- K.M. Browning, L. Hasa, F.H.E. de Haan –de Wilde, *'Development of conservative material properties to account for concrete degradation mechanisms with specific emphasis on rebar corrosion due to chloride ingress'*, PVP2023-105650, Proceedings of the ASME 2023 Pressure Vessels & Piping Conference PVP 2023, Atlanta, Georgia, USA (2023).
- L. Hasa, F.H.E. de Haan –de Wilde, *'An update of the assessment methodology for civil ageing management: Damage development in concrete structures of a reactor due to ageing mechanisms'*, PVP2022-84008, Proceedings of the ASME 2022 Pressure Vessels & Piping Conference PVP 2022, Las Vegas, NV, USA (2022).

Thank you!

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