

NUCLEAR REGULATORY COMMISSION

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[NRC-2011-0087]

RIN 3150-AI96

Non-Power Production or Utilization Facility License Renewal

AGENCY: Nuclear Regulatory Commission.

ACTION: Final rule and guidance; issuance.

SUMMARY: The U.S. Nuclear Regulatory Commission (NRC) is amending its regulations that govern the license renewal process for certain production or utilization facilities. In this final rule, the NRC collectively refers to these facilities as non-power production or utilization facilities (NPUFs). This final rule revises the definitions of “non-power reactor,” “research reactor,” and “testing facility.” This final rule also eliminates license terms for licenses for facilities used for medical therapy or research and development, other than testing facilities; these licenses are issued under the authority of Sections 104a or 104c of the Atomic Energy Act of 1954, as amended (AEA). This final rule defines the license renewal process for licenses issued to testing facilities under the authority of Section 104c of the AEA or commercial or industrial NPUFs (including testing facilities) under the authority of Section 103 of the AEA. This final rule requires all NPUF licensees to submit to the NRC final safety analysis report (FSAR) updates at intervals not to exceed 5 years. In addition, this final rule provides an accident dose criterion of 1 Roentgen equivalent man (rem) (0.01 sievert [Sv]) total

effective dose equivalent (TEDE) for NPUFs other than testing facilities. The NRC is also issuing final implementation guidance for this final rule.

DATES: This final rule is effective on January 29, 2025.

ADDRESSES: Please refer to Docket ID NRC-2011-0087 when contacting the NRC about the availability of information for this action. You may obtain publicly available information related to this action by any of the following methods:

- **Federal Rulemaking Web Site:** Go to <https://www.regulations.gov> and search for Docket ID NRC-2011-0087. Address questions about NRC dockets to Helen Chang; telephone: 301-415-3228; e-mail: Helen.Chang@nrc.gov. For technical questions, contact the individuals listed in the FOR FURTHER INFORMATION CONTACT section of this document.

- **NRC's Agencywide Documents Access and Management System (ADAMS):** You may obtain publicly available documents online in the ADAMS Public Documents collection at <https://www.nrc.gov/reading-rm/adams.html>. To begin the search, select "Begin Web-based ADAMS Search." For problems with ADAMS, please contact the NRC's Public Document Room (PDR) reference staff at 1-800-397-4209, at 301-415-4737, or by e-mail to pdr.resource@nrc.gov. For the convenience of the reader, instructions about obtaining materials referenced in this document are provided in the "Availability of Documents" section.

- **NRC's PDR:** The PDR, where you may examine and order copies of publicly available documents, is open by appointment. To make an appointment to visit the PDR, please send an email to PDR.Resource@nrc.gov or call 1-800-397-4209 or 301-415-4737, between 8 a.m. and 4 p.m. eastern time, Monday through Friday, except Federal holidays.

FOR FURTHER INFORMATION CONTACT: Robert Beall, Office of Nuclear Material Safety and Safeguards, telephone: 301-415-3874, e-mail: Robert.Beall@nrc.gov and Duane Hardesty, Office of Nuclear Reactor Regulation, telephone: 301-415-3724, e-mail: Duane.Hardesty@nrc.gov. Both are staff of the U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001.

SUPPLEMENTARY INFORMATION:

EXECUTIVE SUMMARY:

A. Need for the Regulatory Action

In April 2008, the Commission issued staff requirements memorandum (SRM) M080317B, "Briefing on State of NRC Technical Programs," which directed the staff to "examine the license renewal process for non-power reactors and identify and implement efficiencies to streamline this process while ensuring that adequate protection of public health and safety are maintained." The need for improvement in the reliability and efficiency of the license renewal process was primarily driven by four issues:

1) historic NRC priorities and emergent issues; 2) limited licensee resources; 3) inconsistent existing license infrastructure; and 4) regulatory requirements and the broad scope of the renewal process.

B. Major Provisions

The major provisions of this final rule include changes that:

- Revise the definitions for *Non-power reactor*, *Testing facility*, and *Research reactor*;

- Eliminate license terms for medical therapy or research and development facilities, other than testing facilities, licensed under paragraphs (a) or (c) of § 50.21 of title 10 of the *Code of Federal Regulations* (10 CFR);
- Define the license renewal process for all commercial or industrial NPUFs (including testing facilities) licensed under § 50.22 and testing facilities licensed under § 50.21(c);
- Require all NPUF licensees to submit an updated FSAR and subsequent FSAR updates to the NRC at intervals not to exceed 5 years;
- Amend the current timely renewal provision under § 2.109, allowing NPUFs subject to license renewal to continue operating under an existing license past its expiration date if the licensee submits a license renewal application at least 2 years (rather than 30 days) before the current license expiration date;
- Provide an accident dose criterion of 1 rem (0.01 Sv) TEDE for NPUFs other than testing facilities;
- Extend the applicability of § 50.59 to NPUF licensees regardless of their decommissioning status;
- Clarify an NPUF applicant's requirements for meeting the existing provisions of § 51.45 for submitting an environmental report; and
- Eliminate the requirement for NPUF licensees to submit financial qualification information with license renewal applications under § 50.33(f)(2).

Concurrent with this final rule, the NRC is issuing Regulatory Guide (RG) 2.7, Revision 0, "Preparation of Updated Final Safety Analysis Reports for Non-Power Production or Utilization Facilities."

C. Costs and Benefits

The NRC prepared a regulatory analysis to determine the expected quantitative costs and benefits of this final rule and the final implementing guidance, as well as qualitative factors to be considered in the NRC's rulemaking decision. Based on the analysis, the NRC concluded that this final rule will result in net savings to licensees and the NRC. The analysis examined the benefits and costs of the final rule requirements and the final implementing guidance compared to the baseline for the current license renewal process (i.e., the no-action alternative). Compared to the no-action baseline, the NRC estimates that total net benefits to NPUFs (i.e., cost savings minus costs) will be \$5.5 million (\$3.9 million using a 3-percent discount rate or \$2.6 million using a 7-percent discount rate) over a 20-year period. The average NPUF will receive net benefits ranging from approximately \$78,000 to \$166,000 over a 20-year period. The NRC will receive total net benefits of \$12 million (\$8.6 million using a 3-percent discount rate or \$5.9 million using a 7-percent discount rate) over a 20-year period.

The regulatory analysis also considered, in a qualitative fashion, additional benefits of this final rule and the final implementing guidance associated with regulatory efficiency, protection of public health and safety, promotion of the common defense and security, and protection of the environment.

The regulatory analysis concluded that this final rule and the final implementing guidance are justified because of the cost savings received by both licensees and the NRC while public health and safety are maintained. A detailed discussion of the methodology and complete results is presented in the "Regulatory Analysis" section of this document.

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I. Background

The NRC licenses NPUFs under the authority granted in Sections 103 and 104 of the AEA. Section 103 of the AEA applies to commercial and industrial facilities, and Sections 104a and 104c of the AEA apply to facilities used for medical therapy or research and development activities, respectively. The section of the AEA that provides licensing authority for the NRC corresponds directly to the class of license issued to a facility (e.g., Section 104a of the AEA authorizes the issuance of a “class 104a” license). Furthermore, Sections 104a and 104c of the AEA require that the Commission impose the minimum amount of regulation needed to promote the common defense and security; protect the health and safety of the public; and permit, under Section 104a, the widest amount of effective medical therapy possible and, under Section 104c, the conduct of widespread and diverse research and development.

The NRC regulates 34 NPUFs, of which 29 are research reactors or testing facilities currently licensed to operate. The NRC has issued construction permits for two of the five remaining NPUFs (SHINE Medical Technologies, Inc. (SHINE) and the

Hermes-Kairos Testing facility),¹ and the other three licensees are in the process of decommissioning their facilities (i.e., removing a facility or site safely from service and reducing residual radioactivity to a level that permits release of the site for unrestricted use or use under restricted conditions). Most NPUFs are located at universities or colleges throughout the United States. The NRC regulates one operating testing facility at the National Institute of Standards and Technology.

A. *License Terms*

The AEA dictates an initial license term of no more than 40 years for class 103 facilities, which the NRC licenses under § 50.22, but the AEA does not specify license terms for class 104a or 104c facilities, which are licensed under § 50.21(a) or (c). The regulation that implements this statutory authority, § 50.51(a), currently specifies that the NRC may grant an initial license for NPUFs for no longer than a 40-year license term. If the NRC initially issues a license for a shorter period, then it may renew the license by amendment for a maximum aggregate period not to exceed 40 years. An NPUF license is usually renewed for a term of 20 years. If the requested renewal would extend the license beyond 40 years from the date of issuance, the original license may not be renewed by amendment. Rather, the NRC must issue a renewed license that supersedes the initial license.

Any application for license renewal must include an FSAR describing:

1) changes to the facility or facility operations resulting from new or amended regulatory requirements, and 2) changes and effects of changes to the facility or procedures and new experiments. The FSAR must include the elements specified in § 50.34. The NRC has guidance for preparing the FSAR in NUREG-1537, Part 1, "Guidelines for Preparing

¹ On May 18, 2018, the NRC issued a construction permit for Northwest Medical Isotopes, LLC. That construction permit was terminated on July 11, 2022.

and Reviewing Applications for the Licensing of Non-Power Reactors: Format and Content.” The NRC reviews NPUF initial and renewal license applications using NUREG-1537, Part 2, “Guidelines for Preparing and Reviewing Applications for the Licensing of Non-Power Reactors: Standard Review Plan and Acceptance Criteria.”

As a license term nears its end, a licensee must submit a license renewal application to continue operations. A “timely renewal” provision exists in § 2.109(a) to enable operations to continue beyond the license term during the NRC’s review of a license renewal application. If the licensee files an application for a renewal or for a new license for the authorized activity at least 30 days before the expiration of an existing license, the existing license will not be deemed to have expired until the application has been finally determined.

B. Need for Improvement in the License Renewal Process

In 2008, the NRC recognized a need to identify and implement efficiencies in the NPUF license renewal process while ensuring that adequate protection of public health and safety is maintained. Four issues primarily drove this effort to improve the reliability and efficiency of the process.

1. Historic NRC Priorities and Emergent Issues

Under the Atomic Energy Commission (AEC), the NRC’s predecessor agency, NPUFs were some of the first reactors licensed and the first reactors to undergo license renewal. Most of these reactors were initially licensed in the late 1950s and 1960s for terms that varied from 10 to 40 years. The AEC started renewing these licenses in the 1960s. License renewal was primarily an administrative activity until 1976, when the NRC decided to also conduct a technical review equivalent to the initial licensing of the facility. The licenses that had been issued with initial 20-year terms were due for renewal during this timeframe. As the NRC started developing methods for conducting

these technical reviews, an accident occurred at Unit 2 of the Three Mile Island (TMI) nuclear power plant.

The NRC's focus on post-TMI activities resulted in a suspension of NPUF license renewal activities for several years. After license renewal activities were reinitiated, the NRC issued numerous renewals in a short period of time, primarily by relying on generic evaluations. These 20-year renewals expired starting in the late 1990s. The original licenses issued with 40-year terms also started expiring in the late 1990s, creating a new surge of license renewal applications.

As a result of the NRC's response to the events of September 11, 2001, the NRC deferred work on a number of NPUF license renewal applications. In addition, the NRC's NPUF licensing activities focused on implementing § 50.64, "Limitations on the use of highly enriched uranium (HEU) in domestic non-power reactors," to convert non-power reactors to the use of low-enriched uranium. Therefore, reviews of these license renewal applications extended for many years. In all cases, the timely renewal provision enabled these NPUFs to continue operating during the NRC's review period.

2. Limited Licensee Resources

Many NPUF licensees have limited staff resources available for licensing support. The number of NPUF staff can range from one part-time employee for some low-power facilities to four or five full-time employees for higher-power facilities. The NPUF staff that perform the licensing function typically do so in addition to their normal organizational responsibilities, which often results in delays in the license renewal process, particularly in responding to the NRC's requests for additional information.

3. Inconsistent Existing License Infrastructure

The NPUFs licensed under § 50.21(a) or (c) are primarily at college and university sites. Staff turnover and limited staffing resources at an NPUF often contribute to a lack of historical knowledge of the development of the licensee's FSAR

and changes to the FSAR. During the most recent round of license renewals, the NRC found that some of the submitted FSARs did not adequately reflect the current licensing bases for the respective licensees. Because the only required FSAR submission comes at license renewal, which can be at 20-year or greater intervals, submitted FSARs often contain varying levels of completeness and accuracy. Consequently, the NRC has issued requests for additional information to obtain missing information, seek clarifications and corrections, and document the current licensing basis.

4. Regulatory Requirements and Broad Scope of the Renewal Process

For power reactors, license renewal reviews have a defined scope, primarily focused on aging management, as described in 10 CFR part 54. For NPUFs, there are no explicit requirements on the scope of issues to be addressed during license renewal. Therefore, the scope of review for license renewal was initially treated the same as that for an original license.

In response to Commission direction in SRM-SECY-91-061, "Separation of Non-Reactor and Non-Power Reactor Licensing Activities from Power Reactor Licensing Activities in 10 CFR Part 50," the NRC developed licensing guidance for the first time since many NPUF applicants were originally licensed. In that guidance (NUREG-1537, Parts 1 and 2), the NRC provides detailed descriptions of the scope, content, and format of FSARs and the NRC's process for reviewing initial license applications and license renewal applications. However, the first license renewals using NUREG-1537 had varying levels of consistency and did not propose an acceptable alternative to the guidance. This resulted in the NRC sending requests for additional information and some of the issues already described in Section I.B. of this document.

C. NRC Response to These Issues

As a result of these issues, a backlog of NPUF license renewal applications developed and persisted. The Commission and other stakeholders voiced concerns not only about the backlog, but also about the burdensome nature of the license renewal process itself. The Commission issued SRM-M080317B, "Briefing on State of NRC Technical Programs," in April 2008, directing the staff to "examine the license renewal process for non-power reactors to identify and implement efficiencies to streamline this process while ensuring that adequate protection of public health and safety are maintained."

In October 2008, the staff provided the Commission with plans to improve the review process for NPUF license renewal applications in SECY-08-0161, "Review of Research and Test Reactor License Renewal Applications." In SECY-08-0161, the staff summarized a public meeting held with stakeholders to gather feedback on the current process, ways the process could be improved, and options for improving the review process. The staff provided a detailed description of five options for streamlining the NPUF license renewal process:

- An "alternate safety review approach" that would limit the review of license renewal applications to changes to the facility since the previous license review occurred. Safe operation of the facility would be assured by the review of changes to the facility, compliance with the current regulations, the previous NRC analysis, and the NRC's inspection process.
- A "graded approach" that would base the areas of review on the relative risk associated with the facility applying for a renewed license. The graded approach would ensure safe operation by properly identifying the inherent risk associated with the facility and ensuring those risks are minimized.

- A “generic analysis approach” that would require the NRC to review and approve a generic reactor design similar to the NRC topical report process. The NRC would rely on the previously approved generic analysis and would not reanalyze those items.
- A “generic siting analysis approach” that would require the NRC to develop a generic communication that contains information related to each of the licensee sites. The licensees could then reference this generic communication in their license renewal submittals.
- An “extended license term approach” would permit extended or indefinite terms for NPUF licenses. The staff described this approach in SECY-08-0161:

In order to permit an extended term (including possibly an indefinite term), the staff would have to explain why it is appropriate and, more importantly, demonstrate that there are no aging concerns.

Environmental conditions such as temperature, pressure and radiation levels in most [research and test reactors] are not significant. With surveillance, maintenance and repair, [research and test reactors] can have indefinite lives.

For a facility to be eligible for an extended license term, the staff would complete a detailed renewal with a licensing basis reviewed against NUREG-1537. To maintain the licensing basis over time, the staff would propose a license condition or regulation that requires licensees to revise their [safety analysis reports] on a periodic basis such as every 2 years. The inspection program would be enhanced to place additional focus on surveillance, maintenance and repair, and changes to the facility made under 10 CFR 50.59. The licensee would still be required to adhere to changes in the regulations.

The Commission issued SRM-SECY-08-0161, “Review of Research and Test Reactor License Renewal Applications,” in March 2009. The Commission directed the staff to: 1) immediately implement short-term program initiatives to address the backlog of license renewal applications; 2) work with the regulated community and other stakeholders to develop an interim streamlining process to focus the review on the most safety-significant aspects of the license renewal application; and 3) streamline the

review process to ensure that it becomes more efficient and consistent, thereby reducing uncertainties in the process while ensuring compliance with regulatory requirements.

As part of its direction to develop the program initiatives, the Commission instructed the staff to implement a graded approach commensurate with the risk posed by each facility, incorporate elements of the alternate safety review approach, and use risk insights from security assessments to inform the dose threshold. In addition, the Commission told the staff to develop an interim staff guidance (ISG) document that employs the graded approach to streamline the license renewal application process.

Lastly, the Commission instructed the staff to submit a long-term plan for an enhanced NPUF license renewal process. The Commission directed that the plan include development of a basis for redefining the scope of the process as well as a recommendation regarding the need for rulemaking and guidance development.

The staff responded to the Commission's direction by implementing short-term actions to address the license renewal application backlog and developing ISG-2009-001, "Interim Staff Guidance on the Streamlined Review Process for License Renewal for Research Reactors," hereafter referred to as the ISG. The ISG called for employing a graded approach to streamline the license renewal application process. Since October 2009, the NRC has reviewed license renewal applications according to the streamlined review process presented in the ISG. The ISG identified the three most safety-significant sections of an FSAR: reactor design and operation, accident analysis, and technical specifications. The NRC also has reviewed licensees' radiation protection and waste management programs and compliance with financial requirements. The ISG divided facilities into two groups: 1) those facilities with licensed power of less than 2 megawatts thermal (MW(t)), which would undergo a limited review focusing on the safety-significant aspects, considering the decisions and precedents set by past NRC reviews; and 2) those facilities with licensed power of 2 MW(t) and greater, which would

undergo a full review using NUREG-1537, Part 2. The process outlined in the ISG facilitated the NRC's review of license renewal applications and enabled the NRC to review applications in a timelier manner.

In addition, the staff issued SECY-09-0095, "Long-Term Plan for Enhancing the Research and Test Reactor License Renewal Process and Status of the Development and Use of the Interim Staff Guidance," in June 2009, to provide the Commission with a long-term plan for enhancing the NPUF license renewal process. In the long-term plan, the staff proposed to develop a regulatory basis to support rulemaking to streamline and enhance the NPUF license renewal process. The Commission issued SRM-M090811, "Briefing on Research and Test Reactor (RTR) Challenges," in August 2009, which directed the staff to accelerate the rulemaking to establish a more efficient, effective, and focused regulatory framework.

D. 2012 Regulatory Basis

In August 2012, the staff completed the "Non-Power Reactor (NPR) License Renewal Rulemaking: Regulatory Basis Document," hereafter referred to as the regulatory basis.²

The NRC, in the regulatory basis, analyzed the NPUF license renewal process's technical, legal, and policy issues; effects on public health, safety, and security; effects on licensees; effects on the NRC; and stakeholder feedback. The NRC also considered lessons learned from implementation of the streamlined review process outlined in the ISG. The NRC concluded that a rulemaking was warranted. A public meeting was held

² At the time of publication of the regulatory basis, the rulemaking title was "Non-Power Reactor (NPR) License Renewal Rulemaking." During the development of the proposed rule, the scope of the rulemaking expanded to include licenses for certain facilities that are not reactors, based upon recent license applicants (e.g., for medical radioisotope irradiation and processing facilities). In order to encompass all affected entities, the NRC has changed the title of the rulemaking to "Non-power Production or Utilization Facility License Renewal."

on August 7, 2014, to discuss the regulatory basis and rulemaking options. The NRC held another public meeting on October 7, 2015, to afford stakeholders the opportunity to provide feedback and comment on preliminary proposed rule concepts. Participant comments and questions focused on the potential effects of eliminating license terms, the scope of review under the new process, and how the amended regulation would work compared to the existing license renewal process. The NRC considered the comments when developing the proposed rule.

E. 2017 Proposed Rule

On March 30, 2017, the NRC published the proposed rule, “Non-Power Production or Utilization Facility License Renewal” in the *Federal Register* (82 FR 15643). The NRC proposed to eliminate license terms for facilities used for medical therapy or research and development licensed under the authority of Sections 104a or 104c of the AEA, other than for testing facilities. Other proposed amendments addressed the license renewal process for licenses issued to testing facilities under the authority of Section 104c of the AEA and licenses issued to non-power commercial facilities under the authority of Section 103 of the AEA (including testing facilities). The proposed rule also included a provision to require all NPUF licensees to submit FSAR updates to the NRC every 5 years. The NRC also proposed an accident dose criterion of 1 rem (0.01 Sv) TEDE for NPUFs other than testing facilities. The NRC requested public feedback on specific questions, including the criteria, other than power level, to use when determining the applicability of requirements for low-risk commercial production or utilization facilities and low-risk testing facilities. The proposed rule provided a public comment period of 75 days. The NRC received 16 comment submissions on the proposed rule and draft implementation guidance, as discussed

further in Section IV of this document. The NRC considered those comments in developing this final rule.

II. Discussion

This final rule: 1) revises the definitions for *Non-power reactor*, *Research reactor*, and *Testing facility*; 2) eliminates license terms for NPUFs licensed under § 50.21(a) or (c), other than testing facilities; 3) defines the license renewal process for NPUFs (including testing facilities) licensed under § 50.22 and testing facilities licensed under § 50.21(c); 4) requires all NPUF licensees to submit to the NRC an updated FSAR and subsequent FSAR updates at intervals not to exceed 5 years; 5) amends the current timely renewal provision under § 2.109, allowing an NPUF subject to license renewal to continue operating under an existing license past its expiration date if the licensee submits a license renewal application at least 2 years before the current license expiration date; 6) provides an accident dose criterion of 1 rem (0.01 Sv) TEDE for NPUFs other than testing facilities; 7) extends the applicability of § 50.59 to NPUFs regardless of their decommissioning status; 8) clarifies the requirements for NPUF license applicants to meet the existing provisions of § 51.45; and 9) eliminates the requirement to submit financial qualification information with license renewal applications under § 50.33(f)(2).

This final rule enhances the effectiveness and efficiency of the NPUF license renewal process, consistent with the AEA's criterion for imposing minimum regulation on facilities of these types that is needed to promote the common defense and security and protect the health and safety of the public. Each of the nine main objectives of this final rule are discussed in detail in this section.

1. *Revises the definitions for Non-power reactor, Research reactor, and Testing facility.*

This final rule addresses inconsistencies in definitions and terminology throughout 10 CFR chapter I to improve clarity in determining the applicability of the regulations associated with NPUFs as defined in § 50.2.

The NRC received public comments on the proposed definition of *Non-power production or utilization facility*. In reviewing the comments, the NRC identified that the proposed definition for *Non-power production or utilization facility* was too broad for defining production facilities that are NPUFs. Previously, the definition excluded fuel reprocessing plants, but did not exclude production facilities designed or used primarily for the formation of plutonium or uranium-233 or designed or used for the separation of the isotopes of plutonium. Ultimately, the NRC did not revise the definition for *Non-power production or utilization facility* because an appropriate definition to exclude all production facilities as defined under paragraphs (1) and (2) of the definition of *Production facility* in § 50.2 was added by the rule on Emergency Preparedness for Small Modular Reactors and Other New Technologies (88 FR 80050; November 16, 2023). Production facilities of the type defined under paragraph (1) of the definition of *Production facility* in § 50.2 have been owned by the U.S. Department of Energy to produce plutonium or uranium-233 and have not been NRC licensees. If such a facility were to be licensed by the NRC, the facility's particular use of special nuclear material would require the Commission to determine the licensing path for the facility. Production facilities, as defined under paragraph (2) of the definition of *Production facility* in § 50.2, are not NPUFs because these facilities have a higher potential of radiological risk to the environment and the public than NPUFs (e.g., an inventory of high-level liquid radioactive wastes). This higher risk is evidenced by the applicability to these facilities of NRC regulations in appendix B to 10 CFR part 50, "Quality Assurance Criteria for

Nuclear Power Plants and Fuel Reprocessing Plants” and appendix F to 10 CFR part 50, “Policy Relating to the Siting of Fuel Reprocessing Plants and Related Waste Management Facilities.” The definition of *Non-power production or utilization facility* in § 50.2 excludes production facilities designed or used primarily for the formation of plutonium or uranium-233 or the separation of the isotopes of plutonium.

The NRC also received a comment from the National Institute of Standards and Technology on the definition of *Testing facility* in § 50.2 and *Research reactor* in § 171.11(b)(2). The commenter recommended that the NRC revise the definitions of *Testing facility* and *Research reactor* to “remove the arbitrary 10 MW(t) threshold, and apply instead a risk-based approach to its regulation of a testing facility.” Further, the commenter stated that the risk “is best quantified by accident analyses performed under a licensing safety analysis” and linked the recommended definition to the NRC’s accident dose criterion of 1 rem (0.01 Sv) in the proposed rule.

The technical basis associated with the 10 MW(t) threshold under the current definition for *Testing facility*, while generally based on safety significance, is not explicitly documented. Similarly, the technical basis for the 1 MW(t) threshold (coupled with specific design features) under the current definition for *Testing facility* is not explicitly documented. These prescriptive power thresholds do not account for the safety features that are engineered into the facility design and those barriers that must be breached during an accident before a release of radioactive material to the environment can occur. Therefore, these thresholds do not accurately represent the risk associated with a particular facility. For these reasons, the use of a postulated accident dose is a more risk-informed, performance-based approach, compared to using the power level of the reactor for distinguishing between types of NPUFs, such as research reactors and testing facilities. As a result of this public comment, the NRC revised the definitions of *Testing facility* and *Research reactor* to reflect this risk-informed approach by

incorporating an accident dose criterion of 1 rem (0.01 Sv) TEDE, the basis for which is discussed in section II.6 of this document.

Additionally, the NRC is making conforming changes to the definitions of *Testing facility*, *Research reactor*, and *Non-power reactor* wherever these definitions appear throughout 10 CFR chapter I. The regulations currently refer to many types of facilities that are categorized as NPUFs, such as non-power reactors, research reactors, training reactors, testing reactors, testing facilities, and critical assemblies. The NRC reviewed each instance of these various terms in 10 CFR chapter I. Where appropriate in this final rule, the NRC added, corrected, or standardized the terminology and definitions.

While this final rule revises the definition of *Research reactor* in §§ 170.3 and 171.5 to conform to other definitions in 10 CFR chapter I, the NRC did not change the definition of *Research reactor* in the specific exemption for Federally owned and State-owned research reactors in § 170.11(a)(9) or § 171.11(b)(2). The current definition in § 171.11(b)(2) is based on the language of the Omnibus Budget Reconciliation Act of 1990, as amended (Pub. L. No. 101-508) (OBRA-90), a statutory requirement imposed by Congress. Further, a substantively similar definition of *Research reactor* was included in the provisions of the Nuclear Energy Innovation and Modernization Act (Pub. L. No. 115-439) (NEIMA) that relate to the NRC's fee recovery structure. Changing the definition of *Research reactor* in § 171.11(b)(2) would therefore be inconsistent with OBRA-90 and NEIMA. The definition of *Research reactor* in § 170.11(a)(9) is not based on OBRA-90, but the basis for that exemption from fees parallels the basis for the exemption from annual fees in § 171.11(b)(2). Changing the definition of *Research reactor* in § 170.11(a)(9) would be a substantive change beyond the scope of this final rule.

Where appropriate, this final rule standardizes the terminology in other parts of the regulations to modify the intended scope of regulations citing *Research and test*

reactors to be either *Non-power reactors* or *Non-power production or utilization facilities*. For example, this final rule changes *Research and test reactors* to *Non-power production or utilization facilities* in appendix E to 10 CFR part 50, “Emergency Planning and Preparedness for Production and Utilization Facilities,” while in § 55.40, this final rule changes *Test and research reactors* to *Non-power reactors*. Also, where appropriate, the final rule changes the uses in other parts of the regulations for *Testing facility*, *Research reactor*, and *Non-power reactor* to reference only one definition in the part where that definition is used most, unless the specific meaning is needed and different for a given part. In addition, the final rule adds the definition of *Non-power reactor*, as it is defined in § 50.2, to the definitions section in 10 CFR part 73 because the term is used many times throughout that part. These changes increase clarity by defining all NPUF-related terms consistently where they are most used in the regulations.

This final rule also revises the definition of *Non-power reactor* to distinguish between non-power reactors used for research and development activities and non-power reactors used for commercial or industrial purposes. Before this final rule, all non-power reactors were defined in § 50.2 as “a research or test reactor licensed under §§ 50.21(c) or 50.22 of this part for research and development.” This final rule defines non-power reactors more precisely as one of three mutually exclusive categories of facilities: 1) testing facilities, 2) research reactors that are NPUFs licensed under § 50.21(c), or 3) commercial or industrial reactors that are NPUFs licensed under § 50.22. The second and third categories exclude testing facilities, and the facilities in those categories must meet the accident dose criterion in § 50.34(a)(1)(i). If they do not meet this criterion, then they will be considered testing facilities.

2. *Eliminates license terms for NPUFs, other than testing facilities, licensed under § 50.21(a) or (c).*

The final rule language in § 50.51(c) eliminates license terms for NPUFs, other than testing facilities, licensed under § 50.21(a) or (c). Before this final rule, § 50.51(a) stated, “Each license will be issued for a fixed period of time to be specified in the license but in no case to exceed 40 years from date of issuance.” This included all facility licenses issued under 10 CFR part 50, including licenses for facilities issued under § 50.21(a) or (c). However, the AEA does not establish specific license terms nor the need for license terms for class 104 facilities.

Historically, license renewal afforded both the NRC and the public the opportunity to re-evaluate the licensing basis of the NPUF. The purpose of license renewal was to assess the likelihood of continued safe operation of the facility, such that radioactive materials can be used for beneficial civilian purposes in a safe and secure manner. For several reasons that are unique to NPUFs, this objective can be achieved through existing oversight activities and review of FSAR updates submitted pursuant to the new requirements in § 50.71(e) of the final rule (see Section II.4. of this document). This approach is consistent with the NRC’s goal of efficient and effective licensing and will implement and reflect lessons learned from decades of processing license renewal applications. The NRC reached this conclusion based on three considerations: 1) low overall radiological risk, 2) limited aging-related issues, and 3) slow evolution of the design basis.

First, compared to power reactors, the NPUFs licensed under § 50.21(a) or (c), other than testing facilities, operate at low power levels, temperatures, and pressures, and have a small inventory of fission products in the fuel. Therefore, these NPUFs present a lower potential radiological risk to the environment and the public. Additionally, the consequences of the maximum hypothetical accidents (MHAs) for these facilities fall below the standards in 10 CFR part 20 for protecting the health and safety of the public.

Of the 30 NPUFs that are currently licensed to operate and are eligible for non-expiring licenses (excluding the one testing facility), 26 have cores that are submerged in tanks or pools of water that provide sufficient passive decay heat removal to prevent overheating of the fuel.³ Of these 26 licensed facilities, 24 are not required to have emergency core cooling systems (ECCSs) because conservative accident analyses have shown that these NPUFs do not generate enough decay heat, even after extended operation at maximum licensed power, to be at risk of overheating, failure of a fission product barrier, or posing a threat to public health and safety. Additionally, many of the licensees monitor for leaks by routinely inspecting the facility, tracking and trending water inventory, and performing surveillance on installed pool-level instrumentation and sensors. Licensees sample the water periodically and analyze the radioisotopes in the primary and, if applicable, secondary coolant. Many licensees sample weekly for gross radioactive material content. This data also is used to establish trends to quickly identify fuel or heat exchanger failure. Most of these licensees analyze, in their FSARs, pool and heat exchanger failures and the potential consequences for the safety of the reactor, workers, and public. In general, the radioisotope concentrations in pool or tank water at NPUFs are within the effluent concentration limits specified in appendix B to 10 CFR part 20, and therefore are not radiologically significant.

Only two of the NPUFs eligible for non-expiring licenses are required by their safety analyses to have an ECCS to maintain core cooling in the highly unlikely case that a loss-of-coolant accident uncovers the core.⁴ For these NPUFs, the ECCS is needed only to direct flow into the top of the tank or pool to provide cooling for a limited

³ The three Aerojet-General Nucleonics reactors (University of New Mexico (Docket No. 50-252), Idaho State University (Docket No. 50-284), and Texas A&M University (Docket No. 50-59)), each rated at 5 watts, and the University of Florida Argonaut reactor (Docket No. 50-83), rated at 100 kilowatts, are not considered tank or pool reactors but have similarly low risk profiles.

⁴ The two facilities are Massachusetts Institute of Technology (MIT) (Docket No. 50-20) and the University of California/Davis (Docket No. 50-607).

time after reactor shutdown. This period of time depends on the recent operational history of the reactor, which determines the decay heat present at reactor shutdown. After this relatively brief time, air cooling is adequate to remove decay heat without the ECCS. Additionally, required surveillance and testing of the ECCS at these facilities help ensure the performance of the system. Operation of the facility is not permitted if the ECCS has not been verified to be operable before reactor startup or if the system is deemed inoperable during reactor operation.

Second, the NRC has found that the simple design and operation of these facilities yield a limited scope of aging-related concerns. There have been no significant aging issues identified at the time of license renewal because the NRC currently imposes aging-related surveillance requirements on NPUFs via technical specifications, as needed. Aging of components is specifically addressed in the standard review plan and acceptance criteria used for evaluating license renewal applications (i.e., NUREG-1537, Part 2). Parts 1 and 2 of NUREG-1537 document lessons learned and known aging issues from prior reviews. Since NUREG-1537 was published in 1996, NRC reviews and assessments have not revealed any additional issues or need to update the NUREG. Specifically, based on operating experience over the past 60 years and review of license renewal applications over the past 40 years, and as documented in NUREG-1537, Parts 1 and 2, the NRC has determined that for NPUFs, the two main areas related to aging that could need surveillance because of potential safety concerns are 1) fuel cladding and 2) instrumentation and control features.

Regarding fuel cladding, the NRC currently requires NPUFs to perform periodic fuel inspections. Through years of experience, the NRC has found that aging-related fuel failures either do not occur, or failures that do occur do not release significant amounts of fission products and are quickly detected by existing monitoring systems and

surveillances. If fuel failures are detected, licensees are able to take the facility out of service and remove any failed assemblies from service.

With regard to instrumentation and control, the NRC has found that failures in this area result in automatic facility shutdown. Failures reveal themselves to the licensee and do not prevent safe shutdown. Over the past 60 years of operation of these facilities, the potential occurrence of age-related degradation has been successfully mitigated through inspection, surveillance, monitoring, trending, recordkeeping, replacement, and refurbishment. In addition, licensees are required to report preventive and corrective maintenance activities in their annual reports, which are reviewed by the NRC. This allows the NRC to identify new aging issues if they occur. Therefore, the NRC has concluded that existing requirements and facility design and operational features will address concerns over aging-related issues during a non-expiring license term.

Third, the design bases of these facilities evolve slowly over time, with approximately five license amendment requests from all NPUF licensees combined each year and, on average, only five § 50.59 evaluations per facility per year for changes that do not require prior NRC approval.

Given these considerations, the elimination of license terms for medical therapy or research and development facilities, other than testing facilities, licensed under § 50.21(a) or (c), combined with the addition of requirements for periodic FSAR submittals, will provide a new framework for enabling licensees to continue to operate safely while reducing burden on licensees and the NRC. The final rule at § 50.71(e) requires licensees to submit updated FSARs and subsequent FSAR updates to ensure that a facility's licensing basis is kept up-to-date, a major function previously provided by the license renewal process, while imposing significantly less burden on licensees. Eliminating license terms for these licensees will allow the NRC to focus its resources on

oversight of these facilities, such as conducting routine inspection activities and reviewing annual reports and FSAR updates. Recurring FSAR updates by licensees and reviews by the NRC will increase licensees' focus on maintaining their facilities' licensing bases. Should the NRC identify potential issues with the facility's continued safe operation in its reviews of FSAR updates, the Commission can undertake regulatory actions specified in § 2.202 to modify, suspend, or revoke a license. In addition, the public will remain informed about facility operations through the publicly available FSAR submittals and will continue to have opportunities to participate in the regulatory process through licensing actions and the § 2.206 petition process. By eliminating license terms and requiring periodic FSAR update submittals, coupled with existing oversight processes, the NRC will reduce the burden on the affected licensees and the NRC, which is consistent with the AEA and supports the NRC's goal of efficient and effective licensing.

Most licenses of existing NPUFs licensed under § 50.21(a) or (c), other than testing facilities, will be modified by order to remove the license terms after the effective date of this final rule (see Section II.4. of this document). Facilities licensed under § 50.21(a) or (c), other than testing facilities, that have undergone relicensing using the guidance in NUREG-1537, Part 2 will be eligible to receive a non-expiring license without again renewing the current license. The current NPUF licensees that have not undergone the license renewal process using the guidance in NUREG-1537, Part 2, will each need to submit an application for license renewal if they wish to continue facility operation beyond the current license term. The NRC will review the application using NUREG-1537, Part 2, and the ISG. If the NRC concludes that a licensee's application meets the standard for issuing a renewed license, then the NRC would issue a non-expiring renewed license. If, in the future, the NRC issues an operating license to a new facility, other than a testing facility, under § 50.21(a) or (c), the license would be

non-expiring and would be subject to periodic FSAR submittal requirements applicable to all NPUF licensees.

This final rule makes conforming changes to requirements for facilities that are decommissioning by revising § 50.82(b) and (c). These provisions currently use the expiration of the operating license as a reference point to address license termination applications and collection periods for shortfalls in decommissioning funding for NPUFs. This final rule clarifies that NPUFs (including testing facilities) licensed under § 50.22 and testing facilities licensed under § 50.21(c) are the only NPUFs with license expiration dates. The reference point for NPUFs licensed under § 50.21(a) or (c), other than testing facilities, is the NPUF's permanent cessation of operations.

3. Defines the license renewal process for NPUFs (including testing facilities) licensed under § 50.22 and testing facilities licensed under § 50.21(c).

For NPUFs (including testing facilities) licensed under § 50.22 and testing facilities licensed under § 50.21(c), this final rule defines the license renewal process in § 50.135. This one section consolidates existing regulatory requirements (e.g., requirements regarding written communications, application filing, application contents, and the issuance of renewed licenses) for current and future licensees. This final rule does not impose new regulations on these facilities. The NRC also is making a conforming change to § 50.8 to reflect the approved information collection requirement of § 50.135.

Section 103 of the AEA establishes a license term of no more than 40 years for commercial or industrial facilities licensed under § 50.22. Although the AEA does not establish a fixed license term for testing facilities, licensees for these facilities are currently subject to additional license renewal requirements (e.g., siting subject to 10 CFR part 100, Advisory Committee on Reactor Safeguards review, and environmental impact statements) because of the potential for higher radiological risks associated with

their facilities' design, operation, or use as compared to other class 104a or 104c licensees. Therefore, all commercial or industrial NPUFs (including testing facilities) licensed under § 50.22 and testing facilities licensed under § 50.21(c) will continue to have fixed license terms and undergo license renewal. As described in § 50.135(c)(2), these NPUFs will be able to submit a license renewal application to the Commission no more than 10 years in advance of the expiration of the operating license currently in effect. The requirement in § 50.135(c)(2) is not intended to affect the term of operating licenses granted to NPUFs.

The NRC is making renewed operating licenses for these facilities effective, and thereby replacing the previous operating license, immediately upon the date of issuance. The applicant for the renewed license can propose a schedule for implementation of the renewed license. This implementation schedule would ensure that the licensee can make any necessary and conforming changes to the facility processes and procedures required by the applicable conditions of the renewed license. The NRC will review and make the schedule, if approved, a condition of the renewed license. The immediate effectiveness of the renewed license is a change from the proposed rule, which would have made the renewed license effective 30 days after issuance. This final rule provides a substantively similar result as the proposed rule and provides licensees additional flexibility in the timing of their implementation of the renewed license.

If administrative or judicial appeal affects the renewed license, then the previous operating license will be reinstated unless its term has expired and the facility has failed to submit a license renewal application in a timely manner.

During the development of this final rule, the NRC recognized that § 50.135(e)(2) in the proposed rule could have unnecessarily restricted the license term for a renewed NPUF license to less than 40 years. Section 103 of the AEA allows for license terms of

up to 40 years. To address this issue, this final rule clarifies that renewed licenses are issued for a fixed period of time, not to exceed 40 years.

4. Requires all NPUF licensees to submit to the NRC updated FSARs and subsequent FSAR updates at intervals not to exceed 5 years.

Maintaining up-to-date FSARs facilitates safe management of a facility, including current understanding of the licensing bases and effective training of personnel, and enables the NRC to fulfill its statutory obligations and regulatory responsibilities effectively. Section 50.71(e) of the final rule requires all NPUF licensees to submit to the NRC updated FSARs and subsequent FSAR updates at intervals not to exceed 5 years. The updated FSAR will incorporate the various supplements and amendments that may have been submitted, either in response to NRC questions or on the licensee's own initiative, following the original submittal to create a single and complete updated document that can then serve as the baseline for future changes. Given the requirement to submit subsequent FSAR updates, the NRC anticipates that licensees will document changes to the licensing bases as they occur, which will aid in maintaining continuity of knowledge and the understanding of changes and effects of changes on the facility both for the licensee and the NRC. The NRC anticipates that these changes will result in minimal additional burden on licensees and the NRC because only a small number of changes have occurred per facility each year. In addition, licensees should have already documented these changes under § 50.59 or through a license amendment request under § 50.90.

This final rule requires licensees to submit, in accordance with § 50.4, a complete updated FSAR within 5 years of receipt of a facility operating license (§ 50.71(e)(3)(iv)) and subsequent FSAR updates at successive intervals not to exceed 5 years (§ 50.71(e)(4)(ii)). The NRC will issue orders to existing facilities licensed under § 50.21(c) that have undergone the license renewal process using the guidance in

NUREG-1537, Part 2. These licensee-specific orders will direct these licensees to submit their updated FSARs, after which they will be subject to the new requirement in § 50.71(e)(4)(ii) to submit subsequent FSAR updates.

To issue the licensee-specific orders, the NRC will group the facilities based upon when they have undergone license renewal using NUREG-1537. The orders will dictate when a licensee's initial updated FSAR will be due to the NRC. The NRC plans to stagger the dates over a 5-year period following the effective date of this final rule. The NRC will place existing operating and decommissioning NPUF licensees in three groups as follows:

1) Group 1 consists of licensees that completed the license renewal process most recently using NUREG-1537. The NRC will establish a due date for the updated FSAR that will be at least 1 year and no later than 3 years from the effective date of this final rule. The NRC will require these licensees to submit an updated FSAR first because, with a recent license renewal, the FSARs should require minimal updates.

2) Group 2 generally consists of licensees for which the NRC reviewed the license renewal application before Group 1 using NUREG-1537, and includes the three facilities currently in decommissioning. The NRC will establish a due date for the updated FSAR that will be at least 2 years and no later than 5 years from the effective date of this final rule. The NRC will allow these licensees more time to submit an updated FSAR than Group 1 licensees because more time has passed since license renewal, so additional time may be needed to update their FSARs.

3) Group 3 consists of the remaining NPUF licensees that have not undergone license renewal using NUREG-1537. The licenses for these facilities are all due to expire in less than 5 years from the effective date of this final rule. If these licensees choose to renew their facility operating licenses, they will be subject to the requirements in § 50.71(e) after issuance of the renewed license.

The general approach will be to stagger the submittal dates within Groups 1 and 2 such that licensees that most recently completed license renewal will be the first to submit their updated FSAR. However, the licensee-specific orders will also consider facility-specific circumstances and NRC discretion.

This final rule also corrects a grammatical error in footnote 1 to § 50.71(e). The footnote previously stated, “Effects of changes includes appropriate revisions of descriptions in the FSAR such that the FSAR (as updated) is complete and accurate.” This final rule changes “includes” to “include” so that the plural subject is followed by a plural verb.

5. Amends the current timely renewal provision under § 2.109, allowing an NPUF subject to license renewal to continue operating under an existing license past its expiration date if the licensee submits a license renewal application at least 2 years before the current license expiration date.

The requirements in § 2.101(a) allow the NRC to determine the acceptability of an application for review by the NRC. However, before this final rule, § 2.109 allowed an NPUF licensee to submit its license renewal application as late as 30 days before the expiration of the existing license. Historical precedent indicates that 30 days is not a sufficient period of time for the NRC to adequately assess the sufficiency of a license renewal application for review. As a result, the NRC accepted license renewal applications and addressed their deficiencies in the license renewal process by issuing requests for additional information. This approach increased the duration of the license renewal process and resulted in multiple facilities operating many years into a “timely renewal” period without renewed licenses.

To address this issue, the NRC is revising the timely renewal provision for NPUFs (including testing facilities) licensed under § 50.22 and testing facilities licensed under § 50.21(c) to establish a length of time adequate for the NRC to review the

sufficiency of a license renewal application. Specifically, this final rule amends § 2.109, allowing a facility to continue operating under an existing license past its expiration date if the licensee submits a sufficient license renewal application at least 2 years before the current license expiration date. In such cases, the existing license will not be deemed to have expired until the application has been finally determined by the NRC. This final rule ensures that the NRC has adequate time prior to the expiration of the current license to review the sufficiency of license renewal applications while the facility continues to operate under the terms of its current license.

The proposed rule would have eliminated this provision for medical therapy or research and development facilities, other than testing facilities, licensed under § 50.21(a) or (c), because these facilities would no longer have license expiration dates. The NRC reinstates the provision in this final rule to enable its use for the remaining license renewal applications that may be submitted after this final rule is published. The NRC anticipates that there is one research reactor licensee that would use this provision.

6. Provides an accident dose criterion of 1 rem (0.01 Sv) TEDE for NPUFs other than testing facilities.

The standards in 10 CFR part 20 for protection against ionizing radiation provide a limit on the maximum yearly radiation dose a member of the public can receive from the operation of any NRC-licensed facility. Licensees are required to maintain programs and facility design features to ensure that these limits are met. In addition to the dose limits in 10 CFR part 20, accident dose criteria are also applied during licensing to determine the acceptability of the licensed facility. The accident dose criteria are not dose limits; they inform a licensee's accident analyses and the development of successive safety measures (i.e., defense in depth) so that in the unlikely event of an accident, the NRC has reasonable assurance that no acute radiation-related harm will

result to any member of the public. Before this final rule, the accident dose criterion for NPUFs, other than testing facilities, was the 10 CFR part 20 dose limit to a member of the public. For testing facilities, accident dose criteria are found in 10 CFR part 100: 25 rem (0.25 Sv) to the whole body and 300 rem (3 Sv) to the thyroid.

Before January 1, 1994, the NRC had generally found acceptable accident doses for applicants applying for an initial or renewed NPUF license, other than for testing facilities, that were less than 0.5 rem (0.005 Sv) to the whole body and 3 rem (0.03 Sv) to the thyroid for members of the public. On May 21, 1991,⁵ the NRC amended 10 CFR part 20 to reduce the dose limit to a member of the public to 0.1 rem (0.001 Sv) TEDE (56 FR 23360) with an implementation date of January 1, 1994. Since January 1, 1994, for applicants applying for an initial or renewed NPUF license, other than for testing facilities, the NRC has compared the results from the accident analyses submitted in initial or renewed license applications with the standards in 10 CFR part 20.

The NRC has determined that the public dose limit of 0.1 rem (0.001 Sv) TEDE in 10 CFR part 20 is unduly restrictive to be applied as accident dose criteria for NPUFs except for testing facilities, which are subject to 10 CFR part 100. The NRC bases this determination on the NRC Atomic Safety and Licensing Appeal Board's decision that the standards in 10 CFR part 20 are unduly restrictive as accident dose criteria for research reactors (Trustees of Columbia University in the City of New York, ALAB-50, 4 AEC 849, 854-855 (May 18, 1972)). At the time of this decision, the 10 CFR part 20 public dose limit was 0.5 rem (0.005 Sv) whole body.

However, the NRC considers the accident dose criteria in 10 CFR part 100 to be too high for NPUFs other than testing facilities, because those NPUFs have lower risk profiles than testing facilities. For these reasons, this final rule modifies § 50.34 to add

⁵ In the proposed rule, the NRC misidentified the part 20 rulemaking date as January 1, 1994.

an accident dose criterion of 1 rem (0.01 Sv) TEDE for NPUFs not subject to 10 CFR part 100. The accident dose criterion of 1 rem (0.01 Sv) TEDE is based on the Environmental Protection Agency's (EPA) Protection Action Guides (PAGs). The EPA PAGs are dose guidelines that support decisions during a radiological incident to take protective actions such as staying indoors or evacuating. The proposed rule stated that the 1 rem (0.01 Sv) TEDE accident dose criterion was based on the EPA PAGs published in EPA 400-R-92-001, "Manual of Protective Action Guides and Protective Actions for Nuclear Incidents." In January 2017, the EPA published an update to its PAGs in EPA-400/R-17/001, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incidents." This update to the EPA PAGs does not change the basis for the 1 rem (0.01 Sv) TEDE accident dose criterion.

The PAG is defined as the projected dose to an individual from a release of radioactive material at which a specific protective action to reduce or avoid that dose is recommended. Three principles considered in the development of the EPA PAGs include: 1) prevent acute effects; 2) balance protection with other important factors and ensure that actions result in more benefit than harm; and 3) reduce risk of chronic effects. In the early phase (i.e., the beginning of the radiological incident, which may last hours to days), if the sum of the projected dose from external radiation exposure and the inhalation of radioactive material is 1 rem (0.01 Sv) to 5 rem (0.05 Sv), the EPA PAG recommends the protective action of sheltering-in-place or evacuation of the public to avoid inhalation of gases or particulates in an atmospheric plume and to minimize external radiation exposures. The EPA PAG Manual does not provide a protective action recommendation for the public when the projected dose to an individual from an incident is less than 1 rem (0.01 Sv). In light of this understanding of the early phase EPA PAG, the NRC's accident dose criterion of 1 rem (0.01 Sv) TEDE for NPUFs, other

than testing facilities, provides reasonable assurance of adequate protection of the public from unnecessary exposure to radiation.

The NRC revised § 50.34(a)(1)(ii)(D)(2) of the proposed rule to replace “postulated accidental release of licensed material” with “postulated accident.” This final rule requires applicants and licensees to evaluate the potential dose from postulated accidents to include the potential exposure from all radiological sources, such as direct or scattered radiation from an unshielded source inside the facility, in addition to potential exposure from a release of radioactive materials. This requirement is consistent with the evaluation methodology described in NUREG-1537, Part 1. Under this final rule, these evaluations need to demonstrate that the dose to any individual located in the unrestricted area will not be in excess of 1 rem (0.01 Sv) TEDE for the duration of the accident. Although the EPA PAGs were developed for radiological incidents that lead to the release or potential release of radioactive materials into the environment, the three principles considered in their development are not dependent on whether the dose received is due to exposure from a release of radioactive materials or from direct or scattered radiation.

To provide further clarification on the NRC’s intent of the 1 rem (0.01 Sv) TEDE accident dose criterion for NPUFs, other than testing facilities, a footnote has been incorporated into the final rule text. The footnote clarifies that this 1 rem (0.01 Sv) TEDE accident dose criterion is not a dose limit, as explained in the preceding paragraphs.

In this final rule, the NRC moves proposed § 50.34(a)(1)(ii)(D)(2) to § 50.34(a)(1)(i) and leaves the rule language in § 50.34(a)(1)(ii)(D) unchanged. During the development of this final rule, the NRC recognized that the accident dose criterion more appropriately belongs in § 50.34(a)(1)(i) because the requirements in § 50.34(a)(1)(ii) apply to power reactor construction permit applicants, while the requirements in § 50.34(a)(1)(i) apply to all other construction permit applicants, such as

NPUF applicants. Similarly, proposed § 50.34(a)(1)(ii)(D)(2) would have imposed a requirement on applications for renewed NPUF operating licenses, which more appropriately belongs in § 50.34(b). Therefore, the NRC moved the requirement to new § 50.34(b)(13) in this final rule to clarify that an application for an operating license or a renewed operating license for an NPUF must include in the FSAR a final evaluation of the applicable radiological consequences consistent with § 50.34(a)(1)(i).

7. Extends the applicability of § 50.59 to NPUFs regardless of their decommissioning status.

Before this final rule, § 50.59(b) of the Commission's regulations did not apply § 50.59 to NPUFs whose licenses were amended to reflect permanent cessation of operations and that no longer had fuel on site (e.g., they returned all of their fuel to the U.S. Department of Energy). The former language stated that § 50.59 applied to licensees "whose license has been amended to allow possession of nuclear fuel, but not operation of the facility." Therefore, § 50.59 did not apply to NPUF licensees that no longer possessed nuclear fuel. For these licensees, the NRC has typically added license conditions identical to the provisions of § 50.59 to allow the licensee to make changes to its facility or changes in its procedures that would not otherwise require obtaining a license amendment pursuant to § 50.90. Because most NPUFs promptly return their fuel to the U.S. Department of Energy after permanent shutdown, in contrast to decommissioning power reactors, these licensees had to request the addition of the license conditions, which imposed an administrative burden on the licensees and the NRC. This final rule eliminates this burden by revising § 50.59(b) to extend the applicability of § 50.59 to NPUFs regardless of their decommissioning status.

8. Clarifies an applicant's requirements for meeting the existing provisions of § 51.45.

The NRC is required to prepare either an environmental impact statement or environmental assessment, as appropriate, for all licensing actions pursuant to 10 CFR part 51, unless a categorical exclusion applies as provided in § 51.22. For most types of licenses, 10 CFR part 51 specifies that an applicant must submit environmental documentation in the form of an environmental report, or a supplement to a previously submitted environmental report, to assist the NRC's review and its compliance with the National Environmental Policy Act of 1969, as amended. However, before this final rule, the NRC did not have explicit requirements under 10 CFR part 51 with respect to the nature of the environmental documentation that must accompany applications for construction permits, initial licenses, and renewed licenses for NPUFs.

This final rule adds a new section to 10 CFR part 51 to clarify NPUF environmental reporting requirements. Section 51.56 clarifies an applicant's existing requirements for meeting the provisions of § 51.45. This change improves consistency throughout 10 CFR part 51 with respect to environmental report submissions required from applicants. The NRC also is making a conforming change to § 51.17 to reflect the approved information collection requirement of § 51.56.

9. Eliminates the requirement for NPUF licensees to submit financial qualification information with license renewal applications under § 50.33(f)(2).

This final rule eliminates license renewal financial qualification requirements for NPUFs. Before this final rule, § 50.33(f) required NPUF license applicants to provide information sufficient to demonstrate their financial qualifications to carry out the activities for which the license is sought. Because the regulatory requirements for the content of an application for a renewed NPUF license were the same as those for an original license, NPUF licensees that requested license renewal were required to submit an update to the same financial information that was required in an application for an initial license. In addition, the NRC found that the financial qualification information did

not meaningfully contribute to the NRC's safety determination on the license renewal application. The elimination of NPUF license renewal financial qualification requirements reduces the burden associated with license renewal applications while still enabling the NRC to conduct its review of these applications.

This change is consistent with the current license renewal process for power reactors. On January 30, 2004, the NRC published in the *Federal Register* the final rule, "Financial Information Requirements for Applications to Renew or Extend the Term of an Operating License for a Power Reactor" (69 FR 4439). This final rule discontinued financial qualification reviews for power reactors at the license renewal stage except in very limited circumstances. The Commission stated that "[t]he NRC believes that its primary tool for evaluating and ensuring safe operations at nuclear power reactors is through its inspection and enforcement programs...." Further, the Commission stated that "[t]he NRC has not found a consistent correlation between licensees' poor financial health and poor safety performance. If a licensee postpones inspections and repairs that are subject to NRC oversight, the NRC has the authority to shut down the reactor or take other appropriate action if there is a safety issue."

At NPUF sites, the NRC's inspection and enforcement programs serve as important tools for evaluating licensee performance and ensuring safe operations. The NRC periodically inspects each operating NPUF using a graded approach that prioritizes higher-power facilities. The NRC completes an annual inspection of NPUFs licensed to operate at power levels of 2 MW(t) or greater. For NPUFs operating under 2 MW(t), the inspection program is designed to be completed every two years, although inspector availability and licensee availability sometimes dictate that an inspection cycle is carried out in multiple inspections over the 2-year cycle. Inspections can include reviews of organizational structure, operator training and qualification, design and design control, radiation and environmental protection, maintenance and surveillance activities,

transportation, material control and accounting, operational activities, review and audit functions, experiments, fuel handling, procedural controls, emergency preparedness, and security. The NRC also performs special and reactive inspections. In addition, the NRC manages the NPUF operator license examination program. The NRC also manages the review of NPUF emergency and security plans and develops and implements policy and guidance concerning the NPUF licensing program.

The same basis for the NRC's elimination of financial qualification requirements for power reactor licensees at the time of license renewal supports the NRC's elimination of NPUF financial qualification requirements at the time of license renewal. The NRC is not aware of any connection between an NPUF's financial qualifications at license renewal and safe operation of the facility. The NRC retains broad authority under the AEA and § 50.54(cc), § 50.54(f), and § 2.102 to request additional financial information from its licensees and applicants, as necessary, to protect public health and safety.

III. Opportunity for Public Participation

The NRC hosted two public meetings to engage with external stakeholders on the proposed rule and associated draft guidance document during the public comment period. A public meeting was held on May 24, 2017, to discuss the proposed rule. A public meeting on the implementation schedule of the final requirements was held on April 25, 2019. Summaries of both public meetings are available in ADAMS, as provided in the "Availability of Documents" section. The feedback from these public meetings informed the development of this final rule.

IV. Public Comment Analysis

The NRC prepared a summary and analysis of public comments received on the 2017 proposed rule and draft regulatory guide, as referenced in the “Availability of Documents” section. In response to the proposed rule and draft regulatory guide, the NRC received 16 comment submissions.

The public comment submittals are available from the Federal e-Rulemaking Web site at <http://www.regulations.gov> under Docket ID NRC-2011-0087. Responses to the public comments, including a summary of how the final rule text or guidance changed as a result of the public comments, can be found in the public comment analysis document.

For more information about the associated guidance document, see the “Availability of Guidance” section of this document.

V. Section-by-Section Analysis

The following paragraphs describe the specific changes within this final rule.

Section 2.109 Effect of timely renewal application.

In § 2.109, this final rule revises paragraph (a) to exclude NPUFs (including testing facilities) licensed under § 50.22 and testing facilities licensed under § 50.21(c) from the 30-day timely renewal provision by adding paragraph (f) to require these same licensees to submit a license renewal application at least 2 years before license expiration to be considered timely.

Section 20.1905 Exemptions to labeling requirements.

In § 20.1905, this final rule revises paragraph (g) to standardize terminology by replacing the term “reactors” with the phrase “production or utilization facilities.”

Section 26.3 Scope.

In § 26.3, this final rule revises paragraph (e) to standardize terminology by replacing the term “reactor” with the phrase “production or utilization facility.”

Section 50.2 Definitions.

In § 50.2, this final rule revises the definitions for *Non-power reactor* and *Testing facility*.

Section 50.8 Information collection requirements: OMB approval.

In § 50.8, this final rule revises paragraph (b) to include new § 50.135 as an approved information collection requirement in 10 CFR part 50.

Section 50.33 Contents of applications; general information.

In § 50.33, this final rule revises paragraph (f)(2) to remove the phrase “for a power reactor” from the fourth sentence and to remove the fifth sentence, which required a non-power reactor applicant to submit with license renewal applications the same financial information that is required for initial license applications. It also redesignates the footnote to conform to the Office of the Federal Register’s requirements.

Section 50.34 Contents of applications; technical information.

In § 50.34, this final rule revises paragraph (a)(1)(i) to include an accident dose criterion for applicants for construction permits for NPUFs not subject to 10 CFR part 100 and a new footnote 2. It also redesignates the footnotes to conform to the Office of the Federal Register’s requirements. This final rule also adds paragraph (b)(13) to require an applicant for an operating or a renewed operating license for an

NPUF to include in the FSAR a final evaluation of the applicable radiological consequences in § 50.34(a)(1)(i).

Section 50.36 Technical specifications.

In § 50.36, this final rule revises paragraph (c)(6) to standardize terminology by replacing the term “non-power reactor” with the phrase “non-power production or utilization.”

Section 50.51 Continuation of license.

In § 50.51, this final rule revises paragraph (a) to add the conditional phrase “except as noted under § 50.51(c).” This final rule also adds new paragraph (c) to clarify that NPUFs licensed under § 50.21(a) or (c), other than testing facilities, after the effective date of this final rule, will have non-expiring license terms.

Section 50.59 Changes, tests, and experiments.

In § 50.59, this final rule revises paragraph (b) to extend applicability to NPUFs that have permanently ceased operations and that no longer have fuel on site.

Section 50.71 Maintenance of records, making of reports.

In § 50.71, this final rule revises paragraph (e) to include NPUFs in the requirement and makes a tense correction to footnote 1. This final rule also revises paragraph (e)(3)(i) and redesignates paragraph (4) as paragraph (4)(i) to clarify that these paragraphs only apply to nuclear power reactors. New paragraphs (e)(3)(iv) and (e)(4)(ii) are added to include the requirements for NPUFs. This final rule also revises paragraph (g) to standardize terminology by replacing the phrase “non-power reactor” with the phrase “non-power production or utilization facility.”

Section 50.75 Reporting and recordkeeping for decommissioning planning.

In § 50.75, this final rule also revises paragraphs (d)(1), (e)(1)(iv), and (f)(4) to standardize terminology by replacing the phrase “non-power reactor” with the phrase “non-power production or utilization facility.” This final rule also revises paragraph (f)(5) by replacing the phrase “non-power reactors” with the phrase “non-power production or utilization facilities.”

Section 50.82 Termination of license.

In § 50.82, this final rule revises paragraph (b) to standardize terminology by replacing the term “reactor” with the phrase “production or utilization facility” and revises paragraph (b)(1) to include testing facilities licensed under § 50.21(c) and holders of a license issued under § 50.22. Paragraph (c) is revised by moving the phrase “that has permanently ceased operation before the expiration of its license” to new paragraph (c)(2) to clarify when the collection period for shortfalls in funding will be determined for NPUFs and holders of licenses issued under § 50.21(b) or § 50.22, or testing facilities.

Section 50.135 Renewal of non-power production or utilization facility licenses issued under § 50.22 and testing facility licenses.

This final rule adds new § 50.135 to clearly define the license renewal process for NPUFs (including testing facilities) licensed under § 50.22 and testing facilities licensed under § 50.21(c).

Appendix C to Part 50—A Guide for the Financial Data and Related Information Required To Establish Financial Qualifications for Construction Permits and Combined Licenses.

In appendix C to part 50, this final rule revises paragraph III by replacing the reference to “medical and research reactors” with a reference to “non-power production or utilization facilities of a type described in § 50.21(a) or (c), other than testing facilities.”

Appendix E to Part 50—Emergency Planning and Preparedness for Production and Utilization Facilities.

In appendix E to part 50, this final rule revises footnote 2 in paragraph I.3 to include the title of Regulatory Guide 2.6 and to replace the phrase “research and test reactor” with the phrase “non-power production or utilization facility.”

Section 51.17 Information collection requirements; OMB approval.

In § 51.17, this final rule revises paragraph (b) to add new § 51.56 as an approved information collection requirement in 10 CFR part 51.

Section 51.45 Environmental report.

In § 51.45, this final rule revises paragraph (a) to add a cross reference to new § 51.56.

Section 51.56 Environmental report – non-power production or utilization facility.

This final rule adds new § 51.56 to clarify existing requirements for the submittal and content of environmental reports by applicants seeking a permit to construct, a license to operate, or a renewal of a license to operate a non-power production or utilization facility.

Section 55.5 Communications.

In § 55.5, this final rule revises paragraph (b)(1) to remove the conditional phrase “except for test and research reactor facilities.” It also revises paragraph (b)(3) to clarify the applicability of this paragraph to utilization facilities licensed under 10 CFR part 50 that are not power reactors.

Section 55.40 Implementation.

In § 55.40, this final rule revises paragraph (d) to replace the phrase “test and research reactors” with the phrase “non-power reactors.”

Section 55.53 Conditions of licenses.

In § 55.53, this final rule revises paragraphs (e) and (f)(2) to replace the phrase “test and research reactors” with the phrase “non-power reactors.” It also revises paragraphs (j) and (k) to clarify that these paragraphs apply to utilization facilities licensed under 10 CFR part 50 that are not power reactors.

Section 55.59 Requalification.

In § 55.59, this final rule revises paragraph (c)(7) to clarify that this paragraph applies to utilization facilities licensed under 10 CFR part 50 that are not power reactors.

Section 55.61 Modification and revocation of licenses.

In § 55.61, this final rule revises paragraph (b)(5) to clarify that this paragraph applies to utilization facilities licensed under 10 CFR part 50 that are not power reactors.

Section 73.2 Definitions.

In § 73.2, this final rule adds the definition of *Non-power reactor* as it is defined in § 50.2.

Section 73.21 Protection of safeguards information: performance requirements.

In § 73.21, this final rule revises paragraph (a)(1)(ii) to replace the phrase “research and test reactors” with the phrase “non-power reactors.”

Section 73.23 Protection of safeguards information—modified handling: specific requirements.

In § 73.23, this final rule replaces the phrase “research and test reactors” with the phrase “non-power reactors.”

Section 73.60 Additional requirements for physical protection at non-power reactors.

In § 73.60, this final rule revises all instances of “nonpower” to read “non-power.”

Section 140.3 Definitions.

In § 140.3, this final rule removes the definition of *Testing reactor* and adds the definition of *Testing facility* as it is defined in § 50.2.

Section 140.11 Amounts of financial protection for certain reactors.

In § 140.11, this final rule revises paragraph (a)(3) to standardize terminology by replacing the term “reactor” with the term “facility.”

Section 170.3 Definitions.

In § 170.3, this final rule revises the definition of *Research reactor* and revises the definition of *Testing facility* to align with the definition in § 50.2.

Section 171.5 Definitions.

In § 171.5, this final rule revises the definitions of *Research reactor* and *Testing facility* to align with the definitions in § 170.3 and § 50.2, respectively.

VI. Regulatory Flexibility Certification

Under the Regulatory Flexibility Act (5 U.S.C. 605(b)), the NRC certifies that this rule does not have a significant economic impact on a substantial number of small entities. This final rule affects only the licensing and operation of NPUFs. In general, the companies, universities, and government agencies that own and operate these facilities do not fall within the scope of the definition of “small entities” set forth in the Regulatory Flexibility Act or the size standards established by the NRC (10 CFR 2.810). Additional information is provided in Section 4 of the regulatory analysis, which is available as indicated in the “Availability of Documents” section of this document.

VII. Regulatory Analysis

The NRC has prepared a final regulatory analysis on this regulation and the implementation guidance. The analysis examines the costs and benefits of the alternatives considered by the NRC. The regulatory analysis is available as indicated in the “Availability of Documents” section of this document.

VIII. Backfitting

The NRC's backfitting regulations for entities that are licensed under 10 CFR part 50 and within the scope of the NRC's backfitting policy appear in § 50.109, "Backfitting." "Backfitting" is defined in § 50.109(a)(1), in relevant part, as a modification of or addition to the systems, structures, components, or design of a facility, or the procedures or organization required to design, construct, or operate a facility, which results from a new or amended provision in the Commission's regulations.

The amendments in this final rule include the following:

- revising the definitions for *Non-power reactor*, *Testing facility*, and *Research reactor*; eliminating license terms for medical therapy or research and development facilities, other than testing facilities, licensed under 10 CFR 50.21(a) or (c);
- defining the license renewal process for all commercial or industrial NPUFs (including testing facilities) licensed under § 50.22 and testing facilities licensed under § 50.21(c) by consolidating existing regulatory requirements in one section of the NRC's regulations; requiring all NPUF licensees to submit an updated FSAR and subsequent FSAR updates to ensure that a facility's licensing basis is kept up-to-date;
- amending the current timely renewal provision under § 2.109, allowing NPUFs subject to license renewal to continue operating under an existing license past its expiration date if the licensee submits a license renewal application at least 2 years (rather than 30 days) before the current license expiration date; providing an accident dose criterion of 1 rem (0.01 Sv) TEDE for NPUFs other than testing facilities, for use in applicants' accident analyses; extending the applicability of § 50.59 to NPUF licensees regardless of their decommissioning status;

- clarifying an NPUF applicant's environmental report requirements in § 51.45; and eliminating the requirement for NPUF licensees to submit financial qualification information with license renewal applications under § 50.33(f)(2).

These amendments do not result in a modification of or addition to the systems, structures, components, or design of a facility, or the procedures or organization required to design, construct, or operate a facility. The final rule changes do not meet the § 50.109(a)(1) definition of "backfitting" and, thus, do not constitute backfitting for any NPUF that may be within the scope of backfitting.

The NRC will clarify whether commercial NPUFs (i.e., NPUFs licensed under Section 103 of the AEA) are within the scope of the NRC's backfitting policy as a general matter through an interpretive rule process. An interpretive rule is an agency's interpretation of a statute or its regulations that does not revise the agency's regulations. Examples of NRC interpretive rules include regulatory guides and notices of interpretation.

As described in the "Availability of Guidance" section of this document, the NRC is issuing Regulatory Guide (RG) 2.7, "Preparation of Updated Final Safety Analysis Reports for Non-Power Production or Utilization Facilities," which provides guidance on methods acceptable to the NRC for complying with the requirements in § 50.71(e) of this final rule. Issuance of this RG does not constitute backfitting under § 50.109. As discussed in the "Implementation" section of RG 2.7, licensees generally are not required to comply with the guidance in that RG. If, in the future, the NRC seeks to impose positions stated in the RG in a manner that would constitute backfitting or forward fitting, the NRC would need to make the showing as required in § 50.109 for backfitting or Management Directive 8.4, "Management of Backfitting, Forward Fitting, Issue Finality, and Information Requests," for forward fitting, that would allow the NRC to impose the positions.

IX. Cumulative Effects of Regulation

Cumulative Effects of Regulation (CER) consists of the challenges licensees may face in addressing the implementation of new regulatory positions, programs, and requirements (e.g., rulemaking, guidance, generic letters, backfits, inspections). The CER may manifest in several ways, including the total burden imposed on licensees by the NRC from simultaneous or consecutive regulatory actions that can adversely affect the licensee's capability to implement those requirements, while continuing to operate or construct its facility in a safe and secure manner.

The goals of the NRC's CER effort were met throughout the development of this final rule. The NRC engaged external stakeholders at public meetings and by soliciting public comments on the proposed rule and associated draft guidance document. A public meeting was held on May 24, 2017, to discuss the proposed rule. A public meeting on implementation was held on April 25, 2019. Summaries of both public meetings are available in ADAMS, as provided in the "Availability of Documents" section of this document. The feedback from the April 25, 2019, public meeting informed the NRC's final rule implementation schedule.

Based upon input from the public and affected licensees, the NRC has specified that this final rule will take effect 30 days from the date of publication of this document. For the purposes of implementing the requirements of § 50.71(e), the NRC will be issuing orders to certain holders of operating licenses, as described in Section II.4 of this document.

X. Plain Writing

The Plain Writing Act of 2010 (Pub. L. 111-274) requires Federal agencies to write documents in a clear, concise, and well-organized manner. The NRC has written this document to be consistent with the Plain Writing Act as well as the Presidential Memorandum, "Plain Language in Government Writing," published June 10, 1998 (63 FR 31885).

XI. Voluntary Consensus Standards

The National Technology Transfer and Advancement Act of 1995, Public Law 104-113, requires agencies to use technical standards developed or adopted by voluntary consensus standards bodies unless the use of such standards is inconsistent with applicable law or is otherwise impractical. The NRC is amending its requirements for the license renewal process for certain production or utilization facilities. This action does not constitute the establishment of a standard that contains generally applicable requirements.

XII. Environmental Assessment and Final Finding of

No Significant Environmental Impact

The Commission has determined under the National Environmental Policy Act of 1969, as amended, and the Commission's regulations in subpart A of 10 CFR part 51, that this final rule will not be a major Federal action significantly affecting the quality of the human environment and, therefore, an environmental impact statement is not required. The provision to eliminate license terms for NPUFs, other than testing facilities, licensed under § 50.21(a) or (c) will result in no additional radiological or non-radiological impacts because of the minimal accident consequences of these

facilities, existing surveillance and reporting by licensees, and NRC oversight. In addition, the implementation of this final rule will not affect the environmental review requirements for new facilities and facilities applying for license renewal. The NRC concludes that this final rule will not cause any additional radiological or non-radiological impacts on the human environment.

The NRC requested the views of the States on the environmental assessment for this rule. No States filed comments regarding the environmental assessment for this rule.

The determination of this environmental assessment is that there will be no significant offsite impact to the public from this action. The environmental assessment is available as indicated under the "Availability of Documents" section.

XIII. Paperwork Reduction Act

This final rule contains new or amended collections of information subject to the Paperwork Reduction Act of 1995 (44 U.S.C. 3501, et seq.). The collections of information were approved by the Office of Management and Budget (OMB), approval number 3150-0268.

The burden to the public for the information collections is estimated to average 51 hours per response for information collection requirements contained in 10 CFR part 50 and 0 hours per response for information collection requirements contained in 10 CFR part 51, including the time for reviewing instructions, searching existing data sources, gathering and maintaining the data needed, and completing and reviewing the information collections.

The information collections are being conducted to create a more efficient licensing process that continues to protect public health, safety, and the

environment. Information will be used by the NRC to ensure that licensing bases remain up-to-date and that adequate protection of public health and safety is maintained. Responses to these collections of information are mandatory under § 50.71(e) and § 51.56. Confidential and proprietary information submitted to the NRC is protected in accordance with NRC regulations at § 9.17(a) and § 2.390(b).

You may submit comments on any aspect of the information collections, including suggestions for reducing the burden, by the following methods:

- Federal rulemaking Web Site: Go to <http://www.regulations.gov> and search for Docket ID NRC-2011-0087.
- Mail comments to: FOIA, Library, and Information Collections Branch, Office of the Chief Information Officer, Mail Stop: T-6 A10M, U.S. Nuclear Regulatory Commission, Washington, DC 20555-0001; InfoCollects@nrc.gov; or to the OMB reviewer at: OMB Office of Information and Regulatory Affairs (3150-0268), Attn: Desk Officer for the Nuclear Regulatory Commission, 725 17th Street, NW Washington, DC 20503.

Public Protection Notification

The NRC may not conduct or sponsor, and a person is not required to respond to, a collection of information unless the document requesting or requiring the collection displays a currently valid OMB control number.

XIV. Congressional Review Act

This final rule is a rule as defined in the Congressional Review Act (5 U.S.C. 801-808). However, the Office of Management and Budget has not found it to be a major rule as defined in the Congressional Review Act.

XV. Criminal Penalties

For the purposes of Section 223 of the AEA, the NRC is issuing this final rule that amends 10 CFR 50.34, 50.36, 50.59, 50.71, 50.75, 50.82, 55.40, 55.53, 55.59, 73.21, 73.23, 73.60, and 140.11 and creates § 50.135 under one or more of Sections 161b, 161i, or 161o of the AEA. Willful violations of these provisions would be subject to criminal enforcement.

XVI. Availability of Guidance

The NRC is issuing RG 2.7, Revision 0, "Preparation of Updated Final Safety Analysis Reports for Non-Power Production or Utilization Facilities," for the implementation of the requirements in § 50.71(e) of this final rule. The guidance is available in ADAMS under Accession No. ML18031A007. You can access information and public comment submissions related to the guidance at the federal rulemaking Web Site, www.regulations.gov, by searching on Docket ID NRC-2011-0087.

XVII. Availability of Documents

The documents identified in the following table are available to interested persons through one or more of the following methods, as indicated.

Document	ADAMS Accession No. / Web link / FEDERAL REGISTER CITATION
NUREG-1537, Part 1, "Guidelines for Preparing and Reviewing Applications for the Licensing of Non-Power Reactors, Format and Content"	ML042430055
NUREG-1537, Part 2, "Guidelines for Preparing and Reviewing Applications for the Licensing of Non-Power Reactors, Standard Review Plan and Acceptance Criteria"	ML042430048
Interim Staff Guidance-2009-001, "Interim Staff Guidance on the Streamlined Review Process for License Renewal for Research Reactors"	ML092240244
Non-Power Reactor License Renewal: Preliminary Draft Regulatory Basis; Request for Comment	77 FR 38742; June 29, 2012
Non-Power Reactor (NPR) License Renewal Rulemaking: Regulatory Basis Document	ML12240A677
<i>Federal Register</i> Notice: Final Regulatory Basis for Rulemaking to Streamline Non-Power Reactor License Renewal; Notice of Availability of Documents	ML12250A658
SECY-08-0161, "Review of Research and Test Reactor License Renewal Applications"	ML082550140
SRM-SECY-08-0161, "Review of Research and Test Reactor License Renewal Applications"	ML090850159
SRM-M080317B, "Briefing on State of NRC Technical Programs"	ML080940439
SECY-09-0095, "Long-Term Plan for Enhancing the Research and Test Reactor License Renewal Process and Status of the Development and Use of the Interim Staff Guidance"	ML092150717
SRM-SECY-91-061, "Separation of Non-Reactor and Non-Power Reactor Licensing Activities from Power Reactor Licensing Activities in 10 CFR Part 50"	ML010050021
SRM-M090811, "Briefing on Research and Test Reactor (RTR) Challenges"	ML092380046
Draft Regulatory Guide DG-2006, "Preparation of Updated Final Safety Analysis Reports for Non-Power Production or Utilization Facilities"	ML17068A041
Proposed Rule: Draft Regulatory and Backfit Analysis	ML17068A038
Proposed Rule: Draft OMB Supporting Statement	ML17068A077

Document	ADAMS Accession No. / Web link / FEDERAL REGISTER CITATION
Proposed Rule: Draft Environmental Assessment	ML17068A035
SECY-16-0048, "Proposed Rulemaking: Non-Power Production or Utilization Facility License Renewal (RIN 3150-AI96)"	ML16019A048
EPA 400-R-92-001, "Manual of Protective Action Guides and Protective Actions for Nuclear Incidents"	https://www.epa.gov/sites/production/files/2016-03/documents/pags.pdf
EPA-400/R-17/001, "PAG Manual: Protective Action Guides and Planning Guidance for Radiological Incidents"	https://www.epa.gov/sites/production/files/2017-01/documents/epa_pag_manual_final_revisions_01-11-2017_cover_disclaimer_8.pdf
Summary of August 7, 2014, Public Meeting to Discuss the Rulemaking for Streamlining Non-power Reactor License Renewal	ML15322A400
Summary of October 7, 2015, Public Meeting to Discuss the Rulemaking for Streamlining Non-Power Reactor License Renewal	ML15307A002
<i>Federal Register</i> Notice: Final Rule; Standards for Protection Against Radiation	56 FR 23360; May 21, 1991
<i>Federal Register</i> Notice: Proposed Rule; Non-Power Production or Utilization Facility License Renewal	82 FR 15643; March 30, 2017
SRM-SECY-16-0048, "Staff Requirements—Proposed Rulemaking: Non-Power Production or Utilization Facility License Renewal (RIN 3150-AI96)"	ML17045A543
"Supporting Statement For Information Collections Contained In 10 CFR Part 50 Non-Power Production Or Utilization Facility License Renewal Final Rule," dated December 2024	ML18031A006
"Supporting Statement For Information Collections Contained In 10 CFR Part 51 Non-Power Production Or Utilization Facility License Renewal Final Rule," dated December 2024	ML19113A007
"Environmental Assessment and Finding of No Significant Impact Supporting Final Rule: Non-Power Production or Utilization Facility License Renewal," dated December 2024	ML24241A112
Final Rule: "Regulatory Analysis - Non-power Production or Utilization Facility License Renewal," dated December 2024	ML24241A114
"NRC Response to Public Comments; Non-Power Production or Utilization Facility License Renewal," dated December 2024	ML18031A005

Document	ADAMS Accession No. / Web link / FEDERAL REGISTER CITATION
Regulatory Guide 2.7, "Preparation of Updated Final Safety Analysis Reports for Non-Power Production or Utilization Facilities," dated December 2024	ML18031A007
<i>Federal Register</i> Notice: Final Rule; Financial Information Requirements for Applications to Renew or Extend the Term of an Operating License for a Power Reactor	69 FR 4439; January 30, 2004
Summary of May 24, 2017, Public Meeting to Discuss the Proposed Non-Power Production or Utilization Facility License Renewal Rule	ML17170A066
Nuclear Energy Innovation and Modernization Act (Pub. L. 115-439), enacted January 14, 2019	https://www.congress.gov/115/bills/s512/BILLS-115s512enr.pdf
Summary of April 25, 2019, Public Meeting to Discuss the Implementation Schedule for the Non-Power Production or Utilization Facility License Renewal Final Rule	ML19133A080
NRC Response to Public Comment Non-Power Production or Utilization Facility License Renewal	ML18031A005
SECY-19-0062, "Final Rule: Non-Power Production or Utilization Facility License Renewal (RIN 3150-AI96, NRC-2011-0087)"	ML18031A000 (package)
SRM-M240904: Affirmation Session - SECY-19-0062, "Final Rule: Non-Power Production or Utilization Facility License Renewal (RIN 3150-AI96, NRC-2011-0087)", dated September 4, 2024	ML24248A208 (package)
<i>Federal Register</i> Notice: Final Rule; 10 CFR Part 50 – Licensing of Production and Utilization Facilities	33 FR 9704; July 4, 1968
<i>Federal Register</i> Notice: Final Rule; Elimination of Review of Financial Qualifications of Electric Utilities in Licensing Hearings for Nuclear Power Plants	47 FR 13750; March 31, 1982
<i>Federal Register</i> Notice: Final Rule; Elimination of Review of Financial Qualifications of Electric Utilities in Operating License Reviews and Hearings for Nuclear Power Plants	49 FR 35747; September 12, 1984
<i>Federal Register</i> Notice: Final Rule; National Environmental Policy Act—Regulations	43 FR 55978; November 29, 1978
Advanced Notice of Proposed Rulemaking; Revision of Backfitting Process for Power Reactors	48 FR 44217; September 28, 1983
Policy Statement; Revision of Backfitting Process for Power Reactors	48 FR 44173; September 28, 1983

Document	ADAMS Accession No. / Web link / FEDERAL REGISTER CITATION
<i>Federal Register</i> Notice: Proposed Rule; Revision of Backfitting Process for Power Reactors	49 FR 47034; November 30, 1984
<i>Federal Register</i> Notice: Final Rule; Revision of Backfitting Process for Power Reactors	50 FR 38097; September 20, 1985
<i>Federal Register</i> Notice: Final Rule; Financial Information Requirements for Applications to Renew or Extend the Term of an Operating License for a Power Reactor	69 FR 4439; January 30, 2004
<i>Federal Register</i> Notice: Proposed Rule; Revision of Backfitting Process for Power Reactors	52 FR 34223; September 10, 1987
<i>Federal Register</i> Notice: Final Rule; Revision of Backfitting Process for Power Reactors	53 FR 20603; June 6, 1988
<i>Federal Register</i> Notice: Final Rule; Limiting the Use of Highly Enriched Uranium in Domestically Licensed Research and Test Reactors	51 FR 6514; February 25, 1986
<i>Federal Register</i> Notice: Final Rule; Clarification of Physical Protection Requirements at Fixed Sites	58 FR 13699; March 15, 1993
<i>Federal Register</i> Notice: Final Rule; Requirements for Fingerprint-Based Criminal History Record Checks for Individuals Seeking Unescorted Access to Non-Power Reactors	77 FR 27561, 27572; May 11, 2012
Plain Language in Government Writing	63 FR 31885; June 10, 1998

List of Subjects

10 CFR Part 2

Administrative practice and procedure, Antitrust, Byproduct material, Classified information, Confidential business information; Freedom of information, Environmental protection, Hazardous waste, Nuclear energy, Nuclear materials, Nuclear power plants and reactors, Penalties, Reporting and recordkeeping requirements, Sex discrimination, Source material, Special nuclear material, Waste treatment and disposal.

10 CFR Part 20

Byproduct material, Criminal penalties, Hazardous waste, Licensed material, Nuclear energy, Nuclear materials, Nuclear power plants and reactors, Occupational safety and health, Packaging and containers, Penalties, Radiation protection, Reporting and recordkeeping requirements, Source material, Special nuclear material, Waste treatment and disposal.

10 CFR Part 26

Administrative practice and procedure, Alcohol abuse, Alcohol testing, Appeals, Chemical testing, Drug abuse, Drug testing, Employee assistance programs, Fitness for duty, Management actions, Nuclear power plants and reactors, Privacy, Protection of information, Radiation protection, Reporting and recordkeeping requirements.

10 CFR Part 50

Administrative practice and procedure, Antitrust, Classified information, Criminal penalties, Education, Fire prevention, Fire protection, Incorporation by reference, Intergovernmental relations, Nuclear power plants and reactors, Penalties, Radiation protection, Reactor siting criteria, Reporting and recordkeeping requirements, Whistleblowing.

10 CFR Part 51

Administrative practice and procedure, Environmental impact statements, Hazardous waste, Nuclear energy, Nuclear materials, Nuclear power plants and reactors, Reporting and recordkeeping requirements.

10 CFR Part 55

Criminal penalties, Manpower training programs, Nuclear power plants and reactors, Penalties, Reporting and recordkeeping requirements.

10 CFR Part 73

Criminal penalties, Exports, Hazardous materials transportation, Incorporation by reference, Imports, Nuclear energy, Nuclear materials, Nuclear power plants and reactors, Penalties, Reporting and recordkeeping requirements, Security measures.

10 CFR Part 140

Criminal penalties, Extraordinary nuclear occurrence, Insurance, Intergovernmental relations, Nuclear materials, Nuclear power plants and reactors, Penalties, Reporting and recordkeeping requirements.

10 CFR Part 170

Byproduct material, Import and export licenses, Intergovernmental relations, Non-payment penalties, Nuclear energy, Nuclear materials, Nuclear power plants and reactors, Source material, Special nuclear material.

10 CFR Part 171

Annual charges, Byproduct material, Holders of certificates, registrations, approvals, Intergovernmental relations, Nonpayment penalties, Nuclear materials, Nuclear power plants and reactors, Source material, Special nuclear material.

For the reasons set out in the preamble and under the authority of the AEA, as amended; the Energy Reorganization Act of 1974, as amended; and 5 U.S.C. 552 and

553, the NRC is adopting the following amendments to 10 CFR parts 2, 20, 26, 50, 51, 55, 73, 140, 170, and 171:

PART 2 -- AGENCY RULES OF PRACTICE AND PROCEDURE

1. The authority citation for part 2 continues to read as follows:

Authority: Atomic Energy Act of 1954, secs. 29, 53, 62, 63, 81, 102, 103, 104, 105, 161, 181, 182, 183, 184, 186, 189, 191, 234 (42 U.S.C. 2039, 2073, 2092, 2093, 2111, 2132, 2133, 2134, 2135, 2201, 2231, 2232, 2233, 2234, 2236, 2239, 2241, 2282); Energy Reorganization Act of 1974, secs. 201, 206 (42 U.S.C. 5841, 5846); Nuclear Waste Policy Act of 1982, secs. 114(f), 134, 135, 141 (42 U.S.C. 10134(f), 10154, 10155, 10161); Administrative Procedure Act (5 U.S.C. 552, 553, 554, 557, 558); National Environmental Policy Act of 1969 (42 U.S.C. 4332); 44 U.S.C. 3504 note. Section 2.205(j) also issued under 28 U.S.C. 2461 note.

2. In § 2.109, revise paragraph (a) and add paragraph (f) to read as follows:

§ 2.109 Effect of timely renewal application.

(a) Except for the renewal of licenses identified in paragraphs (b) through (f) of this section, if at least 30 days before the expiration of an existing license authorizing any activity of a continuing nature, the licensee files an application for a renewal or for a new license for the activity so authorized, the existing license will not be deemed to have expired until the application has been finally determined.

* * * * *

(f) If the licensee of a non-power production or utilization facility licensed under 10 CFR 50.22, or a testing facility, files a sufficient application for renewal at least 2 years before the expiration of the existing license, the existing license will not be deemed to have expired until the application has been finally determined.

PART 20 -- STANDARDS FOR PROTECTION AGAINST RADIATION

3. The authority citation for part 20 continues to read as follows:

Authority: Atomic Energy Act of 1954, secs. 11, 53, 63, 65, 81, 103, 104, 161, 170H, 182, 186, 223, 234, 274, 1701 (42 U.S.C. 2014, 2073, 2093, 2095, 2111, 2133, 2134, 2201, 2210h, 2232, 2236, 2273, 2282, 2021, 2297f); Energy Reorganization Act of 1974, secs. 201, 202 (42 U.S.C. 5841, 5842); Low-Level Radioactive Waste Policy Amendments Act of 1985, sec. 2 (42 U.S.C. 2021b); 44 U.S.C. 3504 note.

§ 20.1905 [Amended]

4. In § 20.1905(g), remove the word “reactors” and add in its place the phrase “production or utilization facilities”.

PART 26 -- FITNESS FOR DUTY PROGRAMS

5. The authority citation for part 26 continues to read as follows:

Authority: Atomic Energy Act of 1954, secs. 53, 103, 104, 107, 161, 223, 234, 1701 (42 U.S.C. 2073, 2133, 2134, 2137, 2201, 2273, 2282, 2297f); Energy Reorganization Act of 1974, secs. 201, 202 (42 U.S.C. 5841, 5842); 44 U.S.C. 3504 note.

§ 26.3 [Amended]

6. In § 26.3(e), remove the word “reactor” and add in its place the phrase “production or utilization facility”.

PART 50 -- DOMESTIC LICENSING OF PRODUCTION AND UTILIZATION FACILITIES

7. The authority citation for part 50 continues to read as follows:

Authority: Atomic Energy Act of 1954, secs. 11, 101, 102, 103, 104, 105, 108, 122, 147, 149, 161, 181, 182, 183, 184, 185, 186, 187, 189, 223, 234 (42 U.S.C. 2014, 2131, 2132, 2133, 2134, 2135, 2138, 2152, 2167, 2169, 2201, 2231, 2232, 2233, 2234, 2235, 2236, 2237, 2239, 2273, 2282); Energy Reorganization Act of 1974, secs. 201, 202, 206, 211 (42 U.S.C. 5841, 5842, 5846, 5851); Nuclear Waste Policy Act of 1982, sec. 306 (42 U.S.C. 10226); National Environmental Policy Act of 1969 (42 U.S.C. 4332); 44 U.S.C. 3504 note; Sec. 109, Pub. L. 96–295, 94 Stat. 783.

8. In § 50.2, revise the definitions for *Non-power reactor* and *Testing facility* to read as follows:

§ 50.2 Definitions.

* * * * *

Non-power reactor means:

(1) A testing facility; or

(2) A research reactor, which is a non-power production or utilization facility that is a nuclear reactor licensed under § 50.21(c):

(i) For which a safety assessment demonstrates accident radiation doses consistent with § 50.34(a)(1)(i); and

(ii) That is not a testing facility; or

(3) A commercial or industrial reactor, which is a non-power production or utilization facility that is a nuclear reactor licensed under § 50.22:

(i) For which a safety assessment demonstrates accident radiation doses consistent with § 50.34(a)(1)(i); and

(ii) That is not a testing facility.

* * * * *

Testing facility means a non-power production or utilization facility that is a nuclear reactor licensed under § 50.21(c) or § 50.22 for which:

(1) Analyzed accident radiation doses are in excess of the dose criterion for facilities not subject to 10 CFR part 100 set forth in § 50.34(a)(1)(i); or

(2) The Commission determines that the design, operation, or use and the associated risk warrant classification as a testing facility.

* * * * *

§ 50.8 [Amended]

9. In § 50.8(b), add the number “50.135,” in numerical order.

§ 50.33 [Amended]

10. In § 50.33:

a. Remove the phrase “for a power reactor” from the fourth sentence and remove the last sentence in paragraph (f)(2); and

b. Redesignate footnotes 4 and 5 as footnotes 1 and 2.

11. In § 50.34:

a. Redesignate footnote 5 as footnote 1;

b. Revise paragraph (a)(1)(i);

c. Redesignate footnotes 6 and 7 as footnotes 3 and 4;

d. Remove footnotes 8 and 9;

f. Redesignate footnotes 10 and 11 as footnotes 5 and 6; and

g. Add paragraph (b)(13).

The revision and addition read as follows:

§ 50.34 Contents of applications; technical information.

(a) * * *

(1) * * *

(i) A description and safety assessment of the site on which the facility is to be located, with appropriate attention to features affecting facility design. Special attention should be directed to the site evaluation factors identified in part 100 of this chapter. The assessment must contain an analysis and evaluation of the major structures, systems and components of the facility which bear significantly on the acceptability of the site under the site evaluation factors identified in part 100 of this chapter, assuming that the facility will be operated at the ultimate power level which is contemplated by the applicant. For non-power production or utilization facilities not subject to 10 CFR part 100, the assessment must provide an evaluation of the applicable radiological consequences that demonstrates with reasonable assurance that any individual located in the unrestricted area following the onset of a postulated accident, including consideration of experiments, would not receive a radiation dose in excess of 1 rem (0.01 Sv)² TEDE for the duration of the accident. With respect to operation at the projected initial power level, the applicant is required to submit information prescribed in paragraphs (a)(2) through (a)(8) of this section, as well as the information required by this paragraph, in support of the application for a construction permit, or a design approval.

* * * * *

(b) * * *

(13) Non-power production or utilization facility applicants who apply for an initial or renewed operating license shall provide a final evaluation of the applicable radiological consequences in § 50.34(a)(1)(i).

* * * * *

² The 1 rem accident dose criterion for non-power production or utilization facilities is not a dose limit; it informs the analysis of postulated accidents and the development of safety measures so that in the unlikely event of an accident, the NRC has reasonable assurance that no acute radiation-related harm will result to any member of the public.

§ 50.36 [Amended]

12. In § 50.36(c)(6), remove the phrase “non-power reactor” and add in its place the phrase “non-power production or utilization”.

13. In § 50.51, in the first sentence of paragraph (a) remove the word “Each” and add in its place the phrase “Except as noted in § 50.51(c), each” and add paragraph (c) to read as follows:

§ 50.51 Continuation of license.

* * * * *

(c) Each non-power production or utilization facility license issued under § 50.21(a) or (c), other than a testing facility license, after January 25, 2025, will be issued with no fixed license term.

14. In § 50.59, revise paragraph (b) to read as follows:

§ 50.59 Changes, tests, and experiments.

* * * * *

(b) This section applies to each holder of an operating license issued under this part or a combined license issued under part 52 of this chapter, including the holder of a license authorizing the operation of a nuclear power reactor that has submitted the certification of permanent cessation of operations required under § 50.82(a)(1) or § 50.110, a reactor licensee whose license has been amended to allow possession of

nuclear fuel but not operation of the facility, or a non-power production or utilization facility that has permanently ceased operations.

* * * * *

15. In § 50.71:

a. In the first sentence of paragraph (e), add the phrase “, or non-power production or utilization facility,” after the word “reactor”, and in footnote 1, remove the word “includes” and add in its place the word “include”;

b. In paragraph (e)(3)(i), remove the article “A” at the beginning and add in its place the phrase “For nuclear power reactor licensees, a” and add paragraph (e)(3)(iv);

c. Redesignate paragraph (e)(4) as paragraph (e)(4)(i) and remove the word “Subsequent” and add in its place the phrase “For nuclear power licensees, subsequent”; add paragraph (e)(4)(ii); and

d. In paragraph (g), remove the phrase “non-power reactor” and add in its place the phrase “non-power production or utilization facility”.

The additions and revisions read as follows:

§ 50.71 Maintenance of records, making of reports.

* * * * *

(e) * * *

(3) * * *

(iv) Holders of non-power production or utilization facility licenses issued after January 29, 2025, shall file a revision of the original FSAR containing those original pages that are still applicable plus new replacement pages within 5 years of the date of issuance of the operating license. The revision must bring the FSAR up to date as of a

maximum of 6 months prior to the date of filing the revision.

* * * * *

(4) * * *

(ii) Non-power production or utilization facility licensees shall file an FSAR update no more than 5 years from the date of the submittal of the updated FSAR required by § 50.71(e)(3)(iv) or by order and shall file subsequent updates no more than 5 years from the date of the previous submittal. Each submittal must reflect all changes made to the FSAR up to a maximum of 6 months prior to the date of filing the submittal.

* * * * *

16. In § 50.75:

a. Revise paragraph (d)(1);

b. In paragraphs (e)(1)(iv) and (f)(4), remove the phrase “non-power reactor” and add in its place the phrase “non-power production or utilization facility”; and

c. In paragraph (f)(5), remove the phrase “power and non-power reactors” and add in its place the phrase “power reactors and non-power production or utilization facilities”.

The revision reads as follows:

§ 50.75 Reporting and recordkeeping for decommissioning planning.

* * * * *

(d)(1) Each applicant for or holder of an operating license for a non-power production or utilization facility shall submit a decommissioning report as required by § 50.33(k) of this part.

* * * * *

17. In § 50.82, revise paragraphs (b) introductory text, (b)(1), and (c) to read as follows:

§ 50.82 Termination of license.

* * * * *

(b) For non-power production or utilization facility licensees—

(1) A licensee that permanently ceases operations must make application for license termination within 2 years following permanent cessation of operations, and for testing facilities licensed under § 50.21(c) or facilities licensed under § 50.22, in no case later than 1 year prior to expiration of the operating license. Each application for termination of a license must be accompanied or preceded by a proposed decommissioning plan. The contents of the decommissioning plan are specified in paragraph (b)(4) of this section.

* * * * *

(c) The collection period for any shortfall of funds will be determined, upon application by the licensee, on a case-by-case basis taking into account the specific financial situation of each holder of the following licenses:

(1) A non-power production or utilization facility licensed under § 50.21(a) or (c), other than a testing facility, that has permanently ceased operations.

(2) A facility licensed under § 50.21(b) or § 50.22, or a testing facility, that has permanently ceased operation before the expiration of its license.

18. Add § 50.135 to read as follows:

§ 50.135 Renewal of non-power production or utilization facility licenses issued under § 50.22 and testing facility licenses.

(a) *Applicability.* The requirements in this section apply to applicants for renewed non-power production or utilization facility operating licenses issued under § 50.22 and to applicants for renewed testing facility operating licenses issued under § 50.21(c).

(b) *Written communications.* All applications, correspondence, reports, and other written communications must be filed in accordance with applicable portions of § 50.4.

(c) *Filing of application.*

(1) The filing of an application for a renewed license must be in accordance with subpart A of 10 CFR part 2 and all applicable sections of this part.

(2) An application for a renewed license may not be submitted to the Commission earlier than 10 years before the expiration of the operating license currently in effect.

(d) *Contents of application.*

(1) Each application must include the information specified in §§ 50.33, 50.34, and 50.36, as applicable.

(2) Each application must include conforming changes to the standard indemnity agreement, under 10 CFR part 140 to account for the expiration term of the proposed renewed license.

(3) Each application must include a supplement to the environmental report that complies with the requirements of 10 CFR 51.56.

(e) *Issuance of a renewed license.*

(1) A renewed license will be of the class for which the operating license currently in effect was issued.

(2) A renewed license will be issued for a fixed period of time. The term of any renewed license may not exceed 40 years.

(3) A renewed license will become effective immediately upon its issuance, thereby superseding the operating license previously in effect. If a renewed license is subsequently set aside upon further administrative or judicial appeal, the operating

license previously in effect will be reinstated unless its term has expired and the renewal application was not filed in a timely manner in accordance with 10 CFR 2.109.

(4) A renewed license may be subsequently renewed in accordance with all applicable requirements.

Appendix C to Part 50 [Amended]

19. In paragraph III of appendix C to part 50, remove the phrase “for medical and research reactors” and add in its place the phrase “for non-power production or utilization facilities of a type described in § 50.21(a) or (c), other than testing facilities”.

20. In paragraph I.3 to appendix E to part 50, revise footnote 2 to read as follows:

APPENDIX E TO PART 50—EMERGENCY PLANNING AND PREPAREDNESS FOR PRODUCTION AND UTILIZATION FACILITIES

* * * * *

² Regulatory Guide 2.6, “Emergency Planning for Research and Test Reactors and Other Non-Power Production and Utilization Facilities,” may be used as guidance for the acceptability of non-power production or utilization facility emergency response plans.

* * * * *

PART 51 -- ENVIRONMENTAL PROTECTION REGULATIONS FOR DOMESTIC LICENSING AND RELATED REGULATORY FUNCTIONS

21. The authority citation for part 51 continues to read as follows:

Authority: Atomic Energy Act of 1954, secs. 161, 193 (42 U.S.C. 2201, 2243); Energy Reorganization Act of 1974, secs. 201, 202 (42 U.S.C. 5841, 5842); National Environmental Policy Act of 1969 (42 U.S.C. 4332, 4334, 4335); Nuclear Waste Policy

Act of 1982, secs. 144(f), 121, 135, 141, 148 (42 U.S.C. 10134(f), 10141, 10155, 10161, 10168); 44 U.S.C. 3504 note.

§ 51.17 [Amended]

22. In § 51.17(b), add the number “51.56,” in numerical order.

§ 51.45 [Amended]

23. In § 51.45(a), add the number “51.56,” in numerical order.

24. Add § 51.56 to read as follows:

§ 51.56 Environmental report—non-power production or utilization facility.

Each applicant for a non-power production or utilization construction permit or facility license, or renewal of a non-power production or utilization facility license issued pursuant to § 50.21(a) or (c) or § 50.22 of this chapter shall submit a separate document, entitled “Applicant's Environmental Report” or “Supplement to Applicant's Environmental Report,” as appropriate, with its application to: ATTN: Document Control Desk, Director, Office of Nuclear Reactor Regulation. The environmental report or supplement shall contain the information specified in § 51.45. If the application is for a renewal of a license for which the applicant has previously submitted an environmental report, the supplement, to the extent applicable, shall include an analysis of any environmental impacts resulting from operational experience or a change in operations, and an analysis of any environmental impacts that may result from proposed decommissioning activities.

PART 55 -- OPERATORS' LICENSES

25. The authority citation for part 55 continues to read as follows:

Authority: Atomic Energy Act of 1954, secs. 107, 161, 181, 182, 183, 186, 187,

223, 234 (42 U.S.C. 2137, 2201, 2231, 2232, 2233, 2236, 2237, 2273, 2282); Energy Reorganization Act of 1974, secs. 201, 202 (42 U.S.C. 5841, 5842); Nuclear Waste Policy Act of 1982, sec. 306 (42 U.S.C. 10226); 44 U.S.C. 3504 note.

§ 55.5 [Amended]

26. In § 55.5:

a. In paragraph (b)(1) remove the phrase “Except for test and research reactor facilities, the” and add in its place the word “The”; and

b. In paragraph (b)(3) remove the phrase “a test and research reactor or non-power reactor facility licensed under 10 CFR part 50” and add in its place “a utilization facility licensed under part 50 of this chapter that is not a power reactor”.

§ 55.40 [Amended]

27. In § 55.40(d), remove the phrase “all test and research reactors” and add in its place the phrase “all non-power reactors”.

§ 55.53 [Amended]

28. In § 55.53:

a. In paragraphs (e) and (f)(2), remove the phrase “test and research reactors” and add in its place the phrase “non-power reactors”; and

b. In paragraphs (j) and (k), remove the phrase “non-power reactors” and add in its place the phrase “utilization facilities licensed under 10 CFR part 50 that are not power reactors”, and in paragraph (k) remove the term “non-power” at the end of the paragraph.

§ 55.59 [Amended]

29. In § 55.59(c)(7), in the paragraph heading remove the phrase “*research and test reactor facilities*” and add in its place the phrase “*utilization facilities licensed under 10 CFR part 50 that are not power reactors*”, and in the paragraph remove the phrase “research reactor or test reactor facility” and add in its place “utilization facility licensed under 10 CFR part 50 that is not a power reactor”.

§ 55.61 [Amended]

30. In § 55.61(b)(5), remove the phrase “non-power reactors” and add in its place the phrase “utilization facilities licensed under 10 CFR part 50 that are not power reactors”.

PART 73 -- PHYSICAL PROTECTION OF PLANTS AND MATERIALS

31. The authority citation for part 73 continues to read as follows:

Authority: Atomic Energy Act of 1954, secs. 53, 147, 149, 161, 170D, 170E, 170H, 170I, 223, 229, 234, 1701 (42 U.S.C. 2073, 2167, 2169, 2201, 2210d, 2210e, 2210h, 2210i, 2273, 2278a, 2282, 2297f); Energy Reorganization Act of 1974, secs. 201, 202 (42 U.S.C. 5841, 5842); Nuclear Waste Policy Act of 1982, secs. 135, 141 (42 U.S.C. 10155, 10161); 44 U.S.C. 3504 note. Section 73.37(b)(2) also issued under Sec. 301, Public Law 96-295, 94 Stat. 789 (42 U.S.C. 5841 note).

32. In § 73.2, add in alphabetical order the definition for *Non-power reactor*.

§ 73.2 Definitions.

* * * * *

Non-power reactor is defined at 10 CFR 50.2.

* * * * *

§ 73.21 [Amended]

33. In § 73.21(a)(1)(ii), remove the phrase “Research and test reactors” and add in its place the phrase “non-power reactors”.

§ 73.23 [Amended]

34. In § 73.23, remove the phrase “research and test reactors” and add in its place the phrase “non-power reactors”.

§ 73.60 [Amended]

35. In § 73.60, wherever it may appear, remove the word “nonpower” and add in its place the word “non-power”.

PART 140 -- FINANCIAL PROTECTION REQUIREMENTS AND INDEMNITY

AGREEMENTS

36. The authority citation for part 140 continues to read as follows:

Authority: Atomic Energy Act of 1954, secs. 161, 170, 223, 234 (42 U.S.C. 2201, 2210, 2273, 2282); Energy Reorganization Act of 1974, secs. 201, 202 (42 U.S.C. 5841, 5842); 44 U.S.C. 3504 note.

37. In § 140.3, remove the definition for *Testing reactor* and revise the definition for *Testing facility* to read as follows:

§ 140.3 Definitions.

* * * * *

Testing facility is defined at 10 CFR 50.2.

* * * * *

§ 140.11 [Amended]

38. In § 140.11(a)(3), remove the phrase “testing reactor” and add in its place the phrase “testing facility”.

**PART 170 -- FEES FOR FACILITIES, MATERIALS, IMPORT AND EXPORT
LICENSES, AND OTHER REGULATORY SERVICES UNDER THE ATOMIC ENERGY
ACT OF 1954, AS AMENDED**

39. The authority citation for part 170 continues to read as follows:

Authority: Atomic Energy Act of 1954, secs. 11, 161(w) (42 U.S.C. 2014, 2201(w)); Energy Reorganization Act of 1974, sec. 201 (42 U.S.C. 5841); 42 U.S.C. 2214; 31 U.S.C. 901, 902, 9701; 44 U.S.C. 3504 note.

40. In § 170.3, revise the definitions for *Research reactor* and *Testing facility* to read as follows:

§ 170.3 Definitions.

* * * * *

Research reactor means a non-power production or utilization facility, as defined in 10 CFR 50.2, that is a nuclear reactor licensed under 10 CFR 50.21(c):

- (i) For which a safety assessment demonstrates accident radiation doses consistent with 10 CFR 50.34(a)(1)(i); and
- (ii) That is not a testing facility.

* * * * *

Testing facility is defined at 10 CFR 50.2.

* * * * *

**PART 171 -- ANNUAL FEES FOR REACTOR LICENSES AND FUEL CYCLE
LICENSES AND MATERIALS LICENSES, INCLUDING HOLDERS OF
CERTIFICATES OF COMPLIANCE, REGISTRATIONS, AND QUALITY ASSURANCE
PROGRAM APPROVALS AND GOVERNMENT AGENCIES LICENSED BY THE NRC**

41. The authority citation for part 171 continues to read as follows:

Authority: Atomic Energy Act of 1954, secs. 11, 161(w), 223, 234 (42 U.S.C. 2014, 2201(w), 2273, 2282); Energy Reorganization Act of 1974, sec. 201 (42 U.S.C. 5841); 42 U.S.C. 2214; 44 U.S.C. 3504 note.

42. In § 171.5, revise the definitions for *Research reactor* and *Testing facility* to read as follows:

§ 171.5 Definitions.

* * * * *

Research reactor is defined at 10 CFR 170.3.

* * * * *

Testing facility is defined at 10 CFR 50.2.

* * * * *

Dated: December 19, 2024.

For the Nuclear Regulatory Commission.

/RA/

Carrie Safford,
Secretary of the Commission.